

WASHINGTON PUBLIC POWER
SUPPLY SYSTEMWNP-2
CALCULATION COVER SHEETDCP Page: *NONE*

PAGE 1.000

WNP-2

*Nuclear*1. CALCULATION NO.
*NE-02-86-10*2. QUALITY CLASS
I

4. EQUIPMENT PIECE NO.

*RHR System (LPCI),
Relief Valves*

5. SUPERSEDED BY

None

6. SUBJECT

*Reactor Depressurization Using Six Relief Valves
With Reduced Coolant Injection*

7. REMARKS

*Appendix R Calculation, for Alternate Shut-
down from Remote Control Room*

CALCULATION PERFORMANCE RECORD

8. REVISION NO.	9. STATUS		10. REVISION DESCRIPTION	11. PERFORMED BY / DATE
	PRELIMINARY	FINAL		
<i>0</i>		<i>Final</i>	<i>New Issue</i>	<i>F. J. Markowski 30 JUL 86</i>

VERIFICATION / APPROVAL RECORD

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<i>0</i>	<i>[Signature] 8/15/86</i>	<i>D. L. Whitcomb 8/27/86</i>	

8609150294 860909
PDR ADOCK 05000397
F PDR

Verification Checklist For Calculations

Calculation NE-02-86-10 Revision 0 was checked and verified using the following methods

Checklist below

Alternate Calculation, *See Appendix 5*

The calculation is adequate for the purpose intended.

Verifier Signature/Date

Verifier Initial

Checklist Item

Initial

Logical Consistency of Analysis

Accuracy of Input Data

Consistency of Input Data with Approved Criteria

Validity of Assumptions

Appropriateness of Method

Arithmetical Accuracy

Reasonableness of Output Conclusion

Other Elements Considered

Comparison with LE modeled results verify logical consistency and qualitative behavior of this model. Differences in initial conditions and assumptive parameter quantitative differences in output. Both models indicate large margins of safety for this scenario. This model appears to be adequate for this immediate application - any future applications should be verified.

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Page 1.010

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Calculation Index

Page 1.010
Calculation NE-02-86-10
Revision No. 0

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Calculation Index	1.010 through <u>1.014</u>
Calculation Input Summary	2.000 through <u>2.004</u>
Calculation Output Summary	3.000 through <u>3.000</u>
<u>Manual Analysis Pages</u>	<u>5.000 through 5.061</u>
Computer Runs	Appendix A _____ Pages
Unverified Computer Program Description	Appendix B _____ Pages
Alternate Calculations	Appendix C _____ Pages
Contractor Calculations Package	Appendix D _____ Pages

Continued on Page 1.011

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Bounding Estimate for Heatup
Rate of Unsubmerged Fuel

Appendix 2 12 Pages

Bounding Estimate for Stored
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Table of Contents Continued

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Listing of Computer Code in BASIC
Language

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Printouts of Computer Results

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Independent Calculation G-KK-6-140
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Comparison of Results and Discus-
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G-KK-6-140 and NE-02-86-10

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9 Pages

Completeness of the Postulated
Hypothetical Scenario

WNP-2
CALCULATION INPUT SUMMARY

1. PURPOSE / SCOPE OF ANALYSIS

See Section 1: Purpose of
Calculation, Page

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2.000

3. CALCULATION NO.

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4. REVISION

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5. REFERENCES

DOCUMENT NO.

ISSUE DATE
OR REVISION

TITLE

AUTHOR

1. WNP-2 FSAR as of APR 1986

Chapter 5: Reactor Coolant System and
Connected Systems

Chapter 6: Engineered Safety Features

→ continued on Page 2.001

6. ANALYSIS METHOD (CHECK APPROPRIATE BOXES)

☐ MANUAL (List source of equations in reference section above or document equations on page 5.000)

☒ COMPUTER

☐ VERIFIED PROGRAM: CODE NAME / REVISION: _____

☒ UNVERIFIED PROGRAM:

→ see Appendix 3

7. INPUT DATA AND ASSUMPTION SUMMARY

SOURCE OR REF.

PAGE OR
SECTION

DATA / ASSUMPTION DESCRIPTION

Input data in Appendix 3 and
Appendix 4.

Input assumptions (including scenario)
described in Section 1 and Section 2
(Pages 5.001 thru 5.051)

References Continued

APPENDIX F: Plant Fire Protection
Evaluation

2. GD2-83-243

WNP-2 Fire Protection Safe Shutdown
Analysis

Letter Report, G.D. Buckey to A. Schwencer,
21 MAR 83

3. IE Information Notice 84-09

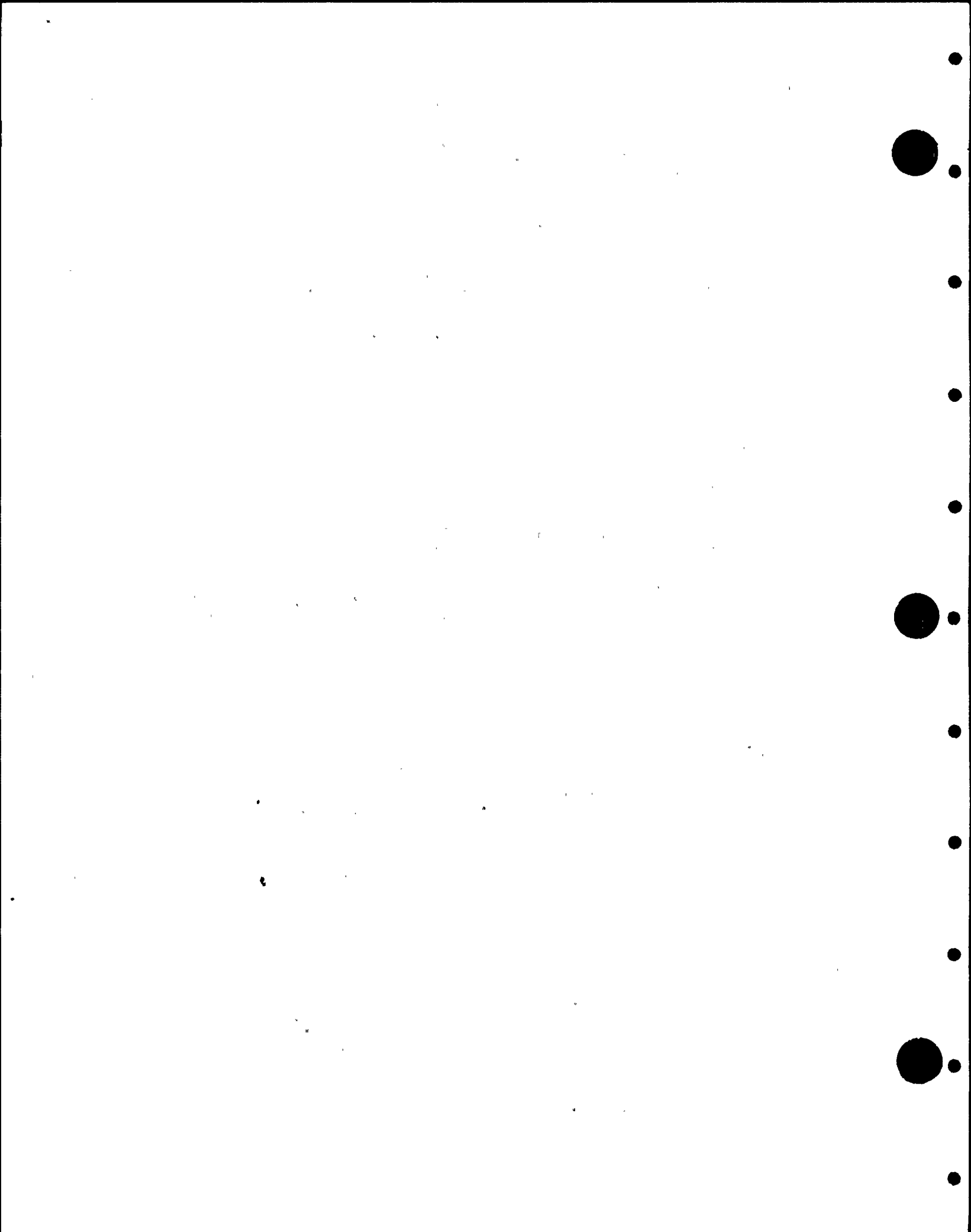
Lessons Learned from NRC Inspections
of Fire Protection Safe Shutdown Systems
USNRC, 13 FEB 84

4. SECK-85-306

Staff Recommendations Regarding Im-
plementation of Appendix R to
10 CFR 50

11 Page Letter with 9 Enclosures
USNRC, 17 SEP 85

5. 10 CFR 50, Appendix R



References Continued

6. NEDO-24708A, Rev. 1

Additional Information for NRC
Staff Generic Report on Boiling
Water Reactors

General Electric Company
2 Volumes, DEC 80

7. NE-02-84-19, Rev. 0

Reactor Depressurization Using 18
Relief Valves
WPPSS, 20 DEC 84

8. NE-02-84-30, Rev. 0

Reactor Depressurization Using 3
Relief Valves
WPPSS, 15 DEC 84

9. NE-02-84-31, Rev. 0

Hydraulic Pressure Drop for RHR
Loop B
WPPSS, 12 DEC 84

References Continued

10. L.S. Marks, Seventh Edition

Handbook for Mechanical Engineers
McGraw Hill, 1967

11. ASME Steam Tables, Fourth Edition

Thermodynamic Properties of Steam
ASME, 1979

12. FJM Engineering Log Vol. II, P. 88, 89

WNP-2 Reactor Liquid Level versus
Volume, Including Void Fractions
Telecon with GE, 07 FEB 84

13. FJM Engineering Log Vol. II, P. 134, 135

WNP-2 Reactor Liquid Level Chan-
ge on Scram

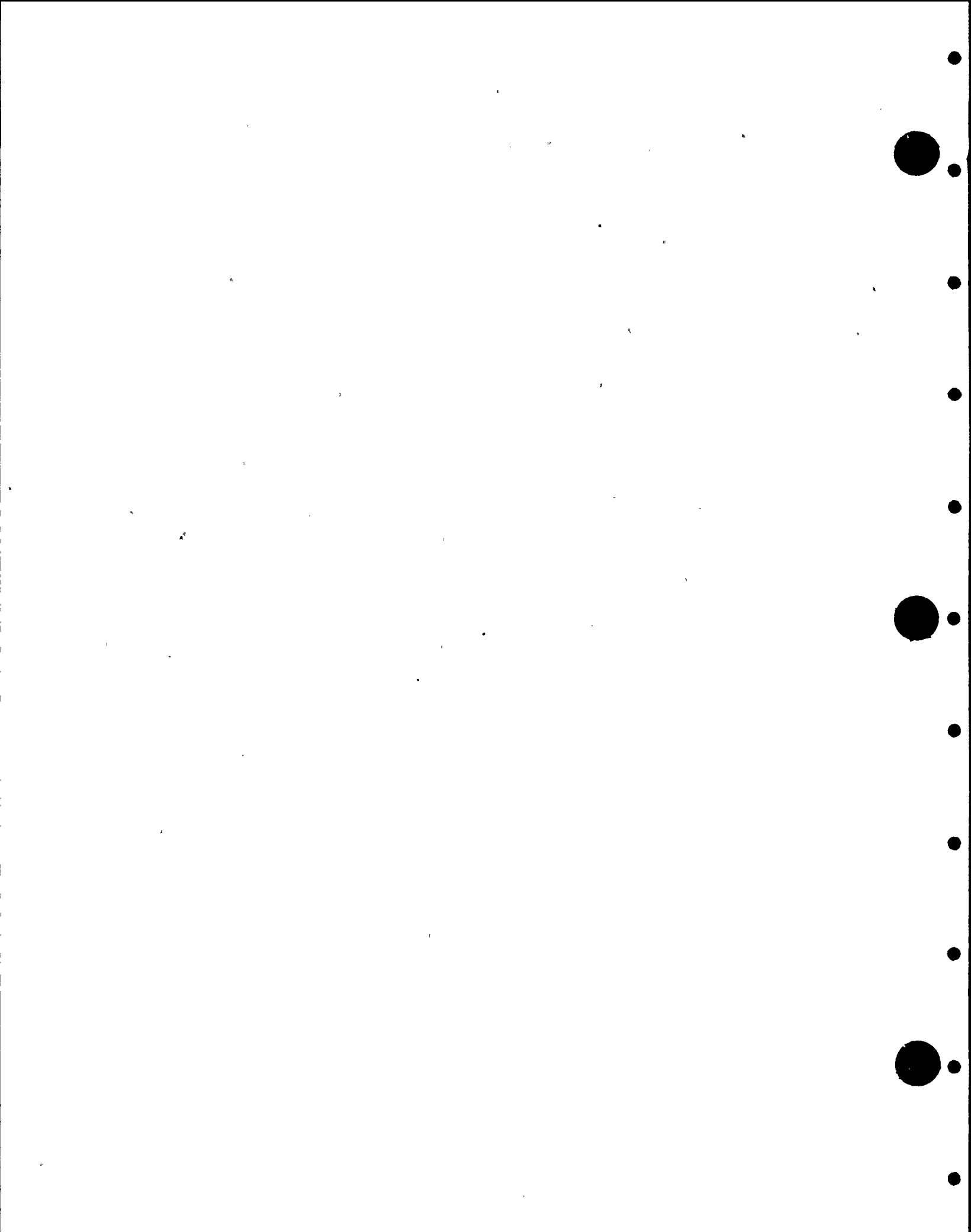
Telecon with Mike Wuestefeld,

16 APR 86

14. FJM Engineering Log Vol. II, P. 137

Coastdown of Feedpumps

Telecon with Richard Burk, 29 MAY 86



References Continued

15. Nureg/CR-2229 (EPRI NP-1783)

BWR Large Break Simulation Tests
Volume 1: Experimental Results
and Analysis, 1982

Volume 2: Appendices, 1982

16. NUREG/CR-2230 (EPRI NP-1782)

BWR Small Break Simulation Tests
With and Without Degraded ECC
Systems, 1982



WASHINGTON PUBLIC POWER
SUPPLY SYSTEM

WNP-2

CALCULATION OUTPUT SUMMARY

1. INTERFACING CALCULATIONS

Calc. No.

Rev.

How Affected

Refs. 7, 8, 9 are related to
this Calc. but not affected

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DATE

07 APR 86

PAGE

3.000

3. CALC. NO.

NE-02-86-10

4. REVISION

0

5. DOCUMENTS / DRAWINGS AFFECTED

Document No.

Rev.

