

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8803140153 DOC.DATE: 88/03/04 NOTARIZED: NO DOCKET #  
 FACIL:50-397 WPPSS Nuclear Project, Unit 2, Washington Public Powe 05000397  
 AUTH.NAME AUTHOR AFFILIATION  
 DAVISON,W.S. Washington Public Power Supply System  
 POWER,C.M. Washington Public Power Supply System  
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 88-004-00:on 880204,standby liquid control sys  
 inoperable longer than allowed by Tech Specs.

W/8 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 5  
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

### NOTES:

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	AEOD/DOA	1	1	AEOD/DSP/NAS	1
	AEOD/DSP/ROAB	2	2	AEOD/DSP/TPAB	1
	ARM/DCTS/DAB	1	1	DEDRO	1
	NRR/DEST/ADS7E4	1	0	NRR/DEST/CEB8H7	1
	NRR/DEST/ESB 8D	1	1	NRR/DEST/ICSB7A	1
	NRR/DEST/MEB9H3	1	1	NRR/DEST/MTB 9H	1
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	NRR/DRIS/SIB9A1	1	1	NRR/PMAS/ILRB12	1
	REG FILE 02	1	1	RES TELFORD,J	1
	RES/DE/EIB	1	1	RES/DRPS DIR	1
	RGN5 FILE 01	1	1		
EXTERNAL:	EG&G GROH,M	5	5	FORD BLDG HOY,A	1
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## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 7										PAGE (3) 1 OF 0 4																																		
TITLE (4) Standby Liquid Control System Inoperable Longer Than Allowed By Technical Specifications Caused By Inadequate Procedure																																																						
EVENT DATE (5)									LER NUMBER (6)									REPORT DATE (7)									OTHER FACILITIES INVOLVED (8)																											
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES												DOCKET NUMBER(S)															
0 2			0 4			8 8			8 8			0 0			4			0 0			0 3			0 4			8 8															0 5 0 0 0												
OPERATING MODE (9) 1									THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																													
POWER LEVEL (10) 1 0 0									20.402(b)									20.405(c)									50.73(a)(2)(iv)									73.71(b)																		
									20.405(a)(1)(i)									50.36(c)(1)									50.73(a)(2)(v)									73.71(c)																		
									20.405(a)(1)(ii)									50.36(c)(2)									50.73(a)(2)(vii)									OTHER (Specify in Abstract below and in Text, NRC Form 365A)																		
									20.405(a)(1)(iii)									50.73(a)(2)(i)									50.73(a)(2)(viii)(A)																											
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20.405(a)(1)(v)									50.73(a)(2)(iii)									50.73(a)(2)(ix)																																				
LICENSEE CONTACT FOR THIS LER (12)																																																						
NAME W.S. Davison, Compliance Engineer																				TELEPHONE NUMBER 5 0 9 3 7 7 - 2 5 0 1																																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Extension 2726																																																						
CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRC					CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRC									
SUPPLEMENTAL REPORT EXPECTED (14)																														EXPECTED SUBMISSION DATE (15)										MONTH					DAY					YEAR				
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO																																		

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While dispositioning a Nonconformance Report concerning the Standby Liquid Control (SLC) System on February 4, 1988, an engineer determined that one of the two pump relief valves, SLC-RV-29B, was set 25 psig below the allowable value listed in Plant Technical Specifications. Further investigation revealed that SLC pump 1B (SLC-P-1B) was inoperable for longer than the seven-day period allowed by Technical Specifications. The relief valve setpoint was immediately readjusted to within the allowable value. The root cause was determined to be procedural inadequacy. The Relief Valve Setpoint Control and Logging Procedure did not contain adequate direction concerning Technical Specification requirements. Corrective actions include a review of Plant Technical Specifications to identify any similar remaining situations and the modification of the Relief Valve Setpoint Control and Logging Procedure to include Technical Specification requirements. This event posed no threat to the safety of Plant personnel or the public.

8803140153 880304  
PDR ADOCK 05000397  
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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	8 8	- 0 0 4	- 0 0	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

- a) Power Level - 100%
- b) Plant Mode - 1 (Power Operation)

Event Description

While dispositioning a Nonconformance Report which documented a problem with the Standby Liquid Control (SLC) system pump number 1B relief valve (SLC-RV-29B) on February 4, 1988, a Plant Technical Staff engineer determined that the relief valve was set 25 psig below the allowable value listed in Plant Technical Specifications. Further investigation revealed that, due to the improper relief valve setpoint, SLC-P-1B was inoperable for longer than the seven-day period allowed by Technical Specification requirement 3.1.5.a.1.

