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SUBJECT: Responds to Generic Ltr 90-04 re status of implementation of GSIs.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

June 28, 1990
G02-90-113

Docket No. 50-397

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: NUCLEAR PLANT NO. 2, OPERATING LICENSE NPF-21
RESPONSE TO GENERIC LETTER 90-04
REGARDING STATUS OF IMPLEMENTATION OF GENERIC
SAFETY ISSUES (TAC NO. 75993)

Reference: Generic Letter 90-04, "Request for Information on the Status of
Licensee Implementation of Generic Safety Issues Resolved with
Imposition of Requirements or Corrective Actions", dated April 25,
1990

In the reference we were requested to review and provide documentation of the
current implementation status of Generic Safety Issues (GSIs) identified in the
reference. This was to be completed by June 29, 1990.

Our response to this request is the attached marked up copy of the table included
with the reference (Attachment 1 of this response) and the list of notes for this
marked up table (Attachment 2).

As noted in Attachment 1, implementation is complete for all of the applicable
GSIs with the exception of the following:

- 51/L913, Reliability of Open-Cycle Service Water Systems (response to GL
89-13 remains to be completed).
- 75/L003, Vendor Interface for Safety-Related Components (GL 90-03 under
review).


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Page Two
RESPONSE TO GENERIC LETTER 90-04

- 75/B093, Reactor Trip System Reliability (NRC issuance of surveillance intervals, Technical Specification change required).
- 67.3.3/A017, Improved Accident Monitoring (NRC issuance of Wide Range Neutron Monitor Technical Specification change required).

Very truly yours,



G. C. Sorensen, Manager
Regulatory Programs

AGH/bk
Attachments

cc: JB Martin - NRC RV
NS Reynolds - BCP&R
PL Eng - NRC
DL Williams - BPA/399
NRC Site
Inspector - 901A

ATTACHMENT 1

FACILITY NAME: WNP-2
 DOCKET NO.: 50-397
 LICENSEE: Supply System

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSJ/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENT</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NC	***
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NC	***
43 (B107)	Reliability Of Air Systems	All Plants	C	***
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I (See Attachment 2 for schedule and comments)	
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I (See Attachment 2 for schedule and comments)	
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	***
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	***

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

*** See Attachment 2 for comments

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<u>GS1/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	***
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	***
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E (See Attachment 2 for schedule and comments)	
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	***
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	***
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	***
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	***
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	N/A	***

ATTACHMENT 1

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<u>GS1/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	N/A	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	N/A	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	N/A	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	N/A	

ATTACHMENT 1

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<u>GS1/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	***
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	I	***
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	C	***
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	N/A	
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	N/A	
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	***

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ATTACHMENT 1

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	***
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants		
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	N/A	***
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	***

ATTACHMENT 2

NOTES FOR GL 90-04 RESPONSE

GSI/MPA

Comment

40/B065

In Reference 1) below the NRC presented this concern to the Supply System for WNP-2. Our responses were provided in References 2) through 5).

In Supplement 3 to the WNP-2 SER for the OL (NUREG-0892) on page 4-4 it is stated that the staff concludes that the Supply System had adequately addressed the concerns of the May 1981 AEOD report titled, "Safety Concerns Associated with a Pipe Break in the BWR Scram System". This is the AEOD report that GSI No. 40 references (through the April 10, 1981 Generic Letter [GL 81-20]) as the document defining the concerns to be resolved. Reference 6) provided additional information subsequent to the issuance of SER Supplement 3. We do not see that it changes the conclusions reached in the SER supplement.

All actions necessary to resolve the concern for the effects of pipe breaks in the WNP-2 Scram System have been implemented.

References

- 1) Letter, RL Tedesco (NRC) to RL Ferguson (SS), "Safety Concerns Associated with Pipe Breaks in the BWR Scram System", dated April 24, 1981.
- 2) Letter, G02-82-27, GD Bouchev to A Schwencer, "Responses to Request for Information", dated January 13, 1982.
- 3) Letter, G02-82-395, GD Bouchev to A Schwencer, "NUREG-0803", dated April 22, 1982.
- 4) Letter, G02-82-975, GD Bouchev to A Schwencer, "Response to NRC Question 010.066, NUREG-0803", dated December 9, 1982.
- 5) Letter, G02-83-091, GD Bouchev to A. Schwencer, "Response to NRC Question 010.066, NUREG-803", dated February 3, 1983.
- 6) Letter, G02-83-1042, GC Sorensen to A Schwencer, "Generic Safety Evaluation Report BWR Scram Discharge System, Dated December 1, 1980", dated November 11, 1983.