Page or Section

Title

EPN

None

6. RESULTS AND CONCLUSIONS

See Pages 5.060 and 5.061

USE ADDITIONAL PAGES IF NECESSARY



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NE-02-86-10

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SUBJECT

Reactor Depressurization Using 6 Relief Valves

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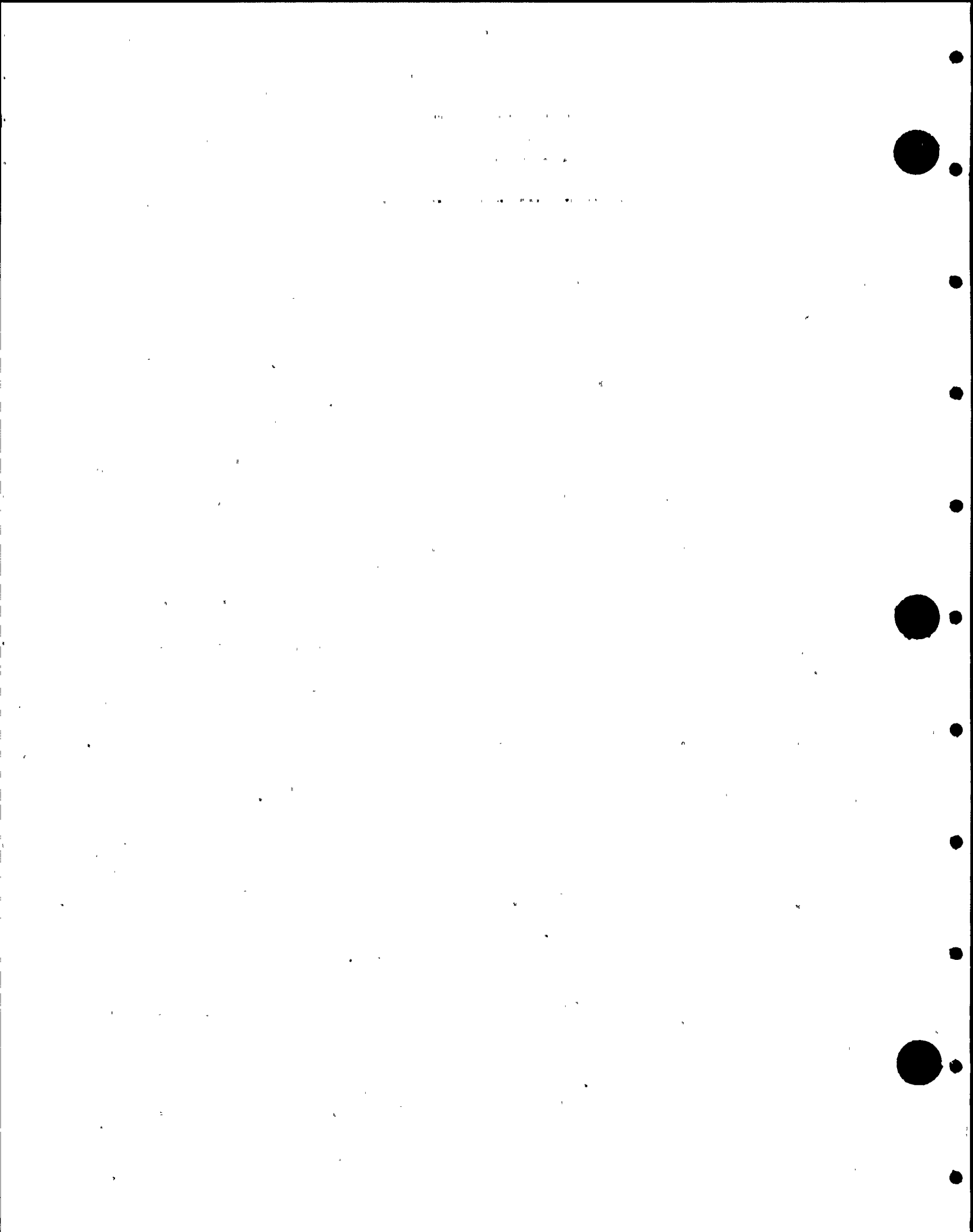
5/5/86

1. Purpose of Calculation

In the course of demonstrating compliance with applicable requirements and/or guidelines in Appendix R to Title 10, Part 50 of the Code of Federal Regulations (10 CFR 50), an Alternate Shutdown Cooling Mode (ASCM) has been developed which takes the reactor from hot standby conditions (right after the scram) directly to cold shutdown conditions by using rapid depressurization plus Low Pressure Coolant Injection (LPCI) from RHR Loop B only.

The necessity for the ASCM derives from the fact that special thermal protection (thermolagging) has not been applied to the cable trays of the HPCS and the RCIC systems, whereas the cable trays for actuation of the ~~reactor~~ ^{ADS} vessel relief valves, for RHR Loop B

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and SSW-Loop B have been thermally protected (thermolagged).

Extensive analysis of plant transient response in the ASCM (and some other extreme modes of operation) has been performed by General Electric for all of their product lines including the BWR-5 (WNP-2 is a BWR-5). This analysis is documented in NEDO-24708A (2 volumes, see Reference 6), which has been submitted to the NRC. Based on this GE analysis, G02-83-243 (Reference 2) had originally addressed the fire protection safe shutdown concern for WNP-2. However, further regulatory activity in the area of postulated hypothetical maximum fire scenario development and/or protection requirements



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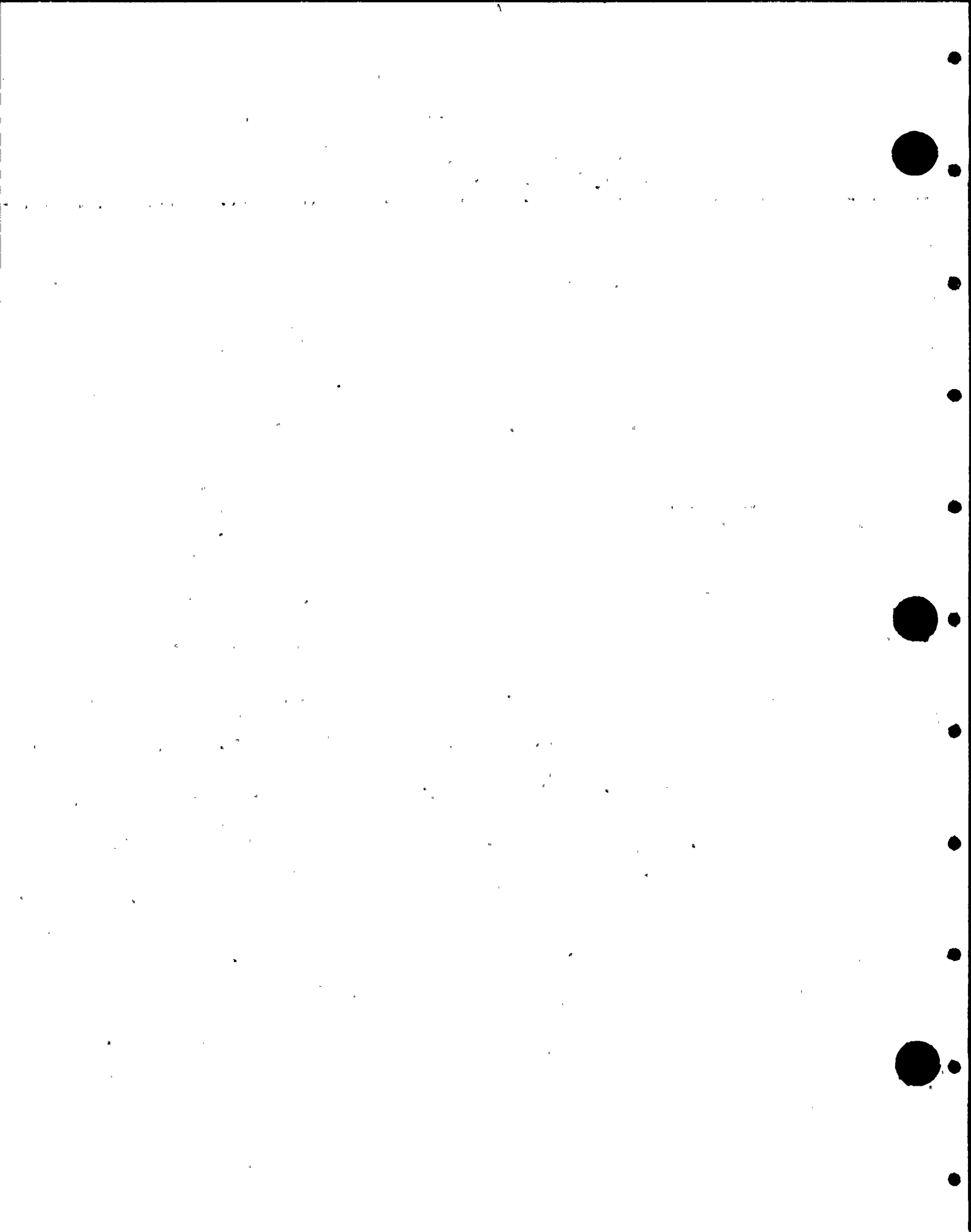
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refinement has resulted in additional guidance issued by the NRC (see References 3 and 4) as well as a requirement for WNP-2 that two plant specific cases be evaluated;

1. The maximum blowdown case, using all 18 relief valves, and
2. The minimum blowdown case, using the 3 relief valves available from the Primary Remote Shutdown Panel (PRSP).

NE-02-84-19 (Reference 7) was performed to address the maximum blowdown case, and NE-02-84-30 (Ref. 8) was performed to evaluate the minimum blowdown case.

During the first refueling outage at WNP-2, the Alternate Remote Shutdown Panel (ARSP) has been made available





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to plant operating personnel, so that now a total of 6 valves is available for remote plant control during emergency shutdown operation. The purpose of this calculation is to update the minimum blowdown case (which is the limiting one of the 2 above mentioned cases with respect to potential short term fuel undercooling) from using 3 valves to using the presently available 6 relief valves. Additionally, NE-02-84-30 (the 3 valve calculation) is based on a more simplified model which does not account for the level swell effect caused by the vapor voids in the reactor coolant. Since the level swell effect is very dominant during depressurization of saturated water (flashing occurs),



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not accounting for this effect causes such extreme conservatisms that the resulting liquid level response may well be considered unrealistically pessimistic. Consequently, this calculation does take the level swell into account and therefore presents a more realistic, yet bounding plant response for the ASCM. Also, this present calculation models explicitly the suppression pool temperature. Furthermore, the heat stored in the thermally effective structural steel, which was previously considered only for the purpose of energy balance, is now simulated in such a way that its retarding effect on pressure decrease during depressurization and on begin of LPCI is fully accounted for.



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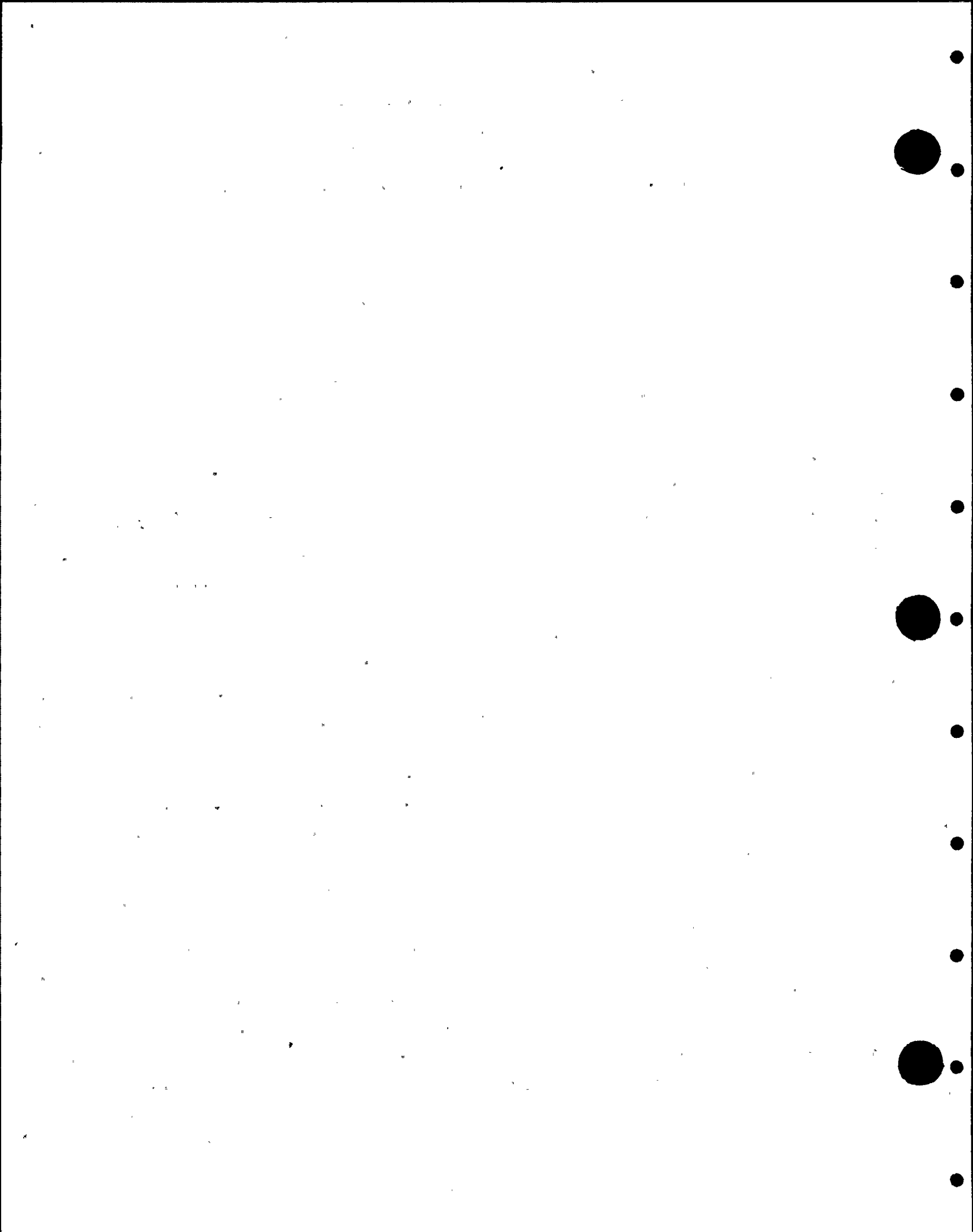
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2. Description of Analysis Model

The thermal hydraulic model consists of the reactor vessel with 18 relief valves, one single (ideally mixed or averaged) liquid mass inside the vessel, vapor voids within the liquid, vapor space above the liquid, suppression pool liquid (single node), RHR Loop B with actual RHR pump curve, plus pressure drop due to elevation difference and due to hydraulic friction in the pipe. Stored heat which is released from the thermally active steel structure during blowdown is modeled also. All of the thermally active structure is lumped into one node, and the heat release rate is calculated as described in the section on Thermal Power Calculation (see Page 5.010). The following pages give a more detailed model description.





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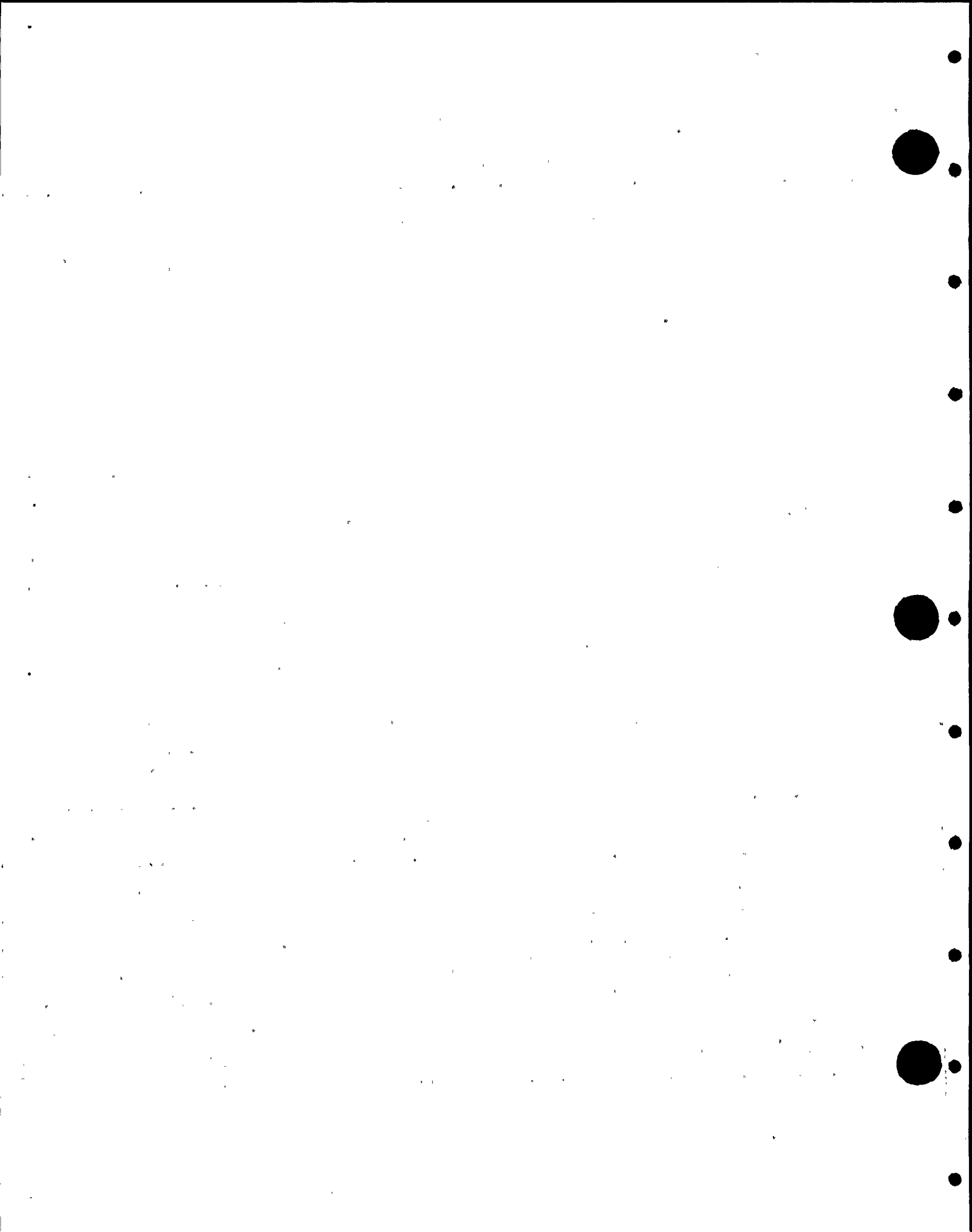
2.1 Mass and Energy Equations

The analysis model has a total of 5 point masses (mass nodes):

1. Saturated liquid in the reactor vessel,
2. Vapor bubbles inside that liquid,
3. Dry, saturated vapor above the liquid level in the vessel,
4. Structural steel, releasing heat to the liquid node during cooldown, and
5. Subcooled liquid in suppression pool.

