

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

P.O. Box 968 • 3000 George Washington Way • Richland, Washington 99352

April 24, 1986  
G02-86-367

*TAC 60804*  
*PDR*

Docket No. 50-397

Director of Nuclear Reactor Regulation  
Attn: E. G. Adensam, Project Director  
BWR Project Directorate No. 3  
Division of BWR Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Ms. Adensam

Subject: NUCLEAR PLANT NO. 2  
OPERATING LICENSE NPF-21, REQUEST FOR AMENDMENT TO  
TECHNICAL SPECIFICATIONS - RELOAD LICENSE AMENDMENT  
(CYCLE 2) - SUPPLEMENT

Reference: 1) Letter, G02-86-173, GC Sorensen (SS) to EG Adensam  
(NRC), same subject, dated February 26, 1986.  
2) Letter, G02-86-311, GC Sorensen (SS) to EG Adensam  
(NRC), same subject, dated April 7, 1986.

In Reference 1) the Supply System stated that, due to the shortfall in first cycle energy production, a supplement to the WNP-2 Reload Summary Report would be submitted to discuss changes in the expected WNP-2 Cycle 2 reload batch which occurred subsequent to initiation of the original licensing analysis. The Supplement attached hereto fulfills that commitment.

The original reload batch was planned for 196 assemblies. It was later revised to 132 assemblies. The reload summary report was based on a neutronics analysis for 132 reload assemblies, an ECCS analysis for an all ENC 8X8 core, and a transient analysis for 196 reload assemblies except for the control rod withdrawal error transient which was performed for 132 assemblies. This supplement addresses re-analysis of all appropriate transients for 132 reload assemblies. The final Cycle 2 reload batch design has been determined to be 128 reload assemblies. This additional batch reduction became possible due to further last minute truncation of the initial cycle by Bonneville Power Administration. Exxon Nuclear Company (ENC) has determined that the 132 assembly analyses are also applicable to the 128 reload batch design.

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As a result of the transient analysis performed on the revised WNP-2 Cycle 2 core loading, it is necessary to make changes to the WNP-2 Technical Specifications submitted with Reference 1. The changes consist primarily in modifications to the MCPR values listed in Table 3.2.3-1 of the WNP-2 Technical Specifications, and are attached hereto. In addition, Technical Specification 3/4.1.3.3 is deleted as it is not applicable to the ENC analysis.

Use of the Black Fox impeller and other internal components in WNP-2 Loop B Recirculation Pump does not change the conclusions reached previously with regard to the WNP-2 Cycle 2 ECCS and transient analyses. The previously developed MAPLHGR limits remain valid and no change in MCPR limits is required.

The Supply System has reviewed this additional information per 10 CFR 50.92 and determined that it does not:

- 1) Involve a significant increase in the probability or consequences of an accident previously evaluated because the appropriate analyses have been reanalyzed for the reload fuel, and the proposed Technical Specification changes are within the previous acceptable criteria; or
- 2) Create the possibility of a new or different kind of accident than previously evaluated because the ENC fuel technology and the design of the fuel is not significantly different than that used in the initial core and has been previously found acceptable to the NRC; or
- 3) Involve a significant reduction in the margin of safety because the calculated safety limit for the new core is identical to that for the initial core.

This change has been reviewed and approved by the WNP-2 Plant Operations Committee and the Supply System Corporate Nuclear Safety Review Board. In accordance with 10 CFR 50.91, the State of Washington has been provided a copy of this letter.



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Due to our having been directed by the BPA to shut down ahead of schedule, we are now planning to restart May 20, 1986, which is somewhat earlier than originally anticipated. In order for the NRC staff to be able to complete their evaluation of our reload submittal, we would like to suggest a meeting as soon as possible wherein we can address any concerns the staff may have.

Please contact Mr. P. L. Powell, Manager, WNP-2 Licensing so that such a meeting can be arranged. Thank you for your cooperation in this matter.

Very truly yours,



G. C. Sorensen, Manager  
Regulatory Programs

HLA/bk  
Attachment

cc: JO Bradfute - NRC  
C Eschels - EFSEC  
JB Martin - NRC RV  
E Revell - BPA  
NS Reynolds - BLCP&R  
NRC Site Inspector

