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 SORESEN, G.C. Washington Public Power Supply System
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SUBJECT: Forwards amend to Tech Specs providing additions &/or deletions to paragraph 6.9.3.2 of insert B, including explanation of reasons for each change, per NRC project manager request.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

April 19, 1991
G02-91-081

Docket No. 50-397

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: NUCLEAR PLANT NO. 2, OPERATING LICENSE NPF-21
REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATIONS
IMPLEMENTATION OF GENERIC LETTER 88-16,
ADDITIONAL INFORMATION (TAC NO. 77311)

Reference: 1) Letter, G02-90-132, GC Sorensen (SS) to NRC,
same subject, dated August 2, 1990
2) Letter, G02-91-038, GC Sorensen (SS) to NRC,
same subject, dated February 25, 1991

At the request of the NRC Project Manager assigned to WNP-2, we have reviewed those technical specification change requests currently at the NRC to determine where additional guidance might be required to handle the word processing aspect of preparing the changed pages.

Relative to the subject request we are providing the attached pages. This submittal suggests how to incorporate the added material, and makes additional clarifications requested by the staff. Attachment I provides additions and/or deletions to paragraph 6.9.3.2 of Insert B (Reference 1), including an explanation of the reasons for each change. Except for the changes identified in Attachment I, this submittal does not change the content of the request.

The Supply System considers these changes to be administrative and do not effect the technical content of the previous submittals.

Very truly yours,

G. C. Sorensen

G. C. Sorensen, Manager
Regulatory Programs

Attachments

cc: JB Martin - NRC RV
NS Reynolds - Winston & Strawn
PL Eng - NRC
DL Williams - BPA/399
NRC Site Inspector - 901A

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ATTACHMENT I

REASONS FOR CHANGES TO INSERT B

DELETIONS

1. XN-NF-512(P)(A), "XN-3 Critical Power Correlation"

Reason: The XN-3 correlation is not applicable to the final Cycle 7 and future designs.

4. XN-NF-79-71(P)(A), "Exxon Nuclear Plant Transient Methodology"

Reason: This methodology is discussed in XN-NF-80-19(P)(A), Volume 3, Rev. 2, and Volume 4, Rev. 1, which is also referenced.

ADDITIONS

1. ANF-1125(P)(A) and Supplements 1 and 2, "ANFB Critical Power Correlation", April 1990

Reason: Reference expanded to include applicable revisions and supplements.

3. XN-NF-524(P)(A), Revision 2 and Supplements 1 and 2, "Exxon Nuclear Critical Power Methodology for Boiling Water Reactors", November 1990

Reason: Reference expanded to include applicable revisions and supplements.

4. ANF-913(P)(A), Volume 1, Revision 1 and Volume 1, Supplements 2, 3 and 4, "COTRANSA 2: A Computer Program for Boiling Water Reactor Transient Analysis", August 1990

Reason: Reference expanded to include applicable revisions and supplements.

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5. ANF-CC-33(P)(A), Supplement 2, "HUXY: A Generalized Multirod Heatup Code With 10 CFR 50, Appendix K Heatup Option", January 1991

Reason: Needed to provide spray heat transfer coefficients for 9x9-9X fuel assemblies; not approved when XN-NF-80-19 approved.

6. XN-NF-80-19(P)(A), Volume 1, Supplements 3 and 4, "Exxon Nuclear Methodology for Boiling Water Reactors", November 1990

Reason: Reference expanded to include applicable revisions and supplements.

7. XN-NF-80-19(P)(A), Volume 4, Revision 1, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads", June 1986

Reason: Reference expanded to include applicable revisions and supplements.

8. XN-NF-80-19(P)(A), Volume 3, Revision 2, "Exxon Nuclear Methodology for Boiling Water Reactors THERMEX: Thermal Limits Methodology Summary Description", January 1987

Reason: Reference expanded to include applicable revisions and supplements.

9. XN-NF-85-67(P)(A), Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump Boiling Water Reactor Reload Fuel", September 1986

Reason: Reference expanded to include applicable revisions and supplements

10. ANF-89-014(P), "Generic Mechanical Design for ANF 9x9-IX and 9x9-9X BWR Reload Fuel", May 1989

Reason: Date added.

11. ANF-89-014(P), Supplement 1, "Generic Mechanical Design of ANF 9x9-IX and 9x9-9X BWR Reload Fuel", June 1990

Reason: Reference expanded to include applicable revisions and supplements

12. (SER for 9x9 mechanical design)

Reason: Added to reserve space for NRC SER, which approves References 10 and 11 above.

13. XN-NF-81-22(P)(A), "Generic Statistical Uncertainty Analysis Methodology", November 1983

Reason: Date added.

14. NEDE-24011-P-A-6, "General Electric Standard Application for Reactor Fuel", April 1983

Reason: Added to provide LHGR and MAPLHGR values for re-inserted initial core fuel.

INSERT B

NO CHANGES MADE

Core Operating Limits Report

6.9.3.1 Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, for the following:

- a. The AVERAGE PLANAR LINEAR HEAT GENERATION RATES (APLHGR) for Specifications 3.2.1 and 3.4.1.
- b. The MINIMUM CRITICAL POWER RATIO (MCPR) for Specification 3.2.3.
- c. The LINEAR HEAT GENERATION RATE (LHGR) for Specification 3.2.4.

and shall be documented in the CORE OPERATING LIMITS REPORT.

6.9.3.2 The analytical methods used to determine the core operating limits shall be those topical reports and those revisions and/or supplements of the topical report previously reviewed and approved by the NRC, which describe the methodology applicable to the current cycle. For WNP-2 the topical reports are:

1. ANF-1125(P)(A), and Supplements 1 and 2, "ANFB Critical Power Correlation", April 1990
2. Letter, R. C. Jones (NRC) to R. A. Copeland (ANF), "NRC Approval of ANFB Additive Constants for ANF 9x9-9X BWR Fuel", dated November 14, 1990
3. XN-NF-524(P)(A), Revision 2 and Supplements 1 and 2, "Exxon Nuclear Critical Power Methodology for Boiling Water Reactors", November 1990
4. ANF-913(P)(A), Volume 1, Revision 1 and Volume 1, Supplements 2, 3 and 4, "COTRANSA 2: A Computer Program for Boiling Water Reactor Transient Analysis", August 1990
5. ANF-CC-33(P)(A), Supplement 2, "HUXY: A Generalized Multirod Heatup Code with 10 CFR 50, Appendix K Heatup Option", January 1991.
6. XN-NF-80-19(P)(A), Volume 1, Supplements 3 and 4, "Exxon Nuclear Methodology for Boiling Water Reactors", November 1990
7. XN-NF-80-19(P)(A), Volume 4, Revision 1, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads", June 1986

8. XN-NF-80-19(P)(A), Volume 3, Revision 2, "Exxon Nuclear Methodology for Boiling Water Reactors THERMEX: Thermal Limits Methodology Summary Description", January 1987
9. XN-NF-85-67(P)(A), Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump Boiling Water Reactor Reload Fuel", September 1986
10. ANF-89-014(P), "Generic Mechanical Design for ANF 9x9-IX and 9x9-9X BWR Reload Fuel", May 1989
11. ANF-89-014(P), Supplement 1, "Generic Mechanical Design of ANF 9x9-IX and 9x9-9X BWR Reload Fuel", June 1990
12. (SER for 9x9 mechanical design)
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