41/B058

In the WNP-2 SER dated March 1982, it is stated on Page 7-6 that modifications to the WNP-2 design satisfies the concern identified by the staff relating to scram discharge volume level sensing. No changes remain to be implemented.

43/B107

In the references below the Supply System responded to GL 88-14 for WNP-2. The September 15, 1989 letter did identify six potential enhancements that were to be evaluated for possible implementation. As we do not believe any of these enhancements are required for resolution of the concerns raised in GL 88-14, we consider our response to the generic letter closed and all required changes implemented.

References

- 1) Letter, G02-89-128, GC Sorensen (SS) to NRC, "Final Response to Generic Letter 88-14 Instrument Air Supply System Problems Affecting Safety-Related Equipment", dated July 28, 1989.
- 2) Letter, G02-89-170, GC Sorensen (SS) to NRC, "Response to GL 88-14 Instrument Air Supply System Problems Affecting Safety-Related Equipment, Follow-Up Report", dated September 15, 1989.
- 3) Letter, G02-90-063, GC Sorensen (SS) to NRC, "Response to Generic Letter 88-14, Instrument Air Supply System Problems Affectin Safety-Related Equipment, Concluding Report (TAC No.71741)", dated March 29, 1990.

51/L913

The Supply System's response to Generic Letter 89-13 was provided in Reference 1) below. Various activities relative to the generic letter remain to be completed as discussed in this reference.

The longest term open item addressed in the reference is the installation of a corrosion coupon system in our spring 1991 refueling outage (Part III.4 of the reference).

However, a design study to evaluate the potential need for biological control of the service water pond water to be concluded by September 1990 (Part I.1 of the reference) may identify the need for a design change whose implementation will extend beyond the spring 1991 outage.

This GSI remains to be fully implemented for WNP-2.

Reference

- 1) Letter, G02-90-017, GC Sorensen (SS) to NRC, "Response to Generic Letter 89-13, Service Water Problems Affecting Safety-Related Equipment", dated February 5, 1990.

67.3.3/A017

In Reference 1) below, the NRC provided an SER for Regulatory Guide 1.97, Revision 2, implementation at WNP-2. In References 2) and 3), we responded to NRC request for additional information for this SER. In Reference 4) the NRC replied to References 2) and 3) by stating that our response to Generic Letter 82-33 and NRC review of conformance to Regulatory Guide 1.97 was closed.

Reference 4) did acknowledge that the requirements for a qualified post-accident neutron flux monitoring capability remained open as a license condition. In Reference 5), we documented closure of this license condition.

A remaining action on the NRC's part is to respond to the Technical Specification change request of Reference 5) relative to the wide range neutron monitor. In Reference 6) the NRC agreed that the license condition was closed.

The issuance of the Technical Specification change and the incorporation of the change into our processes remain to be implemented.

References

- 1) Letter, WR Butler (NRC) to GC Sorensen (SS), "Emergency Response Capability - Conformance to Regulatory Guide 1.97, Revision 2", dated August 9, 1985.
- 2) Letter, G02-85-710, GC Sorensen (SS) to NRC, "Emergency Response Capability - Conformance to Regulatory Guide 1.97, Revision 2", dated October 8, 1985.
- 3) Letter, G02-86-097, GC Sorenson (SS) to NRC, "Emergency Response Capability - Conformance to Regulatory Guide 1.97, Revision 2, Clarification", dated January 23, 1986.
- 4) Letter, GW Knighton (NRC) to GC Sorenson (SS), "Emergency Reponse Capability - Conformance to Regulatory Guide 1.97, Revision 2, (TAC No. 59516)", dated March 23, 1988.
- 5) Letter, G02-89-109, GC Sorensen (SS), "Satisfaction of License Condition 2.C(16) Attachment 2, Item 3(b), Wide Range Neutron Monitor", dated June 15, 1989.
- 6) Letter, R.B. Samworth (NRC) to GC Sorensen (SS), "Satisfaction of License Condition 2.c.(16), Attachment 2, Item 3.(b) Flux Monitoring (TAC No. 73415)", dated May 2, 1990.