Appendix 3 shows a listing of the computer code: part STEADY (Page A3-02) for the initialization, part TRANS (Page A3-10) for the transient calculation.

Mass 1 (reactor coolant) is defined by input: VL (Page A3-03, Line 1580) is the





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volume of the liquid, and its temperature is $TCOL$ (Line 1640). The initial vapor bubble mass (Mass 2) is defined by $VOIDFR$ (Line 1600), which is the ratio of the volume of all vapor bubbles to the volume of the reactor coolant including the bubbles. Mass 3 (dry steam) is defined as volume VVX (Line 5720) at the saturation temperature $TCOL$. Mass 4 (structural steel) is defined as $STLMAS$ (Page A3-03, Line 2400). Mass 5 (subcooled liquid in suppression pool) is defined as volume $VPPOOL$ (Line 1680) at temperature $TPPOOL$ (Line 1700).

In the transient calculation module, we have Mass 1 updated as $LMASS$ (Page A3-13, Line 5020; Page A3-14, Line 5240).

Mass 2 and Mass 3 are lumped together in $SMASS$ (Page A3-13, Line 5040).



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and Page A3-14, Line 5220). Mass 2 by itself is calculated as STMTOT (Page A3-14, Line 5680). Mass 4 is transmitted to the transient calculation module as STLMA5 (Page A3-10, Line 660), it never changes. Mass 5, the suppression pool mass, is calculated as POOLMASS (Page A3-11, Line 1800).

Energy is being tracked as it enters or leaves the reactor. SRVENG (Page A3-11, Line 1520) is the energy leaving thru the relief valves. ELPCI (Page A3-11, Line 1780) is the energy entering the vessel thru coolant injection. Suppression pool energy is updated as POOLENG (Page A3-11, Line 1820). Absorption of decay power by the reactor coolant and thermal power from cooling down the steel structure is addressed in Section 2.2, Thermal Power Calculation.



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2.2 Thermal Power Calculation

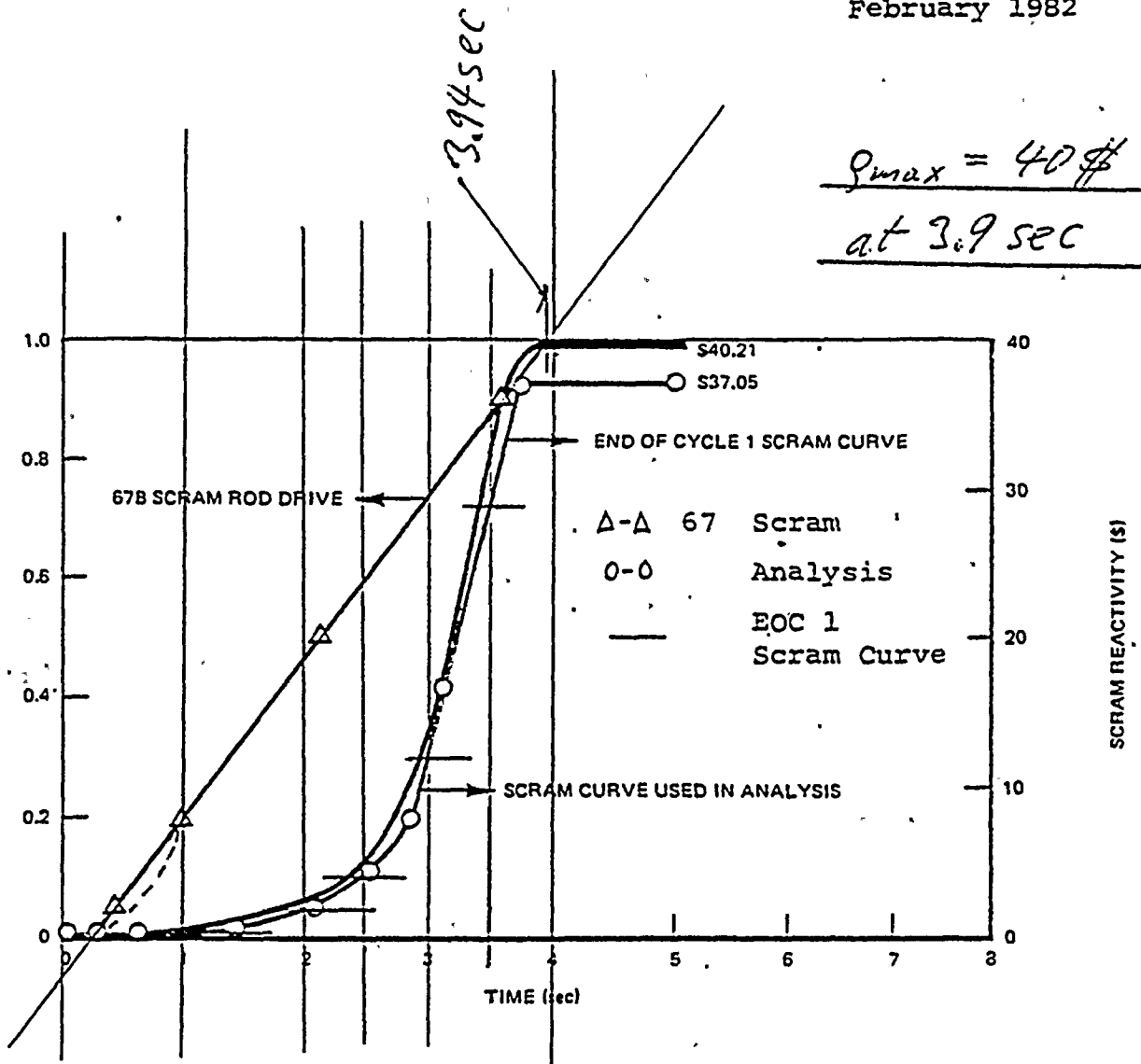
During the first 10 minutes of the transient the reactor is in hot standby. During the first 3.9 seconds of this time (which is the scram time), we have decreasing fission power as shown on the next page (Fig. 15.0-2, FSAR). From $t=3.9$ sec on we have decay power as shown on Page 5.012 (Fig. 6.2-27, FSAR). During the scram, the decreasing fission power is interpolated linearly. After completion of the scram, the decay power is interpolated logarithmically. The values for power versus time are shown on Page A3-08, Line 8140 thru Line 8360. The interpolation is performed in the decay power submodule (Page A3-14, Line 5880 thru Line 6300). A complete list of all tabulated values is shown on Pages A4-04 and A4-05.

15 MAY 86

AMENDMENT NO. 23
February 1982

For power at 3.9 sec see FSAR Figure 6.2-27

CONTROL FRACTION (PER UNIT INSERTION)

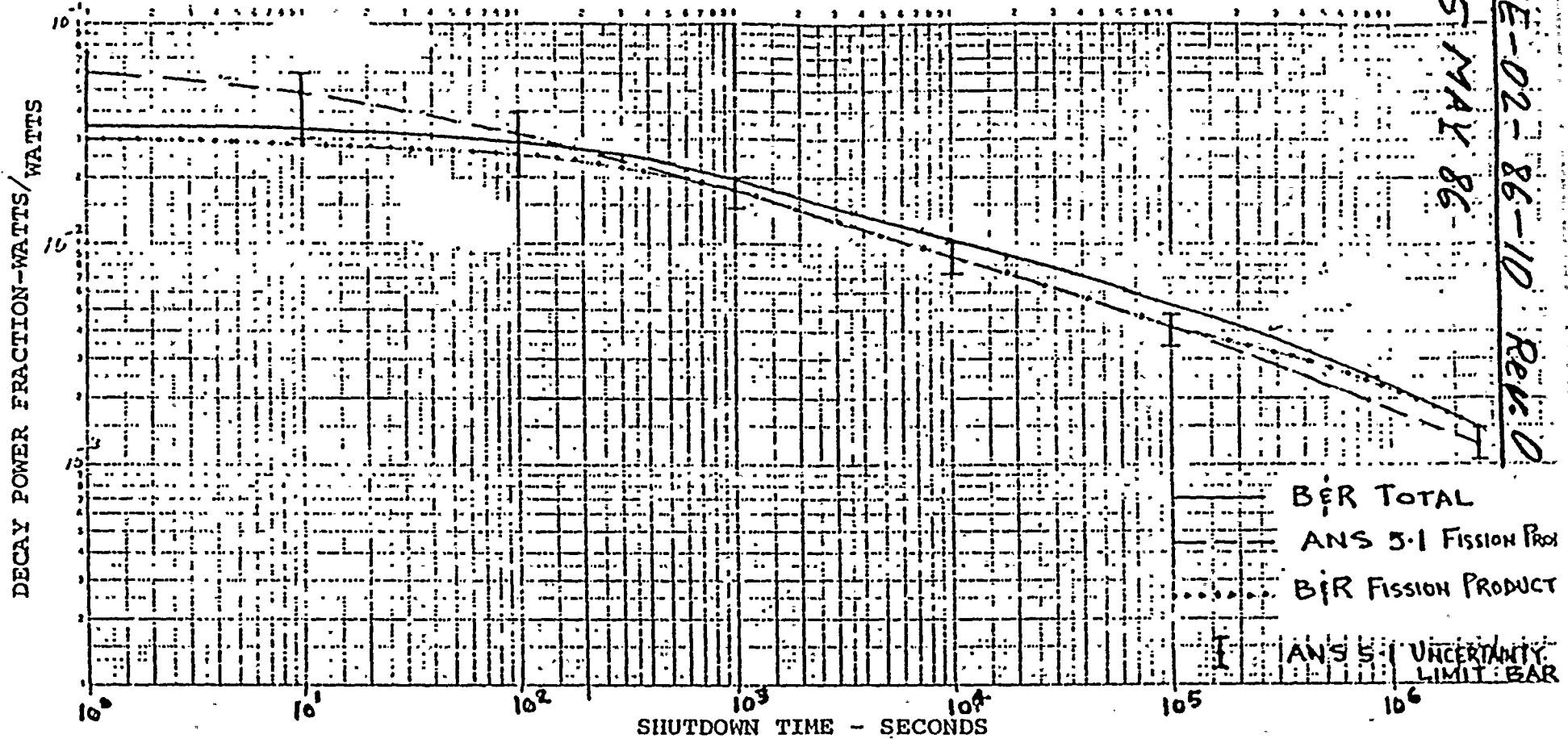


--- Revised Portion of Scram Speed Used in ODYN Analysis

($\frac{t}{\text{sec}}$)	($\frac{g}{\text{pct}}$)	($\frac{\text{Power}}{\text{pct}}$)
0	0	100
1	1.4	98.7
2	5.1	95.3
2.5	10.5	90.3
3.0	30.3	71.9
3.5	72.4	33.0
3.9	100	7.4

interpolate
linearly
between these
points

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FSAR Fig. 6.2-27

GR 5/15/82



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In addition to decay power, we have 2 more sources of thermal energy during the transient: excess enthalpy made available whenever saturated water is depressurized and stored heat in the reactor vessel wall plus the internal steel structure (core support, shroud, etc.).

The integrated decay power (over one time step ΔT) is calculated as DQ (Page A3-11, Line 1400). DQ_{NET} in the thermal state submodule (Page A3-14, Line 5180) is the net energy available for steam generation:

$$DQ_{NET} = DQ + Q_{DHF} + Q_{POOL} + Q_{STEEL} \quad \text{with}$$

DQ = decay heat, as mentioned already

$Q_{DHF} = Q_{\Delta HF}$ = excess enthalpy in liquid, released (or absorbed) during the time step (Line 5140)

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Q_{POOL} = (negative) energy added by LPCI
(pool water), see Line 5160

Q_{STEEL} = (positive or negative) energy from
changing the temperature of
structural steel, see Page A3-11,
Line 1880

Heat transfer from steel to reactor cool-
ant is not modeled explicitly. As
shown on Line 1880, the total available
heat based on the temperature differen-
ce between steel and liquid is calculated
and then multiplied by DT/TAU_{STL} where
 $DT = 0.5 \text{ sec} = \text{time step}$ (Page A3-03, Line
1820) and $TAU_{STL} = 3.0 \text{ sec} = \text{time constant}$
for transferring the available heat^(*). This
implies the assumption that all heat
available will be transferred within 3
seconds.

(*) Page A3-10, Line 960



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2.3 Steam Tables

Ideally, the authentic ASME Steam Tables as shown in Reference 11 should be stored in tabulated form in the computer code. However, this approach requires exceptionally large storage arrays. In order to keep storage requirements within workable limits, least square fits with fourth order polynomials have been chosen. The temperature in Fahrenheit is used as independent variable. Each thermal parameter is described by two curve pieces: one from 50 F to 400 F, the second from 400 F to 700 F. The polynomials are shown on Page A3-08: Line 7540 thru Line 7620 for the lower temperature region and Line 7660 thru Line 7740 for temperatures above 400 F. As Line 7500 indicates, the polynomials are plotted on semilog paper (with $\ln(T)$ instead of



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T on the x-axis). Lines 7860 thru 7940 are linear interpolation statements which eliminate discontinuities between the two curve pieces at the seam (400 F). For checking purposes, a sample from the steam tables is printed on Pages A4-03 and A4-04.



