

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8803080374 DOC. DATE: 87/12/18 NOTARIZED: NO DOCKET #  
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Powe 05000397  
 AUTH. NAME AUTHOR AFFILIATION  
 DAVISON, W. S. Washington Public Power Supply System  
 POWERS, C. M. Washington Public Power Supply System  
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-031-01: on 871118, condition prohibited by plant Tech  
 Specs not identified when performing weekly Technical Spec  
 surveillance for plant batteries. Caused by personnel error.  
 Counseled individuals involved. W/880225 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5  
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

	RECIPIENT		COPIES			RECIPIENT		COPIES	
	ID CODE/NAME		LTTR	ENCL		ID CODE/NAME		LTTR	ENCL
	PD5 LA		1	1		PD5 PD		1	1
	SAMWORTH, R		1	1					
INTERNAL:	ACRS MICHELSON		1	1		ACRS MOELLER		2	2
	AEOD/DOA		1	1		AEOD/DSP/NAS		1	1
	AEOD/DSP/ROAB		2	2		AEOD/DSP/TPAB		1	1
	ARM/DCTS/DAB		1	1		DEDRO		1	1
	NRR/DEST/ADS7E4		1	0		NRR/DEST/CEB8H7		1	1
	NRR/DEST/ESB 8D		1	1		NRR/DEST/ICSB7A		1	1
	NRR/DEST/MEB9H3		1	1		NRR/DEST/MTB 9H		1	1
	NRR/DEST/PSB8D1		1	1		NRR/DEST/RSB 8E		1	1
	NRR/DEST/SGB 8D		1	1		NRR/DLPQ/HFB10D		1	1
	NRR/DLPQ/QAB10A		1	1		NRR/DOEA/EAB11E		1	1
	NRR/DREP/RAB10A		1	1		NRR/DREP/RPB10A		2	2
	NRR/DRIS/SIB9A1		1	1		NRR/PMAS/ILRB12		1	1
	REG FILE 02		1	1		RES TELFORD, J.		1	1
	RES/DE/ETB		1	1		RES/DRPS DIR		1	1
	RGN5 FILE 01		1	1					
EXTERNAL:	EG&G GROH, M		5	5		FORD BLDG HOY, A		1	1
	H ST LOBBY WARD		1	1		LPDR		1	1
	NRC PDR		1	1		NSIC HARRIS, J		1	1
	NSIC MAYS, G		1	1					