75/B076

Generic Letter 83-28, Item 1.1

We responded to this item in Reference 1). In References 2) and 3) the NRC found this response to be acceptable.

References

- 1) Letter, G02-83-1076, GC Sorensen (SS) to A Schwencer, "Response to Generic Letter 83-28", dated November 18, 1983.
- 2) Letter, WR Butler (NRC) to GC Sorensen (SS), "WNP-2 Generic Letter GL 83-28, Item 1.1, Post-Trip Review (Program Description and Procedure)", dated May 17, 1985.
- 3) Letter, WR Butler (NRC) to GC Sorensen (SS), "WNP-2 Generic Letter GL 83-28, Item 1.1, Post-Trip Review (Program Description and Procedure)", dated May 24, 1985.

75/B085

Generic Letter 83-28, Item 1.2

We responded to this item in Reference 1) of Item 1.1 and in a telecon on May 16, 1985 which is discussed in Reference 1) below. In Reference 2) the NRC found our response to this item acceptable.

References

- 1) Letter, WR Butler (NRC) to GC Sorensen (SS), "Draft Technical Evaluation Report (TER) for Salem ATWS Item 1.2, GL 83-28", dated May 22, 1985.
- 2) Letter, WR Butler (NRC) to GC Sorensen (SS), "WNP-2 Generic Letter 83-28, Item 1.2, Post-Trip Review (Data and Information Capability)", dated June 10, 1985.

75/B077

Generic Letter 83-28, Item 2.1

Part 1. We responded to this part in Reference 1) of Item 1.1 and Reference 1) below. In Reference 2) the NRC responded that our response to this item was found acceptable.

Part 2. We provided responses to this part in Reference 1) of Item 1.1 and Reference 1) below. In Reference 3) the NRC states that they believe Part 2 of Item 2.1 to be closed.

References

- 1) Letter, G02-85-257, GC Sorensen (SS) to WR Butler (NRC),

"Generic Letter 83-28, Additional Information", dated May 17, 1985.

- 2) Letter, RB Samworth (NRC) to GC Sorensen (SS), "Closeout of Two Generic Letter 83-28 Items (TAC Nos. 57805 and 53730)", dated November 30, 1988.
- 3) Letter, RB Samworth (NRC) to GC Sorensen (SS), "Safety Evaluation Report for Generic Letter 83-28 Item 2.1 (Part 2) (Vendor Interface Programs - RTS Components) for WNP-2 (TAC No. 60758)", dated February 16, 1989.

75/B086

Generic Letter 83-28, Item 2.2.1

We responded to this item in References 1) and 2) of Item 1.1. In Reference 2) of Item 2.1 the NRC states that our response was acceptable.

75/L003

With the recent issuance of Generic Letter 90-03 concerning Part 2 of this item, we consider the implementation of this GSI remains to be completed. Our response to this generic letter is due to the NRC September 24, 1990. By this date we should know the schedule for any remaining activities.

75/B078

Generic Letter 83-28, Items 3.1.1 and 3.1.2

We responded to these items in Reference 1) of Item 1.1 and in References 1) and 2) below. In Reference 3) the NRC found our response to these two items acceptable.

References

- 1) Letter, GC Sorensen (SS) to WR Butler (NRC), "NRC Generic Letter 83-28, Request for Additional Information", dated July 3, 1985.
- 2) Letter, G02-85-702, GC Sorensen (SS) to WR Butler (NRC), "NRC Generic Letter 83-28, Supplementary Information", dated October 7, 1985.
- 3) Letter, E Adensam (NRC) to GC Sorensen (SS), "Safety Evaluation for Generic Letter 83-28, Post-Maintenance Testing, Items 3.1.1 and 3.1.2 (Reactor Trip System Components) and Items 3.2.1 and 3.2.2 (All Other Safety-Related Components) for WNP-2", dated December 3, 1985 (reissued December 13, 1985).