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2.4 Steam Flow Thru Relief Valves

The next Page shows a schematic illustration of a WNP-2 relief valve. The basic hysteresis curve for the valve is shown on Page 5.019. Both Figures are from the FSAR (Ref. 1). Setpoint values for the valves are taken from the FSAR, Table 5.2-2 (Ref. 1, Page 5.2-49). Opening and closing setpoints are printed out on Page A4-05. The opening pressures are:

Group 1 $(1076 + 15)$ psia = 1091 psia

Group 2 $(1086 + 15)$ psia = 1101 psia

Group 3 $(1096 + 15)$ psia = 1111 psia

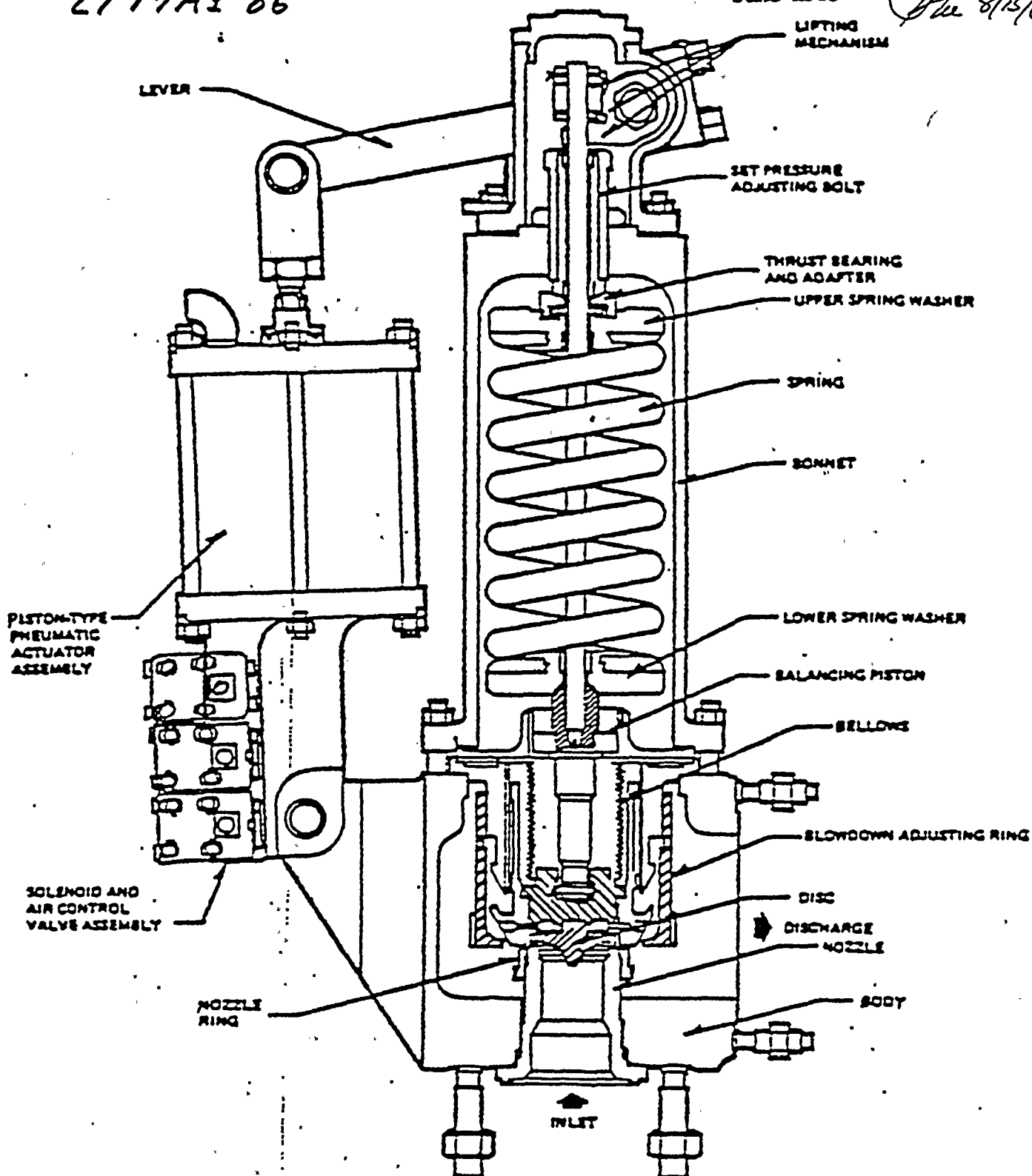
Group 4 $(1106 + 15)$ psia = 1121 psia

Group 5 $(1116 + 15)$ psia = 1131 psia

The massflow thru a valve is calculated by the basic formula

$$\dot{M} = C A \frac{S}{\sqrt{v_g}} \quad \text{in lb/sec}$$

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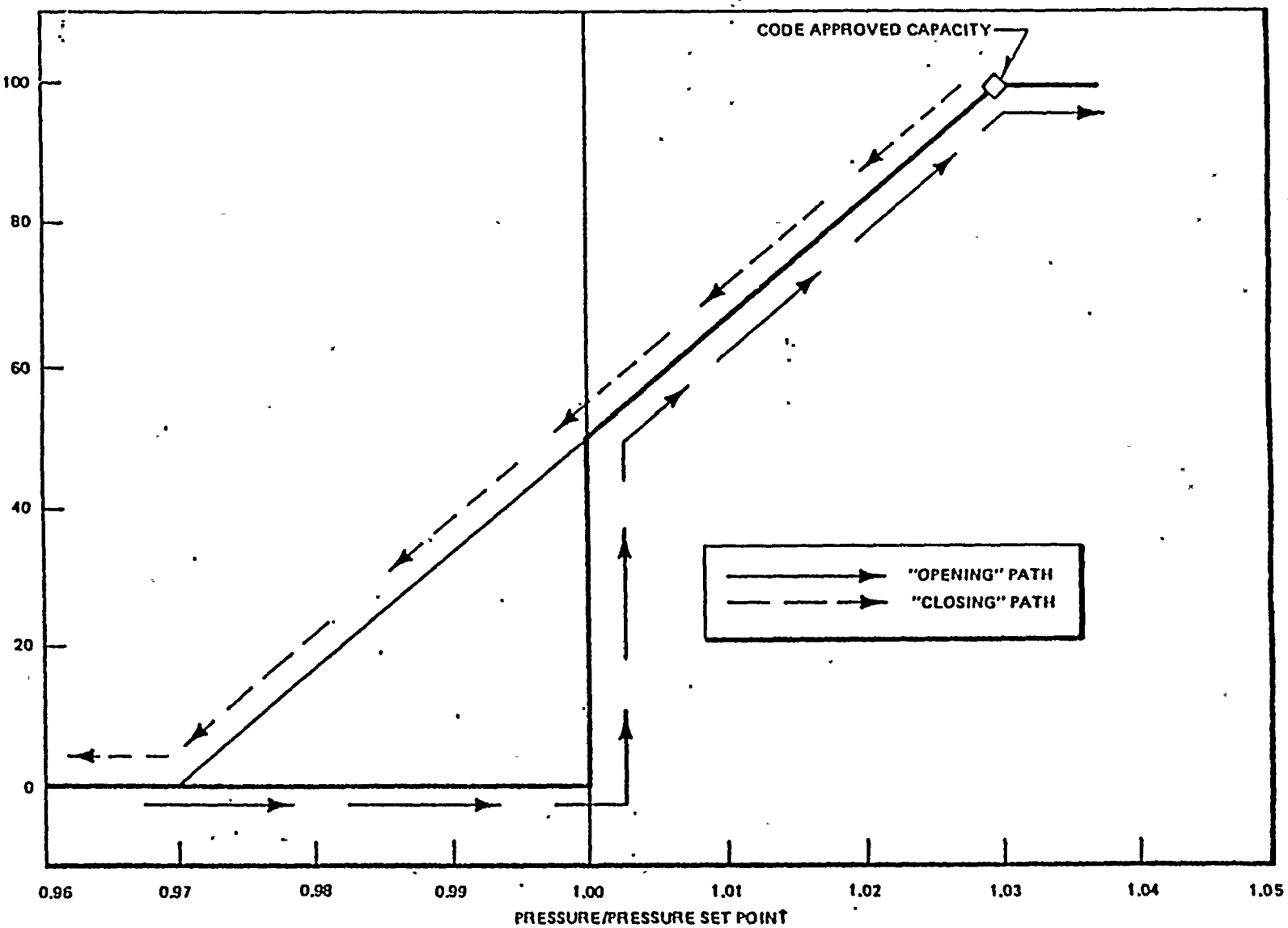


Schematic of Crosby 6R10/8R10 Dual-Function Type Spring-Loaded Direct-Acting Safety/Relief Valve

NE-02-86-10 Rev.0

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Page 5.019
The 8/15/86



WASHINGTON PUBLIC POWER SUPPLY SYSTEM
NUCLEAR PROJECT NO. 2

SIMULATED SAFETY RELIEF VALVE SPRING MODE
CHARACTERISTIC USED FOR CAPACITY-SIZING ANALYSIS

FIGURE
5.2-2



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with

 $C =$ discharge coefficient $A =$ crosssectional area in valve throat,
in ft^2 $S =$ sonic velocity in saturated steam
at upstream pressure, in ft/sec $v_g =$ specific volume of steam at up-
stream pressure, in ft^3/lb With the valve throat diameter of 4.84 in,
the throat area is

$$A = \frac{\pi}{4} \left(\frac{4.84}{12} \right)^2 \text{ft}^2 = 0.127,767 \text{ft}^2$$

According to FSAR Page S.2-49, the
relief valve has a flow capacity of

$$\dot{M} = \frac{863,900}{3,600} \frac{\text{lb}}{\text{sec}} \text{ at}$$

$$P = (1.03)(1148) \text{ psig or}$$

$$P = (1.03)(1148) + 14.7 = 1197.14 \text{ psia}$$



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At 1197.14 psia, we have from Ref. 11:

$$v_g = 0.363,462 \text{ ft}^3/\text{lb}$$

$$S = 1,449.572 \text{ ft/sec}$$

With these data we calculate the discharge coefficient:

$$\frac{863.9}{3.6} = C(0.127,767) \frac{1,449.572}{0.363,462}$$

$$C = \left(\frac{863.9}{3.6} \right) \left(\frac{0.363,462}{1,449.572} \right) \left(\frac{1}{0.127,767} \right)$$

$$C = 0.470,936$$

For simplicity we combine the two constants in the massflow formula into one:

$$K = CA = (0.470,936)(0.127,767) = 0.060,17$$

so that

$$\dot{M} = (0.060,17) \frac{S}{v_g}$$



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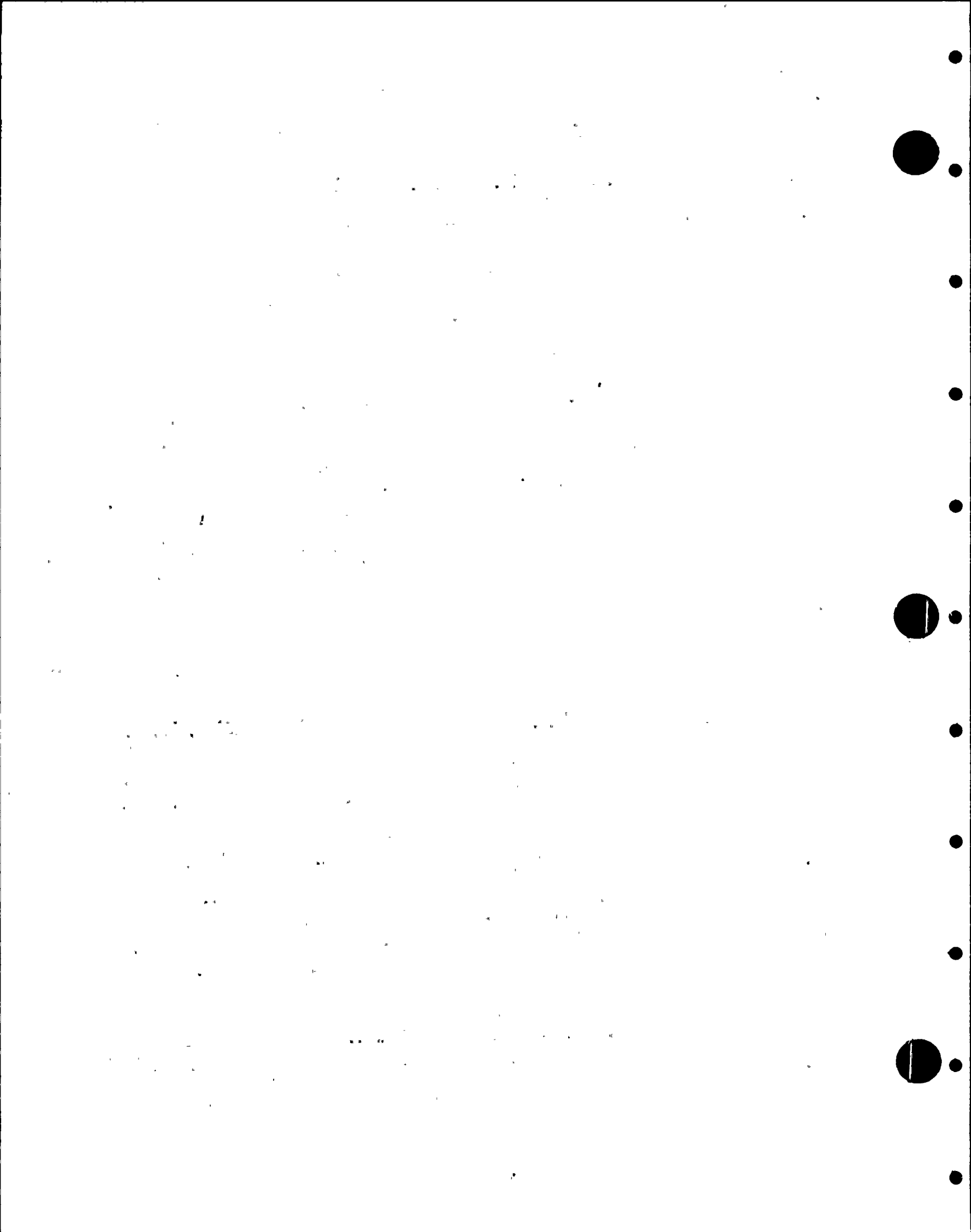
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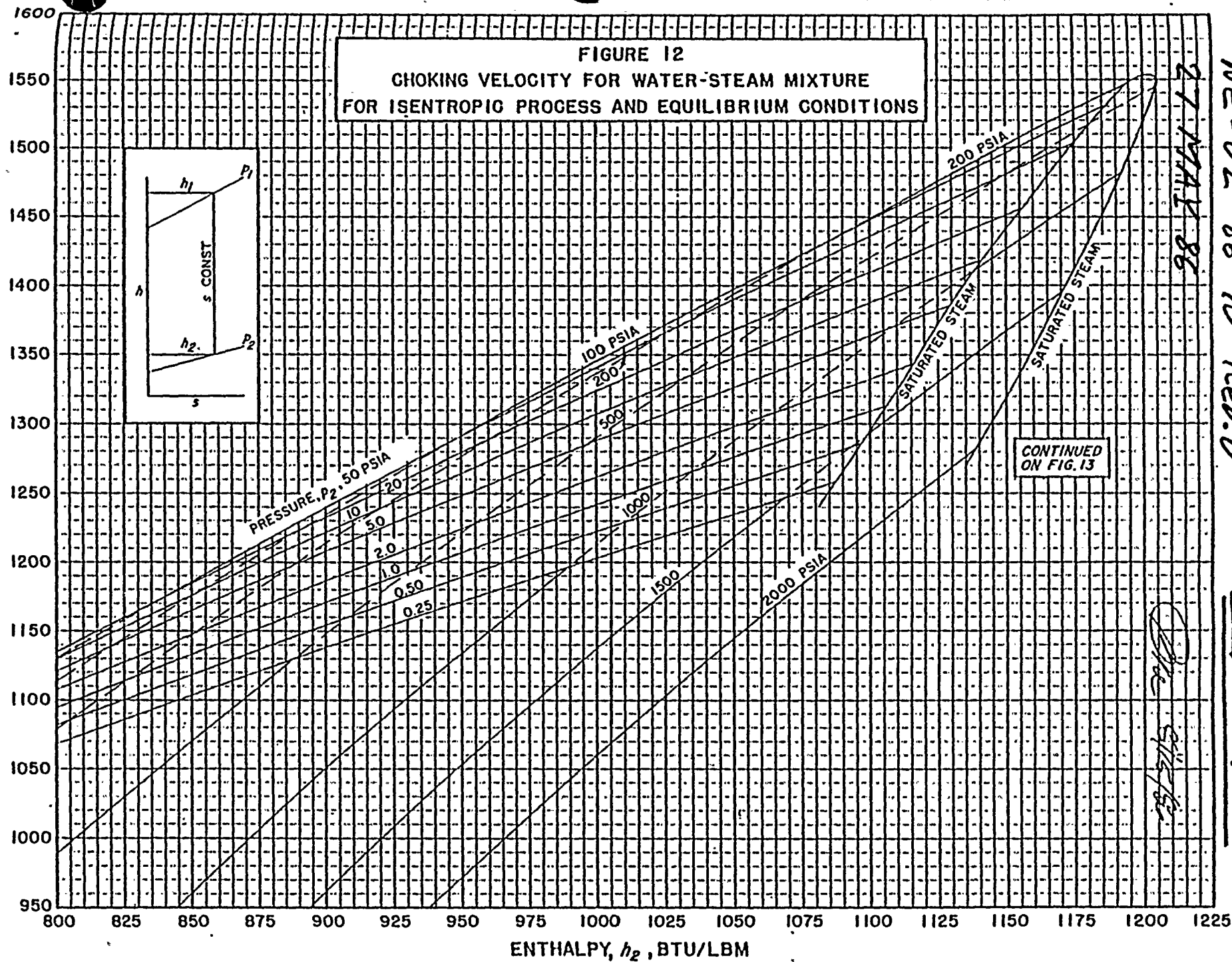
This massflow is calculated in the transient code module on Page A3-12, Line 2980 (VFLO).

