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MARTIN,J.B. Region 5, Ofc of the Director

SUBJECT: Special rept:on 890928 & 1009.post-accident sampling primary coolant radiation monitor inoperable.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

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Docket No. 50-397

October 19, 1989

Mr. J. B. Martin
Regional Administrator
USNRC, Region V
1450 Maria Lane, Suite 210
Walnut Creek, CA 94596

Dear Mr. Martin:

Subject: NUCLEAR PLANT NO. 2
LICENSE NO. NPF-21
SPECIAL REPORT: POST-ACCIDENT SAMPLING PRIMARY COOLANT RADIATION MONITOR

Reference: Letter, C. M. Powers (SS) to J. B. Martin (NRC), dated September 14, 1989

This special report is submitted pursuant to the requirements of WNP-2 Technical Specification Table 3.3.7.5-1 (Instrument No. 29: Post-Accident Sampling Primary Coolant Radiation Monitor), Action Statement 81.

Action Statement 81 requires, "with the number of operable accident monitoring instrumentation channels less than required by the minimum channels operable requirement, either restore the inoperable channel(s) to operable status within 72 hours, or:

(a) Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and

(b) In lieu of any other report required by Specification 6.9.1 prepare and submit a Special Report to the commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status."

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The referenced letter described a condition where part of the Post-Accident Sampling System (PASS) was isolated due to leaking PASS Containment Isolation Valves PSR-V-X77A/3 and PSR-V-X77A/4. The valves were isolated by closing PSR-V-104 (Jet Pump 20 Sample Line Isolation Valve). Valve PSR-V-107, the Jet Pump 10 Sample Line Isolation Valve, had previously been closed due to leakage from PASS Containment Isolation Valves PSR-V-X77A/1 and PSR-V-X77A/2.

On September 28, 1989, during Plant startup from a forced outage due to main condenser problems, PSR-V-104 was opened and the PASS declared operable.

On October 9, 1989, PSR-V-104 was again isolated to repair leaking PASS sample valve PSR-V-616. By having both PSR-V-104 and PSR-V-107 isolated, the system is considered inoperable and Post-Accident Sampling Primary Coolant Radiation Monitor PSR-RI-665 is unable to perform its intended function as required by the Technical Specifications.

In the event of an inoperable PASS during accident conditions, the alternate preplanned method for assessing core damage is Plant Procedure (PPM) 9.3.22, "Core Damage Evaluation." Included in this procedure are evaluations of water level history, installed radiation monitor indication and hydrogen content in the containment atmosphere.

Regarding restoring the system to operable status, current plans are to replace valve PSR-V-616. Due to the long lead time required to procure a replacement valve, it is anticipated that the new valve will be installed and the system declared operable by mid-January, 1990. Maintenance Work Requests (MWRs) have also been prepared to repair PASS Containment Isolation Valves PSR-V-X77A/1, A/2, A/3 and A/4. As stated in the referenced letter, current plans are to repair these valves during the next maintenance and refueling outage (Spring 1990).

Very truly yours,

C. M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:lr

cc: Mr. C. J. Bosted, NRC Site (M/D 901A)
Ms. Dottie Sherman, ANI
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Document Control Desk, NRC