**LICENSEE EVENT REPORT (LER)**

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 05000397										PAGE (3) 1 of 04																																							
TITLE (4) 250 Volt Battery Float Voltage Below Technical Specification Limit - Not Corrected Due to Personnel Error During Battery Surveillance																																																											
EVENT DATE (5)									LER NUMBER (6)									REPORT DATE (7)									OTHER FACILITIES INVOLVED (8)																																
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES												DOCKET NUMBER(S)																				
11			11			88			78			7			03			1			0			11			21			88			7															05000397											
OPERATING MODE (9)									THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																																		
POWER LEVEL (10) 093									20.402(b)									20.405(c)									50.73(a)(2)(iv)									73.71(b)																							
									20.405(a)(1)(i)									50.38(c)(1)									50.73(a)(2)(v)									73.71(c)																							
									20.405(a)(1)(ii)									50.38(c)(2)									50.73(a)(2)(vii)									OTHER (Specify in Abstract below and in Text, NRC Form 365A)																							
									20.405(a)(1)(iii)									50.73(a)(2)(i)									50.73(a)(2)(viii)(A)																																
									20.405(a)(1)(iv)									50.73(a)(2)(ii)									50.73(a)(2)(viii)(B)																																
									20.405(a)(1)(v)									50.73(a)(2)(iii)									50.73(a)(2)(ix)																																
LICENSEE CONTACT FOR THIS LER (12)																																																											
NAME W.S. Davison, Compliance Engineer																					TELEPHONE NUMBER 509377-2501																																						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2726																																																											
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC																																
SUPPLEMENTAL REPORT EXPECTED (14)																					EXPECTED SUBMISSION DATE (15)									MONTH			DAY			YEAR																							
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO																																															
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																																																											
On November 18, 1987, a condition prohibited by Plant Technical Specifications was not identified when performing the weekly Technical Specification surveillance for Plant batteries. The test was completed by Plant electricians as a successful test and given to the Operations Department Shift Manager for review and acknowledgement of completion. It was then reviewed and signed by the Shift Manager at 0531 hours and submitted to the Plant Electrical Maintenance group for review. At 0730 hours, the department cognizant reviewer recognized that the 250 Volt Battery voltage recorded was below the Technical Specification minimum required value. After the Shift Manager was informed of the low voltage condition, the voltage was returned to within allowable limits at 0821 hours. The immediate cause was personnel error in that contrary to the approved procedure the electricians did not inform the Operations staff of the discrepancy and signed the test results as being satisfactory. The root cause was a combination of lack of mental attention and poor work practice. The electricians failed to recognize the low battery voltage and did not perform the procedure steps in the proper order. Corrective actions include the counselling of individuals involved, providing instruction to Maintenance Department personnel concerning performance of Technical Specification surveillances, placing the LER on the Required Reading List for Operations and Maintenance Departments, adding of this event to the subject list for Operations Requalification Training and Maintenance shop meetings and modification of the Technical Specification Surveillance Testing Program. No significant degradation of 250 Volt Battery capacity occurred. All systems with safety-related isolation valves supplied from the 250 Volt Battery have redundant AC powered valves for isolation. The Shutdown Cooling System (SDC) functions can be performed by the Alternate Shutdown Cooling Flowpath if SDC is not operable. Additionally, the DC power system at WNP-2 contains two other Class IE 125 Volt DC power supply subsystems, each of which alone is capable of supplying sufficient power to ensure safe shutdown capability. Consequently, this event had no affect on the health and safety of plant personnel or the public.																																																											

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PDR ADOCK 05000397  
PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	8 7	— 0 3 1	— 0 1	0 2	OF	0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

- a) Power Level - 93%
- b) Plant Mode - 1 (Power Operation)

Event Description

On November 18, 1987, a condition prohibited by Plant Technical Specifications was not identified when performing the weekly Technical Specification surveillance for Plant batteries. The test was completed by Plant electricians and given to the Operations Department Shift Manager for review and acknowledgement of completion. It was then reviewed and signed by the Shift Manager at 0531 hours and submitted to the Plant Electrical Maintenance group for review. At 0730 hours, the cognizant Maintenance Department reviewer recognized that the 250 Volt Battery voltage was recorded as being below the Technical Specification minimum required value. After the Shift Manager was informed of the low voltage condition, the voltage was returned to within allowable limits at 0821 hours.

The immediate cause of this event was determined to be personnel error in that two utility electrical technicians failed to recognize the actual plant conditions (i.e., that the 250 Volt Battery was inoperable due to low battery float voltage). Contrary to the approved procedure, the electricians did not inform the Operations staff of any discrepancies and signed the test results as being satisfactory.

The root cause of this event was determined to be a combination of lack of mental attention and poor work practice. The electricians recorded the voltage properly as required by procedure, but failed to recognize the significance of the Technical Specification required battery system parameters. The low voltage reading of 256.9 volts (minimum voltage is 258 volts) should have initiated action to inform the Operations staff on an immediate basis and to signify the test results as unsatisfactory on the cover page of the procedure. Poor work practice was used when the technicians did not perform the procedure step by step, but instead recorded the data in convenient groups. Subsequently, they then initialed all the procedure steps on that page, as a group, without carefully reading each step individually. As a result of this method, the Technical Specification acceptance criteria listed in the procedure were overlooked.