The sonic velocity in saturated steam as a function of pressure is taken from the ASME Steam Tables (Ref. 11), Page 299, see next Page. The data are tabulated in the code on Page A3-09, starting at Line 8800. The table is printed out on Pages A4-05 and A4-06.

Page 5.024 shows a graph of that function.

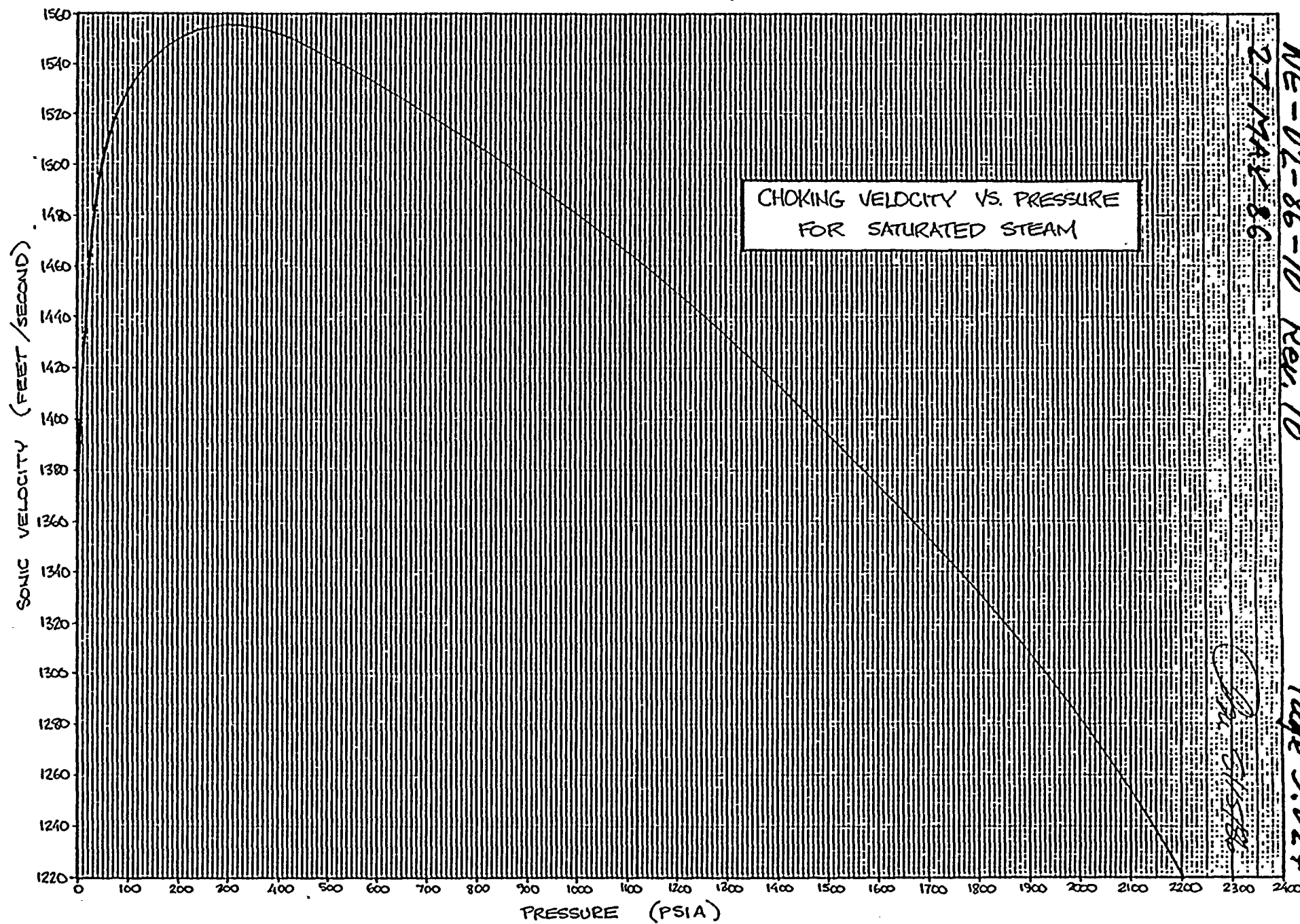


CHOKING VELOCITY, FT/SEC



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 Page 5023
 Charts



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John C. Hester

Page 5.024



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2.5 Calculation of Thermal State

The submodule for calculation of the new thermal state starts on Page A3-13, Line 4980. This submodule calculates the new thermal state at the end of each time step, based on

1. Conservation of energy
2. Conservation of mass
3. Boundary conditions

The (thermal, hydraulic and geometric) boundary conditions include valve opening setpoints, pump shutoff pressure for coolant injection, amount of decay power forced into the reactor liquid and reactor volume as a function of elevation.

The equations for conservation of mass and energy are coupled by the thermal parameters tabulated in the steam tables.



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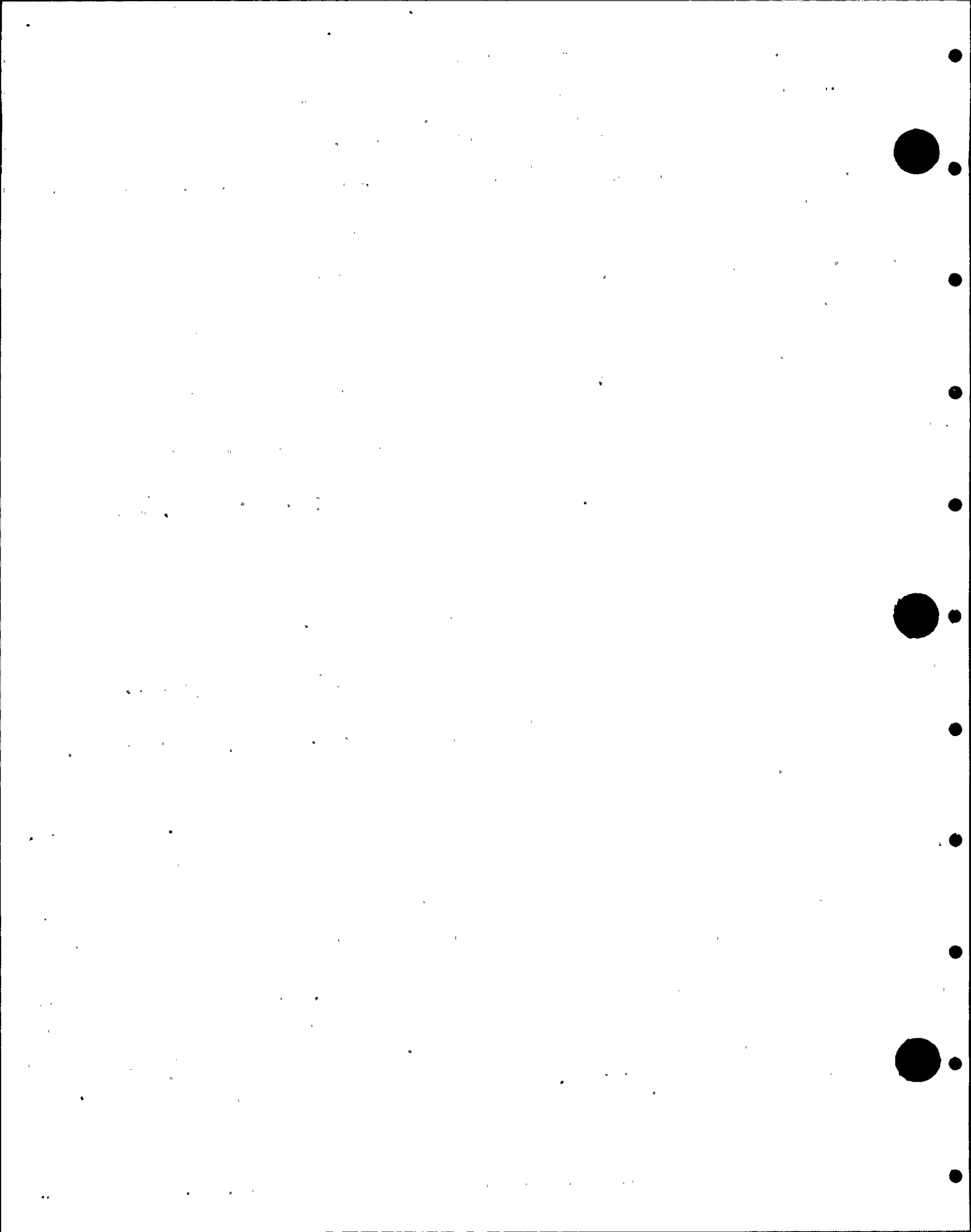
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Iterating on these equations produces the new thermal state values, provided the iteration scheme is convergent. Basically, the iteration procedure decides how much of the fluid residing in the reactor vessel is in liquid form and how much in vapor form. The iteration scheme used here utilizes the high sensitivity of the specific volume of steam (v_g) with respect to pressure or temperature.

First, the available net energy for evaporation is calculated, see Line 5180 on Page A3-14. Based on this energy and the heat of evaporation at the given pressure, the amount of new steam is calculated, Line 5200. Subtracting the evaporated mass from the liquid and ad-



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ding it to the steam, the new required volume (at the given pressure) is calculated and then compared to the geometrically available volume. Based on the volume deviation, the iteration procedure then adjusts the temperature (temperature is the independent variable in the steam tables), recalculates the net energy available for evaporation, the amount of new steam, the required total volume, and so on. The convergence criterion (see ERMAX, Lines 5000 and 5300, Pages A3-13 and A3-14) is satisfied when the volume difference between thermally required volume and geometrically available volume is less than 0.1 percent of the available volume (SYSVOL). The procedure requi-



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res typically 3 to 6 iterations to satisfy this criterion. In case the procedure should run into numerical turbulence, warnings are provided from iteration 30 thru 35, see Lines 5400 and 5420, Page A3-14. The procedure has demonstrated excellent numerical stability.

2.6 Void Fraction Calculation

When operating at full power, the reactor liquid level is controlled to stay at its normal setpoint:

$$LN = 46.963 \text{ ft}$$

above the vessel ~~level~~^{bottom}. When the reactor is scrammed, some void collapse

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occurs, resulting in a decreased liquid level. Based on operating experience (see Reference 13), the typical level decrease on scram is about 20 inch. For conservatism, we assume here 24 in = 2.0 ft of level decrease on scram. With liquid volume versus level information received from GE (Reference 12), we can tabulate the data as shown on the next page:



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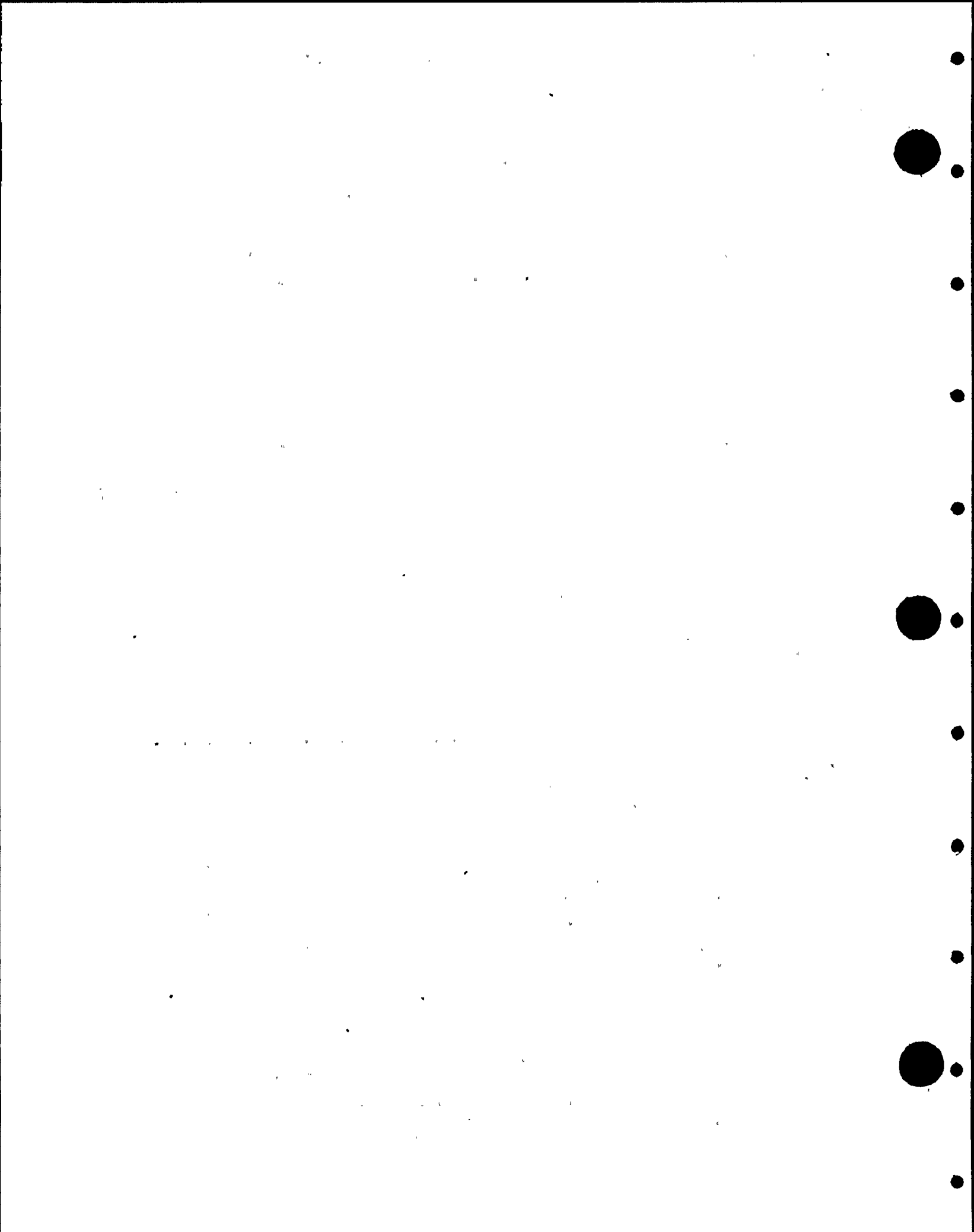
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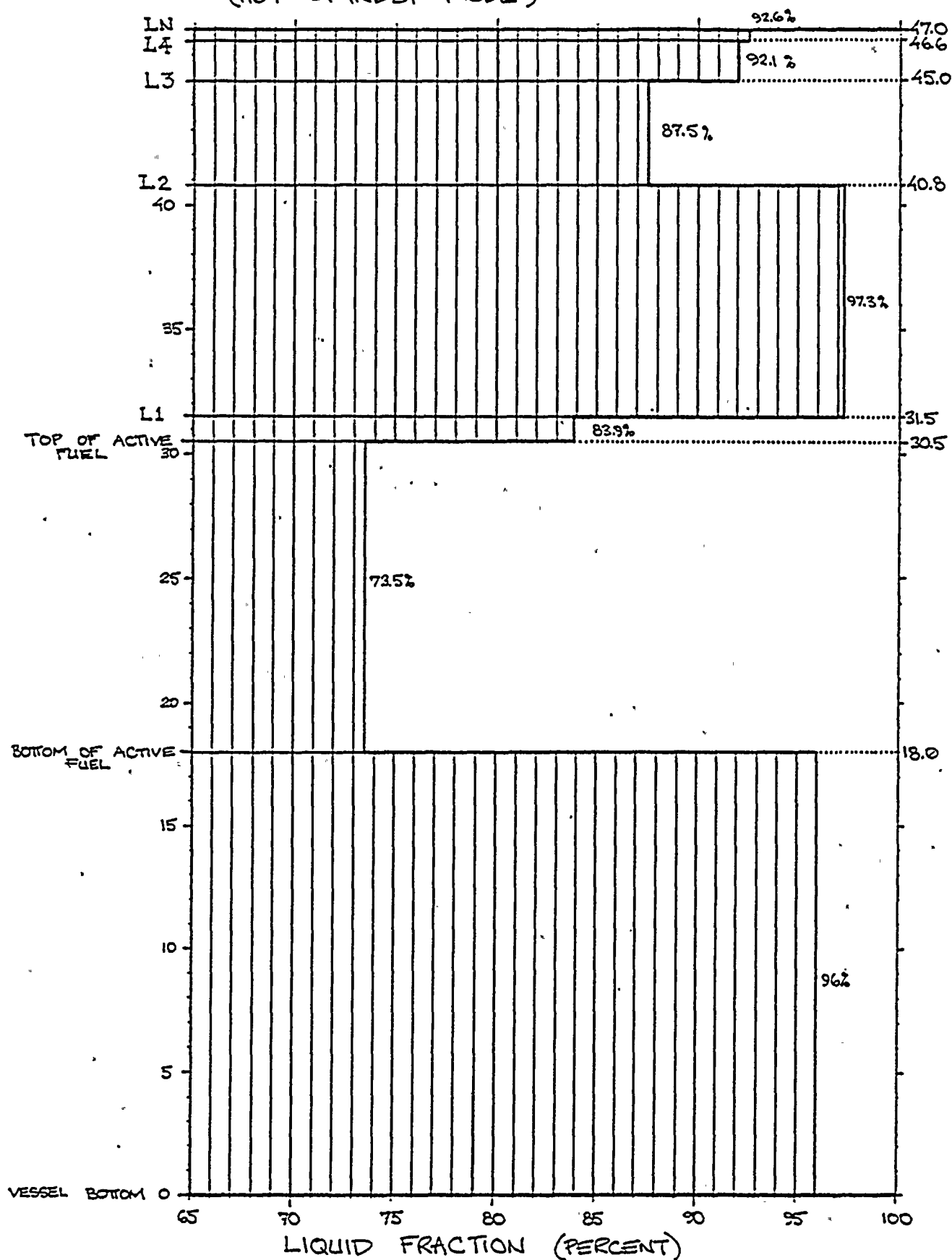
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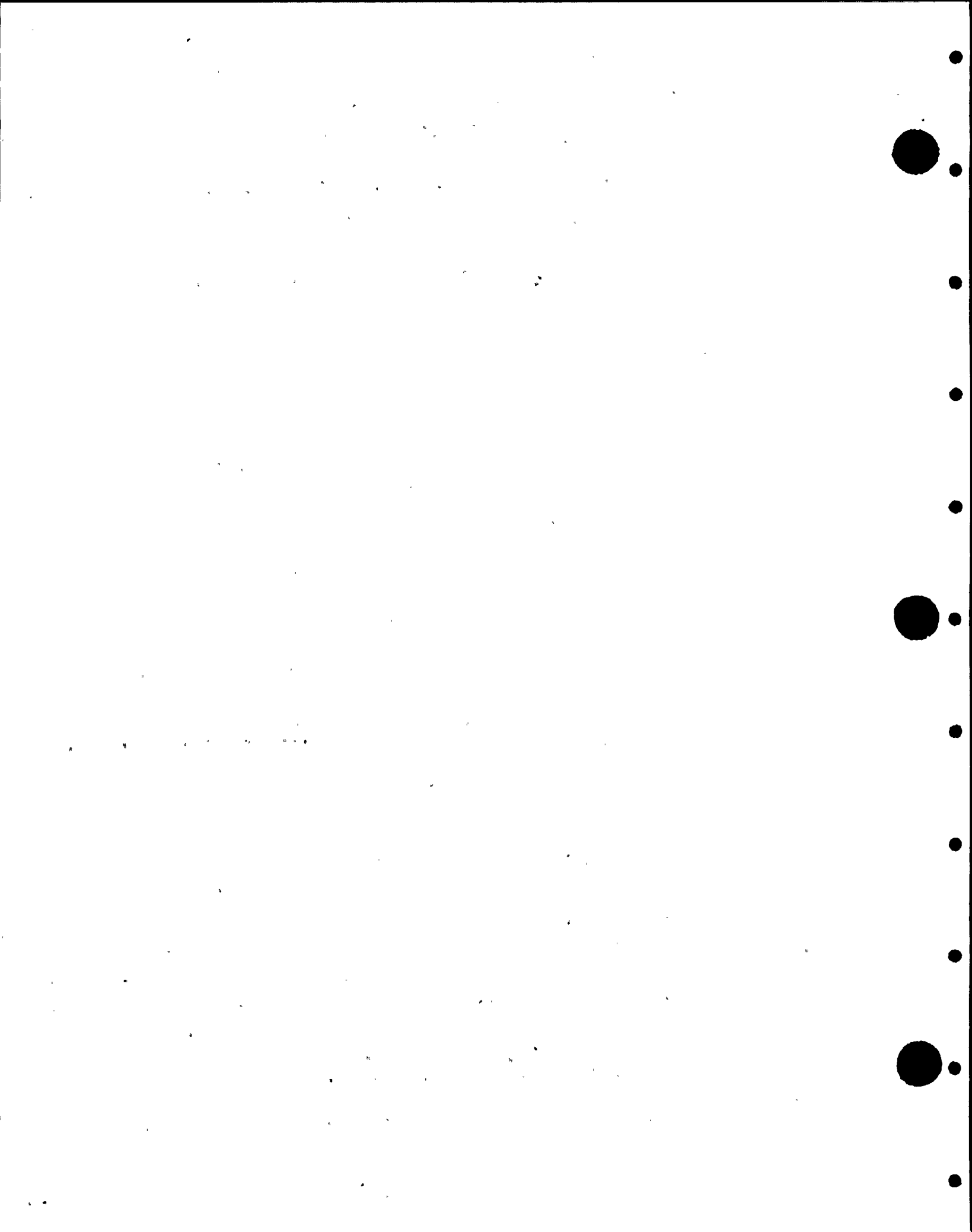
Vessel Section	Geometric Volume ft ³	Percent Liquid	Volume of Liquid only ft ³
O-BAF	4,658	96.0	4,471.7
BAF-TAF	3,725	73.5	2,737.9
TAF-L1	293	83.9	245.8
L1-L2	3,092	97.3	3,008.5
L2-L3	1,266	87.5	1,107.8
L3-L4	501	92.1	461.4
L4-LN	122	92.6	113.0
Σ	13,657	—	12,146.1

The cumulative volume data (from the vessel bottom up) with corresponding level values are printed out on Page A4-02. These data apply to hot standby conditions. The data indicate that at full power temperature (which



VOLUMETRIC LIQUID FRACTION AS A FUNCTION OF VESSEL LEVEL
(HOT STANDBY MODE)





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is the same as hot standby temperature), 12,146.1 ft³ of liquid reside in a geometric volume of 13,657 ft³. This defines the averaged void fraction for the vessel (below the liquid level) at full power operation:

$$\alpha_{FP} = \frac{13,657 - 12,146.1}{13,657} = 11.06 \text{ pct}$$

With the normal level at LN = 46.963 ft, the hot standby level is

$$L_{HS} = (46.963 - 2) \text{ ft} = 44.963 \text{ ft} \approx 45 \text{ ft} = L_3$$

From Page A4-02 we see that the geometric volume at L₃ is

$$V_{L_3} = 13,034.0 \text{ ft}^3$$

This defines the averaged void fraction.

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at hot standby conditions:

$$\alpha_{HS} = \frac{13,034 - 12,146.1}{13,034} = 6.81 \text{ pct}$$

This value is used for initialization, see Line 1600 on Page A3-03.

During the transient calculation, the void fraction is determined by the bubble release rate on one hand and by the generation rate of new bubbles on the other hand. In addition, the void fraction is linked to pressure changes: at a given bubble population, as pressure increases, bubbles get compressed, decreasing the void fraction; as pressure decreases, bubbles expand, increasing the void fraction. Decreasing pressure will also release ex-



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cess enthalpy, causing an increase in bubble generation rate. For the transient analysis model, the thermal state at any point in time is approximated by the equilibrium conditions (saturated conditions at all times). The bubble generation rate depends on the net thermal energy available for evaporation together with the specific evaporation enthalpy at the present pressure. The bubble release rate does not directly depend on pressure, temperature or bubble generation rate. It does depend on local bubble density (local void fraction), local natural circulation currents, local buoyancy effects and other complicated phenomena which are not modeled explicitly. Rather,



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we model the sum total effect of these phenomena in one single parameter, the level swell parameter. This simplification is consistent with other simplifications in the hydraulic model: local effects in three dimensional space are neglected, the liquid is lumped into one homogeneous water node, called a point node. The level swell parameter is initialized to

$$\underline{TBVBL = 4.7}$$

in Line 940, Page A3-10. This value has been derived from a series of parametric computer runs replicating transient level traces in Reference 6. Based on these level traces, the value is a lower bound estimate, that is,



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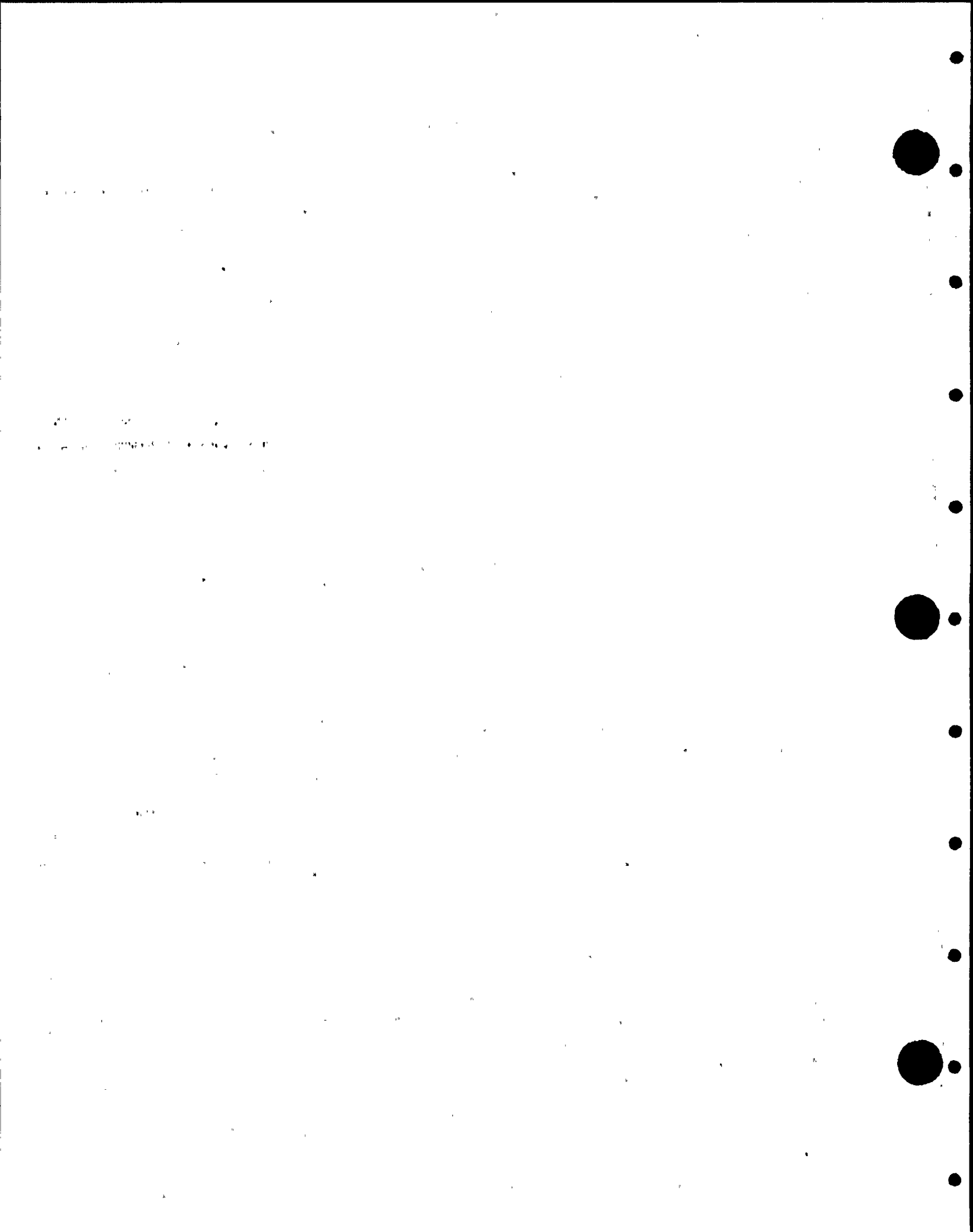
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the value has been determined to NOT be smaller than 4.7. In most cases, it produced no more than 80 percent of the level swell of the calibration trace. The submodule for transient void fraction calculation is on Page A3-14. In Line 5640 a retention factor is calculated for steam bubbles that have existed during the previous time step. In Line 5680 the total steam mass inside the liquid is calculated. It consists of a residual amount of old steam (retained from the previous time step) plus the new steam generated during the present time step. The resulting void volume (sum





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of all bubble volumes) is calculated in Line 5700, and the void fraction is determined in Line 5740.

2.7 Reactor Liquid Level Calculation

Initially the reactor is running at full power with the liquid level at its normal setpoint. As described in Section 2.6, a 2.0 ft level drop on scram is assumed (see Page 5.029). During the first 10 minutes of the transient, liquid inventory is lost thru evaporation. A first order estimate of the amount of liquid lost is the integrated decay power over 10 min, divided by the heat



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of evaporation at 1,000 psia. Actually, the
calculated amount of liquid lost is slightly higher
than that because of loss of vapor thru
the relief valves. The relief valves cycle
on and off at a rate of approximately
once per minute.

