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SUBJECT: Provides actions planned for improving performance of
 reactor bldg post-accident grab sampling sys (REA-SR-48),per
 insp rept 50-397/90-29.State-of-art in-line post-accident
 monitoring sys to be installed in elevated release duct.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

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October 18, 1991
G02-91-193

Docket No. 50-397

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C.

Gentlemen:

Subject: NUCLEAR PLANT NO. 2, LICENSE NO. NPF-21
NRC INSPECTION REPORTS 85-20 AND 90-29
REACTOR BUILDING POST-LOCA GRAB SAMPLER (REA-SR-48)

Reference: Letter, G02-91-012, GD Bouchey (SS) to NRC
"NRC Inspection Report 90-29 - Response to Notice of
Violation", Appendix B, dated January 23, 1991.

The purpose of this letter is to provide the actions planned for improving the performance of the Reactor Building Post-Accident Grab Sampling System (REA-SR-48). In Inspection Report 90-29, the NRC followed up on a previous concern (Inspection Report 85-20) pertaining to the ability of REA-SR-48 to quantify post-accident radioiodine concentrations in effluents.

In Appendix B to the referenced letter, the Supply System agreed that improvements in system performance was advisable for the long term, and that an evaluation was in progress to determine the best option for improvement with regard to the requirement of NUREG-0737. Table II.F.1, Attachment 2, of the NUREG requires the ability to representatively sample and quantify the effluent release rate for gaseous/particulate radioiodine under accident conditions.

As a result of the Engineering evaluation, the decision has been made to install a state-of-the-art, in-line post accident monitoring system (with gamma spectroscopy capability) in the elevated release duct. The new in-line system will be capable of identifying and quantifying any of the reactor building effluents, including noble gases and gases particulate radioiodine, during normal and post-accident conditions. The current schedule is to have the system installed and operational by the end of the 1993 maintenance and refueling outage (June, 1993).

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REACTOR BUILDING POST-LOCA GRAB SAMPLER (REA-SR-48)

In addition, as stated in our response to Inspection Report 90-29 (Appendix B to the referenced letter) the use of the correction factor is considered to be a valid, near-term compensation for considering the operability of Sample Rack REA-SR-48 until such time that the new monitoring system is installed.

Very truly yours,


G. D. Bouchey, Director
Licensing and Assurance

JDA/bk

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