Immediate Corrective Action

Plant operators returned 250 Volt Battery voltage to within Technical Specification required limits as soon as the out-of-specification condition was recognized.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Washington Nuclear Plant - Unit 2	0500039787	87	031	01	03	OF	04

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Further Evaluation and Corrective Action

- o The Shift Manager accepted the surveillance as complete at 0531 hours. The cognizant Maintenance Department reviewer recognized that an error had been made at 0730 hours and float voltage was returned to within Technical Specification limits at 0821 hours. Therefore, 250 Volt Battery voltage was below the minimum value and the battery was inoperable for 2 hours and 51 minutes. Since Specification 3.8.2.1.a allows a maximum of 2 hours to restore the battery to operable status, this event is reportable per 10CFR50.73(a)(2)(i)(B) in that it was a "condition prohibited by the Plant's Technical Specifications."
- o A confirmatory check of battery terminal voltage, individual battery cell electrolyte levels and individual battery cell specific gravities verified the battery to be in an operable status.
- o The individuals involved with this event have been counselled concerning responsibility for identification of operability sensitive surveillance data and proper procedure performance.
- o Maintenance Department personnel will receive instruction on the proper methods of performing Technical Specification surveillances and identification and reporting of discrepancies.
- o This LER will be required reading for all Operations and Maintenance Department personnel.
- o This LER will be added to the subject lists of items to be discussed during Operations Requalification Training and Maintenance Department shop meetings.
- o The Technical Specification Surveillance Testing Program will be modified to identify the Shift Manager's test completion signature as the final review for equipment operability and to specify that any applicable Technical Specification LCO time requirement start at that point.
- o During the two-year periodic review of Technical Specification Surveillance Procedures, each procedure will be modified as necessary to improve identification of test acceptance criteria by technicians and to improve the capability for timely, thorough review by the Shift Manager.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
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Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7 8 7	—	0 3 7	—	0 1	0 4 OF 0 4	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Safety Significance

A review of the details of this event shows that, although the Class IE 250 Volt Battery was inoperable per Technical Specifications for 51 minutes longer than allowed, this DC power source contained sufficient power to ensure safe shutdown capability during the entire time period. The 250 Volt DC System supplies power to Reactor Core Isolation Cooling System (RCIC), Reactor Water Cleanup System (RWCU) and the Reactor Heat Removal Shutdown Cooling System (SDC) isolation valves. The RCIC, SDC and RWCU systems each contain a redundant AC powered isolation valve which would function to isolate if the DC powered valve was inoperable. The RCIC and RWCU systems are not required to function during accident conditions. The SDC system functions can be performed by the Alternate Shutdown Cooling Flow path if required. Additionally, the DC power system at WNP-2 contains two other Class IE 125 volt DC power supply subsystems each of which alone is capable of supplying sufficient power to ensure safe shutdown capability. Consequently, this event had no affect on the health and safety of Plant personnel or the public.

Similar Events

Other events which involved Technical Specification equipment operability problems during surveillance activity are documented on LERs 87-23, 86-05, 85-54, 85-51 and 85-49.

EIIS InformationText ReferenceEIIS Reference

	System	Component
250 Volt Battery	EI	BTRY
125 Volt DC Power Supply Systems	EI	BTRY
RCIC	BN	----
RWCU	CE	----
SDC	BO	----

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

Docket No. 50-397

February 25, 1988

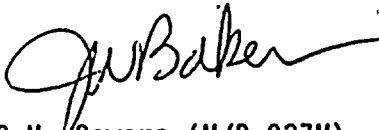
Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2  
LICENSEE EVENT REPORT NO. 87-031-01

Dear Sir:

Transmitted herewith is Licensee Event Report No. 87-031-01 for the WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the items of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,



C.M. Powers (M/D 927M)  
WNP-2 Plant Manager

CMP:db

Enclosure:  
Licensee Event Report No. 87-031-01

cc: Mr. John B. Martin, NRC - Region V  
Mr. C. J. Bosted, NRC Site (M/D 901A)  
INPO Records Center - Atlanta, GA  
Ms. Dottie Sherman, ANI  
Mr. D. L. Williams, BPA (M/D 399)

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