75/B079

Generic Letter 83-28, Item 3.1.3

We responded to this item in Reference 1) of Item 1.1. In Reference 1) below the NRC found our response to this item acceptable.

Reference

- 1) Letter, WR Butler (NRC) to GC Sorensen (SS), "Safety Evaluation for Generic Letter 83-28, Post-Maintenance Testing, Item 3.1.3 (Reactor Trip System Components) and Item 3.2.3 (All Other Safety-Related Components) for WNP-2", dated September 3, 1985.

75/B087

Generic Letter 83-28, Items 3.2.1 and 3.2.2

We responded to these two items in Reference 1) of Item 1.1 and Reference 2) of Items 3.1.1 and 3.1.2. In Reference 2) of these two items the NRC found our response to Items 3.2.1 and 3.2.2 acceptable.

75/B088

Generic Letter 83-28, Item 3.2.3

We responded to the item in Reference 1) of Item 1.1. In Reference 1) of Item 3.1.3, the NRC found this response acceptable.

75/B080

Generic Letter 83-28, Item 4.1

While GL 90-04 indicates this applies to all plants, GL 83-28 states it only applies to PWRs. We believe GL 83-28 is correct as BWRs do not use reactor trip breakers.

75/B092

Generic Letter 83-28, Item 4.5.1

We responded to this item in Reference 1) of Item 1.1. In Reference 1) below the NRC found the WNP-2 design features to deal with this issue acceptable.

Reference

- 1) Letter, WR Butler (NRC) to GC Sorensen (SS), "WNP-2 Generic Letter 83-28, Item 4.5.1, On-Line Functional Testing of Pilot and Backup Scram Valves", dated July 3, 1985.

75/B093

Generic Letter 83-28, Items 4.5.2 and 4.5.3

We provided our response to Item 4.5.2 in Reference 1) of Item 1.1 and a telecon with the staff on April 4, 1989. In Reference 1) the NRC states that with these responses they considered Item 4.5.2 closed for WNP-2.

We provided responses to Item 4.5.3 in References 1) and 2) of Item 1.1, and Reference 2) below. In Reference 3) the NRC concluded that Item 4.5.3 was complete for WNP-2. However, NRC issuance of the Technical Specification change request of Reference 2) regarding optimized RPS surveillance intervals remains to be issued.

References

- 1) Letter, RB Samworth (NRC) to GC Sorensen (SS), "Closeout of Generic Letter 83-28 Item 4.5.2 (TAC No. 54040)", dated April 25, 1989.
- 2) Letter, G02-89-161, GC Sorensen (SS) to Document Control Desk (NRC), "Request for Amendment to Technical Specification 3/4.3.1 Reactor Protection System Instrumentation and Closeout of Item 4.5.3 of Generic Letter 83-28", dated September 14, 1989.
- 3) Letter, RB Samworth (NRC) to GC Sorensen (SS), "Safety Evaluation for Generic Letter 83-28, Item 4.5.3, Reactor Trip Reliability -- On-Line Functional Testing of the Reactor Trip System (TAC No. 54040)", dated October 3, 1989.

86/B084

The Supply System responded to NUREG-0313 Revision 1 and Generic Letter 84-11 in References 1 and 2 below respectively. Those changes identified in these two references have been implemented for WNP-2. References 3 and 4 concern open issues relative to Generic Letter 88-01, but Generic Letter 90-04 states that GL 88-01 is beyond the scope of GSI 86.

References

- 1) Letter, G02-83-833, GC Sorensen (SS) to A Schwencer (NRC), "Update on Implementation of NUREG-0313, Rev: I", dated September 14, 1983.
- 2) Letter, G02-84-364, GC Sorensen (SS) to D Eisenhut (NRC), "Supply System's Response to NRC's Generic Letter 84-11", dated May 30, 1984.
- 3) Letter, GI2-89-44, RB Samworth (NRC) to GC Sorensen (SS), "Generic Letter 88-01 Response, Request for Additional Information", May 16, 1990.
- 4) Letter, G02-88-164, GC Sorensen (SS) to NRC, "Supply System's Response to NRC's Generic Letter 88-01", dated July 26, 1988.