On January 26, 1988, during plant operation at approximately 100 percent power, SLC-P-1B was declared inoperable at 0758 hours to allow repair of SLC-RV-29B which was leaking during pump operation. Entry was made into Technical Specification LCO Action Statement (TSAS) 3.1.5.a.1 which allowed seven days to return the pump to operable status.

On January 28, 1988, after repair of SLC-RV-29B was completed, the valve was set and tested at a lift setpoint of 1375 psig as directed by the Relief Valve Setpoint Data Sheet section of the work package. At 2215 hours, after the relief valve was reinstalled, the SLC System was declared operable and the plant exited TSAS 3.1.5.a.1.

On February 4, 1988, during evaluation of a Nonconformance Report written to document the failure of SLC-RV-29B to lift within tolerance specified by the Plant Maintenance Procedures, the Technical Staff SLC System engineer determined that the final setpoint of the relief valve was not within values specified by Technical Specification 3.1.5. The minimum allowable value specified was 1400 psig. The relief valve setpoint, as recorded in the work package, was 1375 psig. Investigation determined that the Relief Valve Setpoint Data Sheet supplied to the Maintenance group by Plant Technical Staff was in error, listing the setpoint as 1400 psig  $\pm$  3 percent.

Immediate Corrective Action

Since the plant was in the Shutdown condition, no action was required to comply with Technical Specification 3.1.5. The relief setpoint was immediately readjusted to within the allowable value and at 0110 hours on February 5, 1988, the SLC System was declared operable.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	8 8	- 0 0 4	- 0 0	0 3	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Further Evaluation and Corrective Actiono Further Evaluation

1. This event is being reported as a condition prohibited by the Plant's Technical Specifications in accordance with 10CFR50.73(a)(2)(i)(B).
2. SLC-P-1B was inoperable from January 26, 1988, to February 4, 1988. This period totals nine days, which is two days in excess of the seven days allowed by Technical Specification 3.1.5. This constitutes noncompliance with the Plant Technical Specifications.
3. The immediate cause of the event was determined to be an error as a direct result of an approved procedure in that the incorrectly set relief valve was reinstalled in the SLC System as operable as a result of the incorrect Relief Valve Setpoint Data Sheet.
4. The root cause of this event was determined to be procedural inadequacy in that the Relief Valve Setpoint Control and Logging Procedure did not contain adequate direction concerning Technical Specification requirements to enable the engineer to correctly complete the Relief Valve Setpoint Data Sheet.

o Corrective Action

1. A review of the Plant Technical Specifications will be performed to identify any similar limitations which may be more restrictive than the ASME Code for relief valve setpoints.
2. The Relief Valve Setpoint Control and Logging procedure will be modified to strengthen the setpoint verification process to specifically include Technical Specification requirements.

Safety Significance

Technical Specifications allow operation of the SLC system with one of the redundant components inoperable for a period of seven days. The fact that SLC-P-1B (redundant to SLC-P-1A) was technically inoperable for a slightly longer period did not decrease the system's ability to perform its design function. As shown by testing, the highest pressure expected to be seen at this relief valve during accident conditions is less than 1300 psig. With SLC-RV-29B set at 1375 psig, sufficient margin existed to ensure system injection capability at full capacity. This event posed no threat to the safety of Plant personnel or the public.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0   5   0   0   0   3   9   7	8   8	-   0   0   4	-   0   0	0   4	OF	0   4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Similar Events

None.

EIIS InformationText ReferenceEIIS Reference

	System	Component
SLC	BR	---
SLC-RV-29B	BR	RV
SLC-P-1A	BR	P
SLC-P-1B	BR	P

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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

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P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

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Docket No. 50-397

March 7, 1988

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 88-04

Dear Sir:

Transmitted herewith is Licensee Event Report No. 88-04 for the WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the items of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

*C.M. Powers*

C.M. Powers (M/D 927M)  
WNP-2 Plant Manager

CMP:sm

Enclosure:  
Licensee Event Report No. 88-04

cc: Mr. John B. Martin, NRC - Region V  
Mr. C.J. Bosted, NRC Site (M/D 901A)  
INPO Records Center - Atlanta, GA  
Ms. Dottie Sherman, ANI  
Mr. D.L. Williams, BPA (M/D 399)

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