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 MARTIN, J.B. Region 5, Ofc of the Director

SUBJECT: Special rept: on 890821, post-accident sampling primary
 coolant radiation monitor.

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

Docket No. 50-397

September 14, 1989

Mr. J. B. Martin
Regional Administrator
USNRC, Region V
1450 Maria Lane, Suite 210
Walnut Creek, CA 94596

Dear Mr. Martin:

Subject: NUCLEAR PLANT NO. 2
LICENSE NO. NPF-21
SPECIAL REPORT: POST-ACCIDENT SAMPLING PRIMARY
COOLANT RADIATION MONITOR

This special report is submitted pursuant to the requirements of WNP-2 Technical Specification Table 3.3.7.5-1 (Instrument No. 29: Post-Accident Sampling Primary Coolant Radiation Monitor), Action Statement 81.

Action Statement 81 requires, "with the number of operable accident monitoring instrumentation channels less than required by the minimum channels operable requirement, either restore the inoperable channel(s) to operable status within 72 hours, or:

- (a) Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
- (b) In lieu of any other report required by Specification 6.9.1 prepare and submit a Special Report to the commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status."

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J. B. Martin

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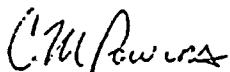
SPECIAL REPORT: POST-ACCIDENT SAMPLING PRIMARY
COOLANT RADIATION MONITOR

On August 21, 1989, during routine Post-Accident Sampling System (PASS) training for Plant Chemistry Personnel, it was noted that PASS Containment Isolation Valves PSR-V-X77A/3 and PSR-V-X77A/4 appeared to leak when in the closed position. Accordingly, the valves were isolated by closing PSR-V-104 (Jet Pump 20 Sample Line Isolation Valve) and the LCO was entered. On August 24, 1989 PSR-V-104 was opened and the system returned to service, within the 72-hour LCO time frame. After further review of the situation, the decision was made to manually isolate the valves to prevent back leakage of Jet Pumps 10 and 20 to the Demineralized Water System. On August 31, 1989, the valves were again isolated by closing PSR-V-104 (PSR-V-107, Jet Pump 10 Sample Line Isolation Valve, had previously been closed due to leakage from PASS Containment Isolation Valves PSR-V-X77A/1 and PSR-V-X77A/2). Because the system is inoperable, Post-Accident Sampling Primary Coolant Radiation Monitor PSR-RI-665 is unable to perform its intended function as required by the Technical Specifications.

In the event of an inoperable PASS during accident conditions, the alternate preplanned method for assessing core damage is Plant Procedure (PPM) 9.3.22, "Core Damage Evaluation." Included in this procedure are evaluations of water level history, installed radiation monitor indication and hydrogen content in the containment atmosphere.

Regarding restoring the system to operable status, Maintenance Work Requests (MWRs) have been prepared to repair PASS Containment Isolation Valves PSR-V-X77A/1, A2, A3 and A4. Current plans are to repair the valves during the next maintenance and refueling outage (Spring 1990).

Very truly yours,



C. M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:lr

cc: Mr. C. J. Bosted, NRC Site (M/D 901A)
Ms. Dottie Sherman, ANI
Mr. D. L. Williams, BPA (M/D 399)
Document Control Desk, NRC