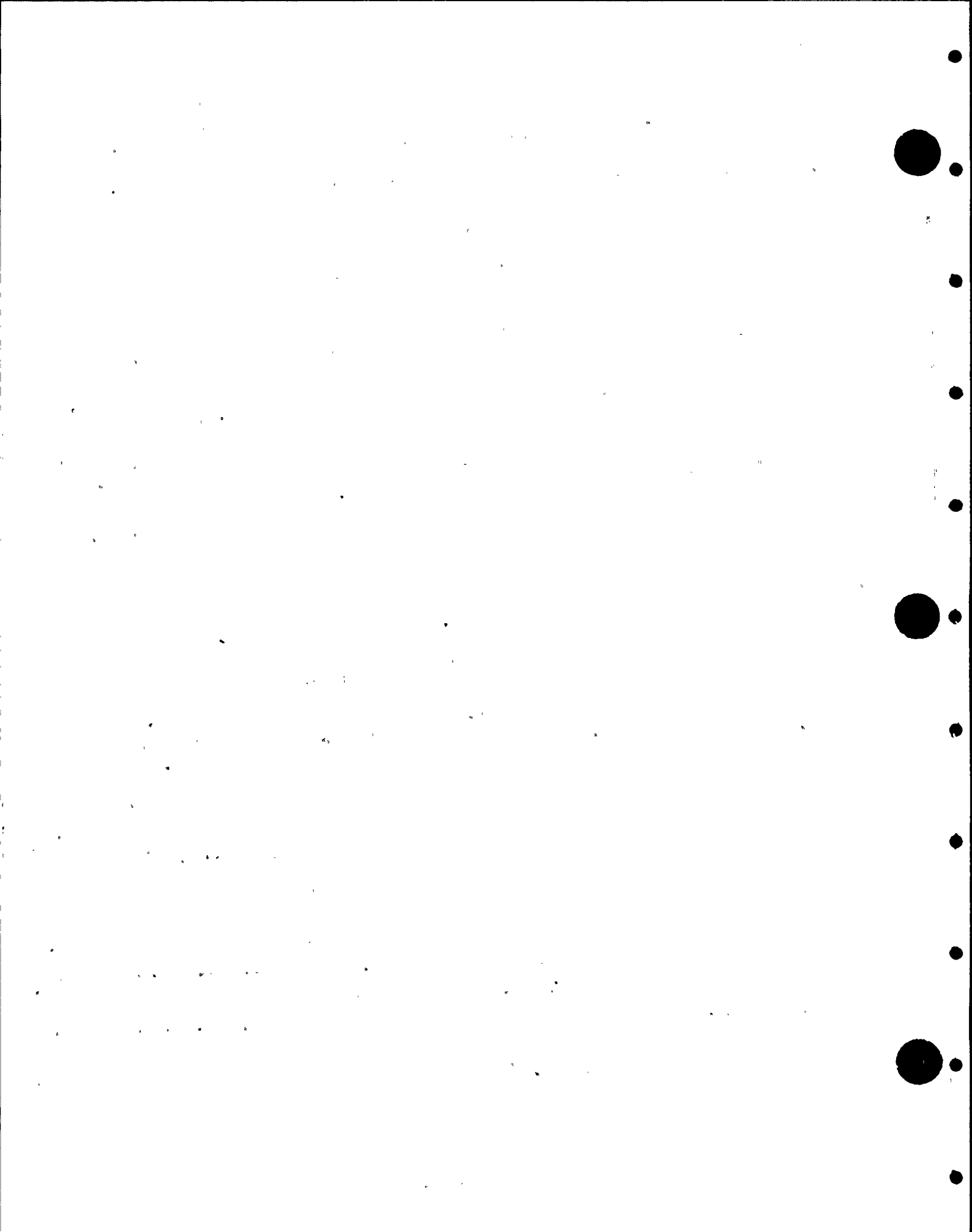
An extra conservatism has been included,
as described in Section 2.2. The
scram process itself (see Page 5.011)
has been included as part of the trans-
ient, covering the first 3.9 sec. Normally
this portion of decreasing fission power
is assumed to be offset by water injection
from the feed pumps, for which
the coastdown time is estimated at
about 15 sec.⁽¹⁾ The transient is then
started with normal water inventory

(1) see Ref. 14

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after completion of the scram (with decay power only). In our case, we left out the coastdown of the feed pumps but retained the fission heat generated during scram.

As described on Page 5.031, the liquid level is initially at 45 ft above the vessel bottom, which happens to be level setpoint L3. During the transient, the remaining liquid is calculated in the "new thermal state" submodule called on Line 1860, Page A3-11. On Line 1920, the "void fraction" submodule is called. On Line 2180, the "reactor level" submodule is called. The level submodule





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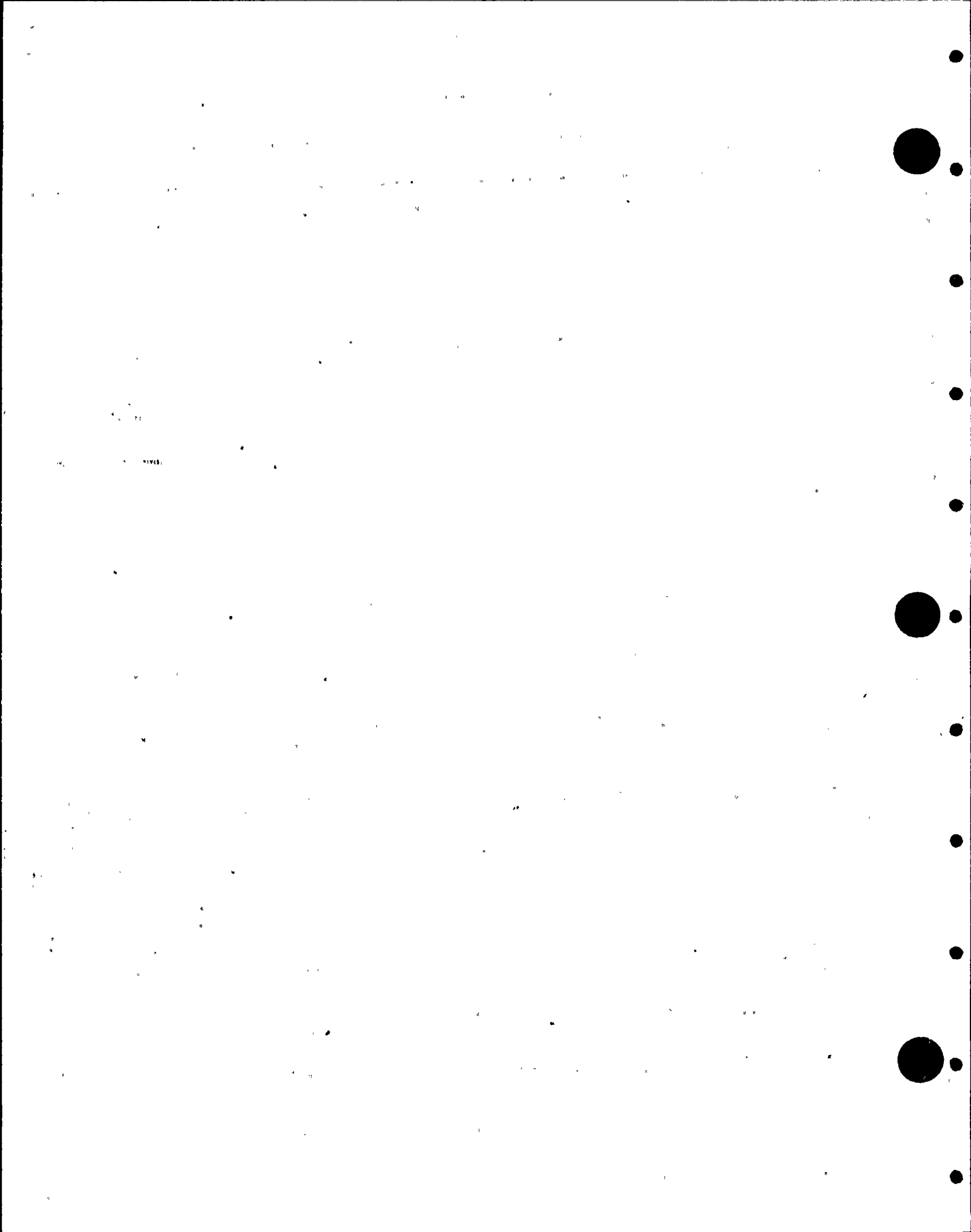
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calculates the liquid level based on the volume of liquid plus volume of voids. This is done by linearly interpolating in the volume versus level function shown as a table on Page A4-02. This function was provided by General Electric, documented in Reference 12 (the Table on Page 5.030 is from the same source).





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2.8 Peak Fuel Cladding Temperature Calculation

The peak fuel cladding temperature is not modeled in the computer code. A fairly accurate upper bound estimate can be made by using a constant heatup rate which is based on the level of decay power at the time of beginning of fuel uncover. This method is based on the rationale that over a small interval ~~of time~~ of a few minutes the decay power can be taken as a constant, provided the scram is at least 10 min away (otherwise the power decay should be taken into account). Likewise, the specific heat of the fuel pins is constant over a reason-



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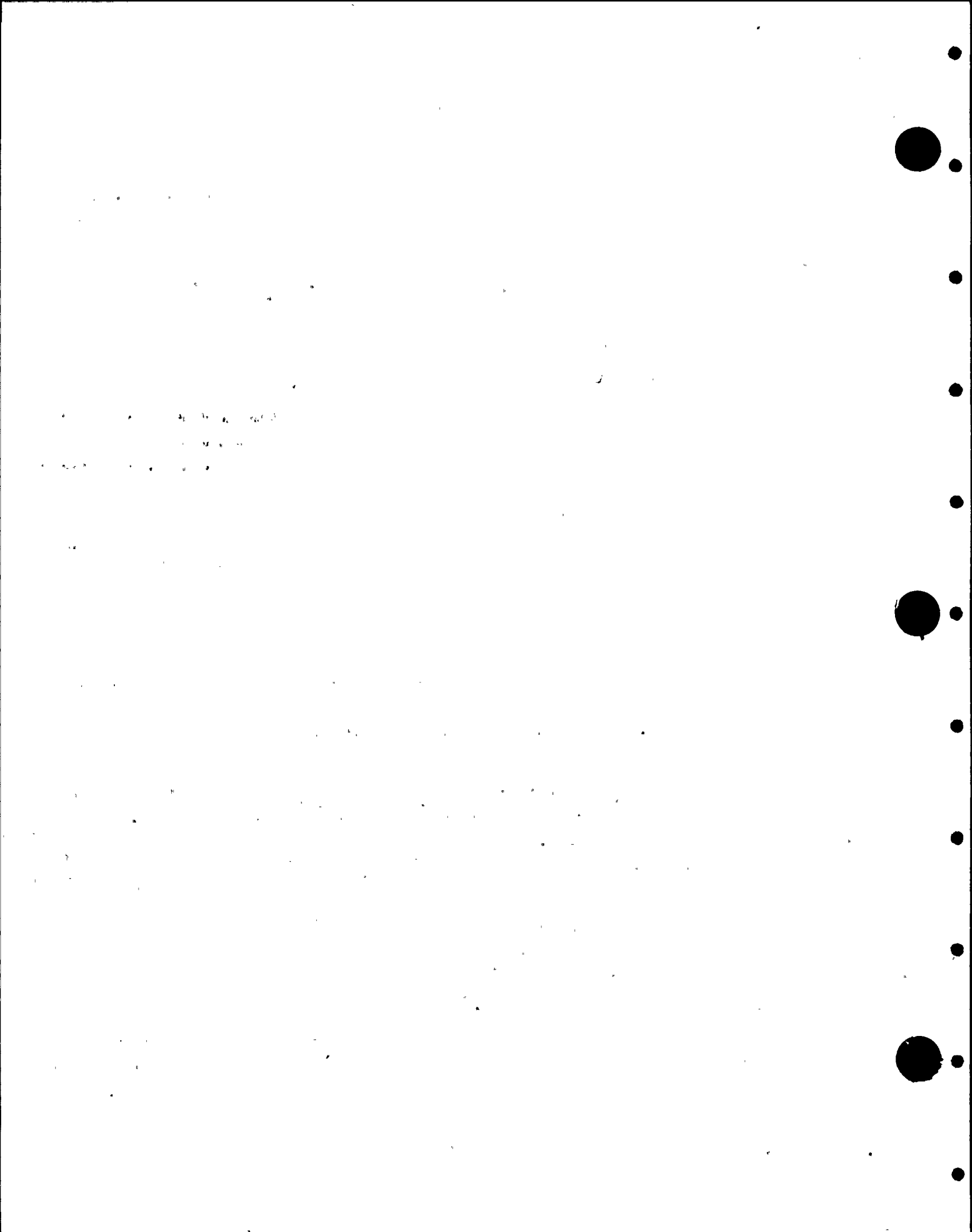
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ably limited temperature range. This results in a linear heatup function. Numerous traces in NEDO-24708A (Reference 6) support this rationale.

In the previous minimum blowdown case, a heatup rate of 2 F/sec was used (see Reference 8, Page 5.072). Appendix 1 of this calculation gives supporting documentation for this heatup rate. As the level graph (see Page 5.054) shows, the fuel uncover starts at $t = 791$ sec, and at $t = 968$ sec it is resubmerged, for a total uncover period of $\Delta t = 177$ sec. As we see from the printout (Page A4-20), the temperature of the liquid coolant





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node in the reactor vessel is

$$T_c = 434 F \text{ at } t = 791 \text{ sec}$$

Setting the outside surface peak cladding temperature equal to the coolant bulk temperature (they are actually not equal; however, the difference is amply covered by conservative margin in the estimation methodology), we estimate an upper bound peak cladding temperature at

$$T_{pc} \leq 434 F + 2 F/sec (177) sec$$

$$T_{pc} \leq (434 + 354) F = 788 F$$

This temperature is only about 200 F above the value at normal full power operation.



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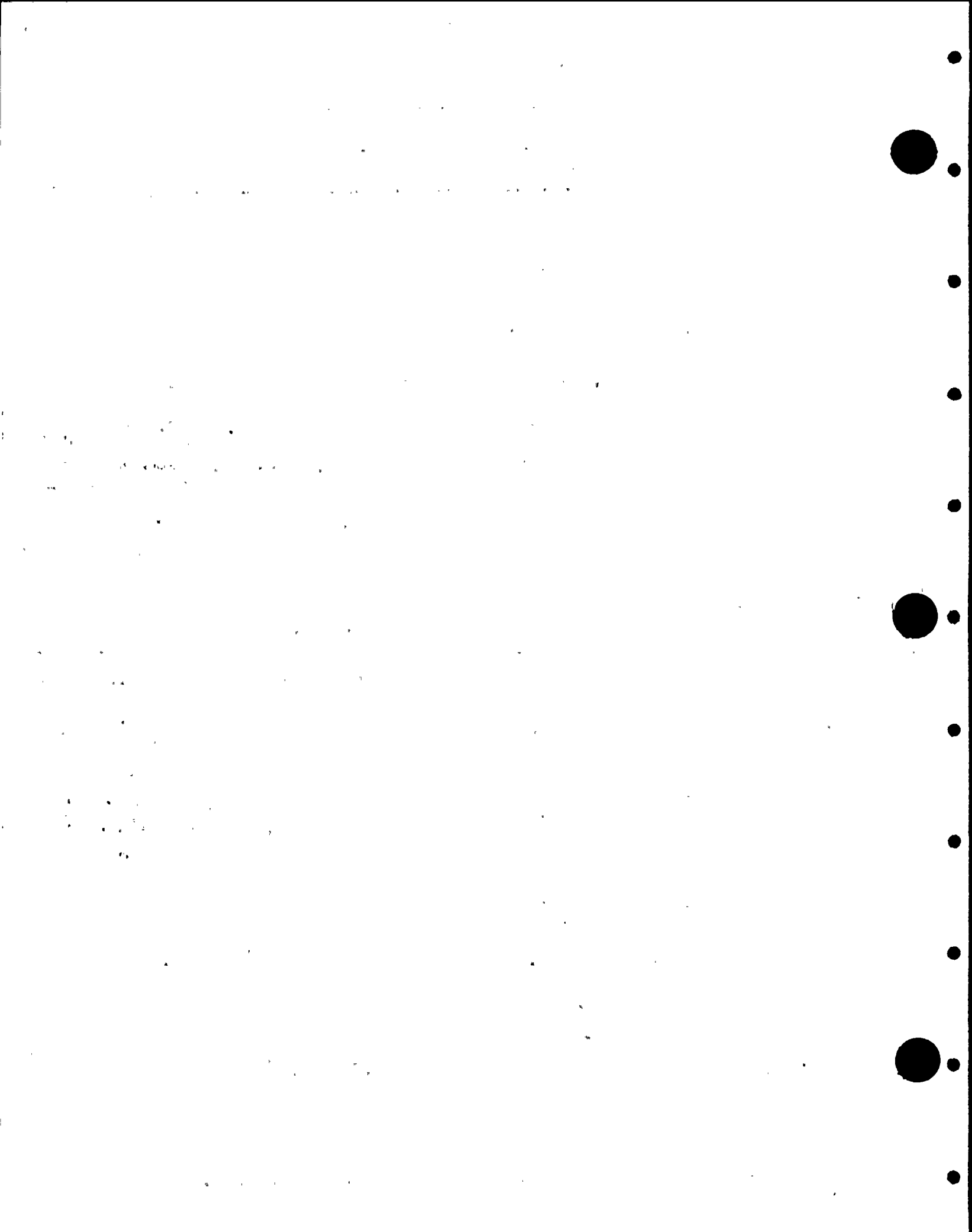
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2.9 Suppression Pool Temperature