A-13/B017

The WNP-2 Technical Specifications were issued by the NRC on December 20, 1983 which was subsequent to the November 20, 1980 NRC letter mentioned in Generic Letter 90-04. As such, we assume the NRC's concerns for snubber operability assurance were adequately addressed in the Technical Specifications as issued by the NRC.

A-13/B022

See above note for A-13/B017

A-16/D012

Generic Letter 90-04 states no actions were requested of BWR 3, 4

and 5s.

A-35/B023

The original WNP-2 SER issued in March, 1982 (NUREG-0892) discussed the NRC review of WNP-2 against BTP PSB-1 (referenced in Generic Letter 90-04) in Section 8.4.4 (Pages 8-16). Concerns for the WNP-2 design are mentioned for which the staff found commitments by the Supply System adequate to resolve these concerns. The need for a license condition and FSAR revision are mentioned. This was included in Section 1.7 of the SER as "open" in Item 14. The Supply System amended the FSAR to address this concern (including a change to the response to Question 040.36) in June 1982. In Supplement No. 3 of the SER issued in May 1983, this is an item documented as being resolved.

B-36

The WNP-2 application was submitted for acceptance review on March 17, 1978 and, having passed this process, formal docketing occurred on June 16, 1978.

B-63/B045

The issue of interfacing LOCAs was raised by the NRC during the WNP-2 OL review by Question 040.079. No additional questions were raised, or license conditions established, relative to this issue prior to the issuance of the operating license.

In Reference 1 below the NRC raised a concern relative to fire-induced failure of a high/low pressure boundary. This was eventually resolved by a change to the prior design which has been implemented as described in References 2) and 3).

References

- 1) Letter, DF Kirsch (NRC) to GC Sorenson (SS), "Nonconformance of Safe Shutdown Equipment to Appendix R Requirements", dated June 7, 1985.
- 2) Letter, GL2-88-209, GC Sorenson (SS) to Document Control Desk (NRC), "Fire Protection and Safe Shutdown Capability Response to Safety Evaluation Report (Revised Response to Question No. 25)", dated September 30, 1988.
- 3) Letter, G02-88-214, GC Sorenson (SS) to JB Martin (NRC), "Schedule for Completion of Fire Protection Issues (Inspection Report 88-16)", dated October 7, 1988.

ENCLOSURE 2

FACILITY NAME: WNP-2
DOCKET NO: 50-397
LICENSEE: Washington Public Power Supply System

GSI Status Summary

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	NC	NC	SSER #3
41 (B058)	BWR Scram Discharge Volume Systems	NC	NC	SSER #2
43 (B107)	Reliability Of Air Systems	C 9/15/89	C 9/15/89	
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	I 6/91	C 2/5/90	Licensee continuing to install enhancements beyond the scope of this GSI.
67.3.3 (A017)	Improved Accident Monitoring	I 5/2/90	C 5/2/90	Licensee awaiting license change which is related but beyond the scope of this GSI.
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	C	NC	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	C 6/10/85	C 6/10/85	
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	C	NC	
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	C	NC	
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	E	E	Response to GL 90-03 due 9/24/90
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	C 10/7/85	C 10/7/85	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B079)	Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	C	NC	
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	C 10/7/85	C 10/7/85	
75 (B088)	Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety-Related Components)	C	NC	
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	N/A	N/A	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	N/A	N/A	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	N/A	N/A	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	N/A	N/A	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	C	NC	
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	I 10/3/89	C 10/3/89	Licensee awaiting related license change which is beyond the scope of this GSI
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	C 5/30/84	C 5/30/84	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	N/A	N/A	
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	N/A	N/A	
124	Auxiliary Feedwater System Reliability	N/A	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	NC	NC	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	NC	NC	
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	NA	NA	
A-35 (B023)	Adequacy of Offsite Power Systems		NC	FSAR Question 040.36 (6/82) SSER 2
B-10	Behavior of BWR Mark III Containments	NA	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	NA	NA	

<u>GSI/MPA No.</u>	<u>Title</u>	<u>Licensee Status Determination</u>	<u>Staff Status Determination</u>	<u>Remarks</u>
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	C	NC	FSAR Qestion 040.079 SSER 4 (12/83)