Calculation

The suppression pool temperature is explicitly calculated in the computer model. The pool is started out at a conservatively high 90 F (normal pool temperature is 70 F), see Page A4-02. Whenever steam is released thru the relief valves, the condensation energy for that steam is deposited in the pool. As the transient calculation shows, a pool temperature of 136 F is reached at $t = 1193 \text{ sec} = 19.9 \text{ min}$ into the transient. The reactor level at that time is about 0.1 ft below level setpoint L2, increasing at a rate of about 2.5 ft/min. To reach the normal level setpoint LN would take approximately





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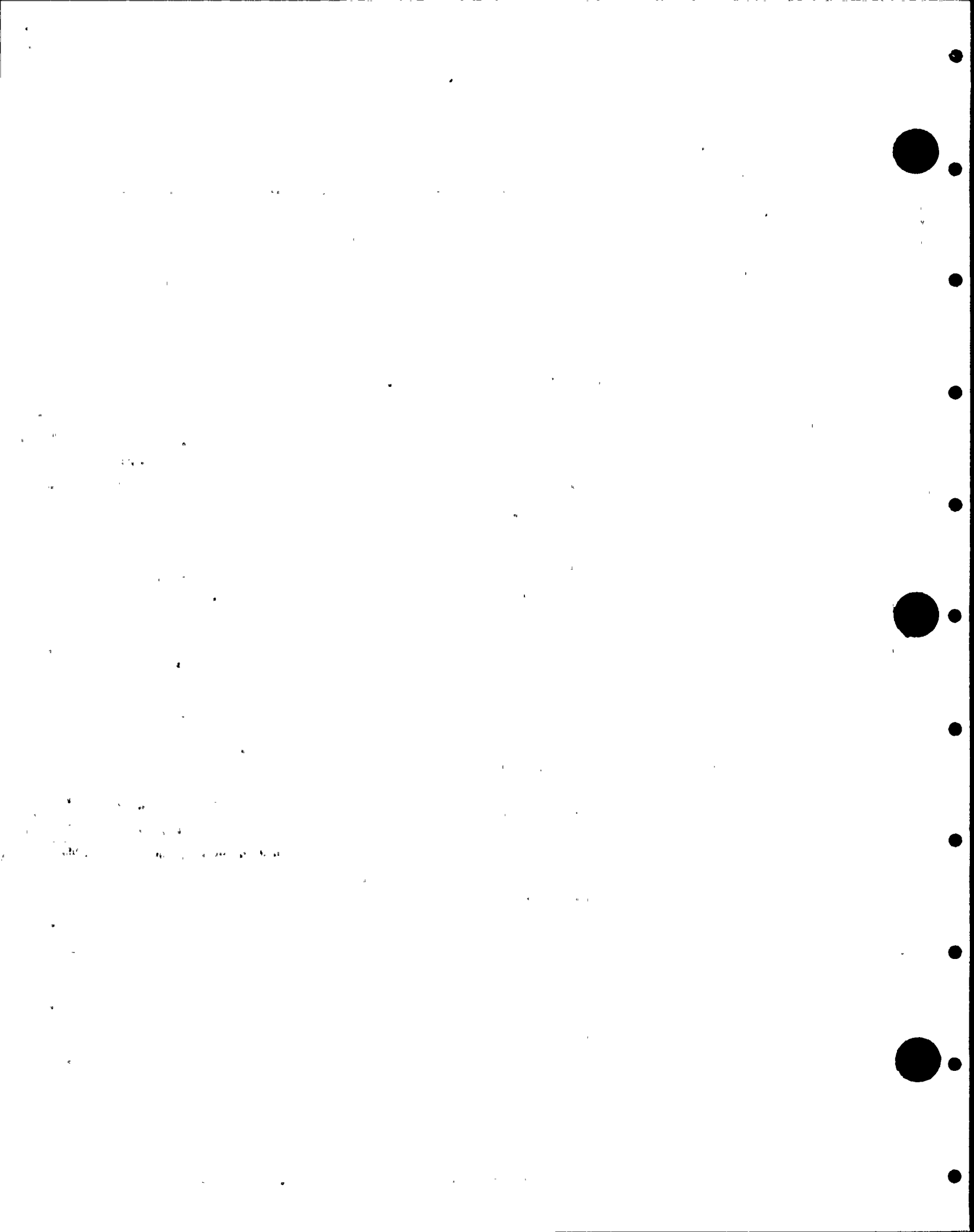
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$$\frac{(46.96 - 40.89) \text{ ft}}{2.5 \text{ ft/min}} = 2.4 \text{ min}$$

At that point in time, the plant operator will turn off the LPCI and reconfigure the RHR system for the RSC (reactor shutdown cooling) mode. The pool could gain another 2F during that time, for a total of 138F. Once the RSC mode is initiated (Loop B of the RHR system and Loop B of the SSW system are both operable from the remote shutdown stations), the decay power is removed directly from the reactor vessel, so that no further pool heatup takes place. At 138F, the pool can be left to itself until Loop A of the RHR system has been returned to operability and can be used in the





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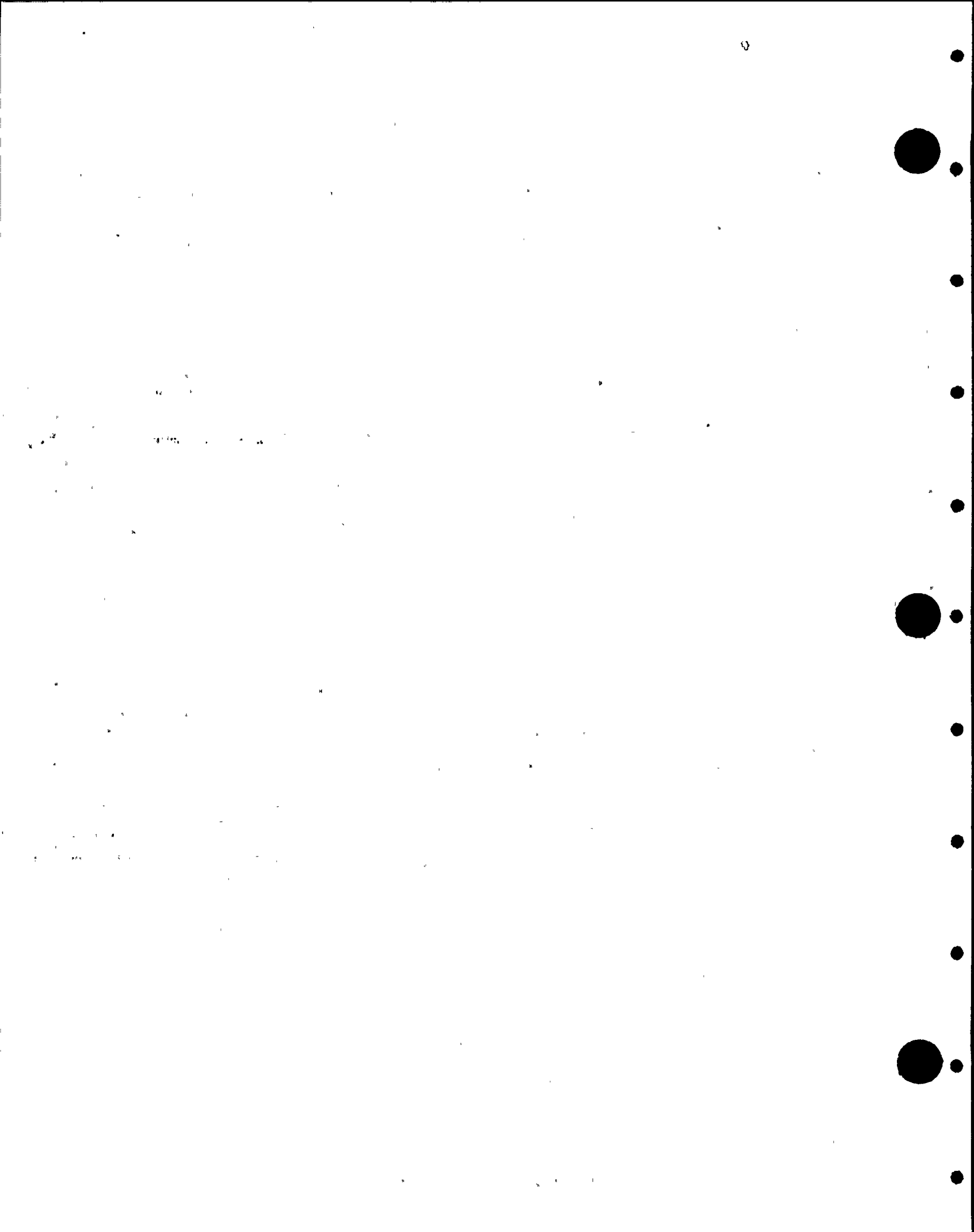
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SPC (suppression pool cooling) mode. If the repair time for Loop A of the RHR system should be longer than a week, or if the plant operator decides to use Loop A for reactor shutdown cooling also, the pool will simply cool down by itself. Concrete (plant foundation) is a fairly good ^{thermal} conductor, and the soil below the foundation is at a constant 55 F, constituting a heat sink of practically unlimited size.





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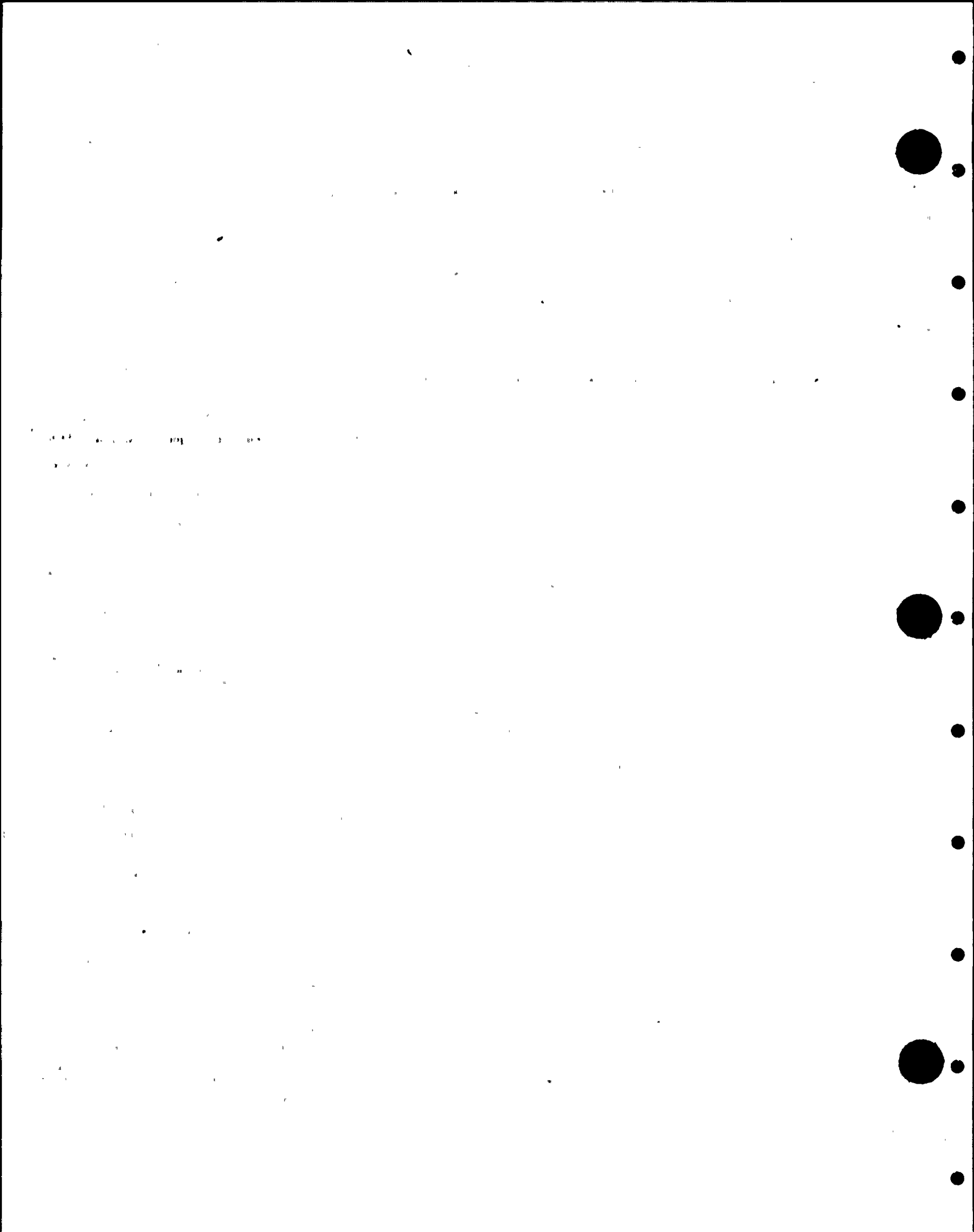
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2.10 LPCI Pump Characteristic

The next page shows performance test data for one of the RHR pumps, reproduced from Page A3-02 of NE-02-84-19 (Reference 7). For all practical purposes the three RHR pumps are identical with respect to performance.

The pump data documented in Appendix 3 of Reference 7 have been used in tabular form here, see Line 8660 on Page A3-08. For easy reference the table is printed out, see bottom of Page A4-02. Pump flow and head are calculated in the transient segment of the code, see Page A3-11, Line 1700. The pump curve submodule is on Page A3-12, Line 3440. The iteration in this submodule is required because the flow



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PROPOSAL NO. **5527-1578**

SPECIAL NOTES: **SA-047011**

TESTED IN CRACK SPALL

DESIGN CONDITIONS

GPM **7450** EFF **—**
 T.H. (FT) **230** DHP **—** SG. **1.0**
 RPM **1770** DRIVER **800 HP MOTOR**

RHR Pump

CURVE **N-623-1.0**

PUMP **29 APRD-5**

DRAWN BY **D.A.W.** DATE **12-26-74**

Head
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 0.04
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 0.02
 0.01
 0.00

NOTES:
 ○ PERF. DATA FOR HEAD, FLOW
 EFFICIENCY & BHP
 △ NPSH TEST BASIS 11/2" H. HEAD
 CONSTANT AT APPROX. 31.6 FT.
 REF. TO C.L. SUCTION NOZZLE
 □ NPSHR FROM 7850 GPM TO RUNOUT,
 REF. TO C.L. SUCTION NOZZLE

NPSHA @ C.L.
 SUCTION NOZZLE

THIS CERTIFIES THAT THIS CURVE
 IS BASED UPON ACTUAL TEST
 PERFORMANCE

Handwritten signature

SYSTEM
 RESISTANCE

NPSHR @ C.L. SUCTION
 NOZZLE

GALLONS PER MINUTE X 1000

6 Kgal/min

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Page 5.047

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SUBJECT

Reactor Depressurization Using 6 Relief Valves

PREPARED BY

F.J. Markowski

DATE

29 MAY 86

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[Signature]

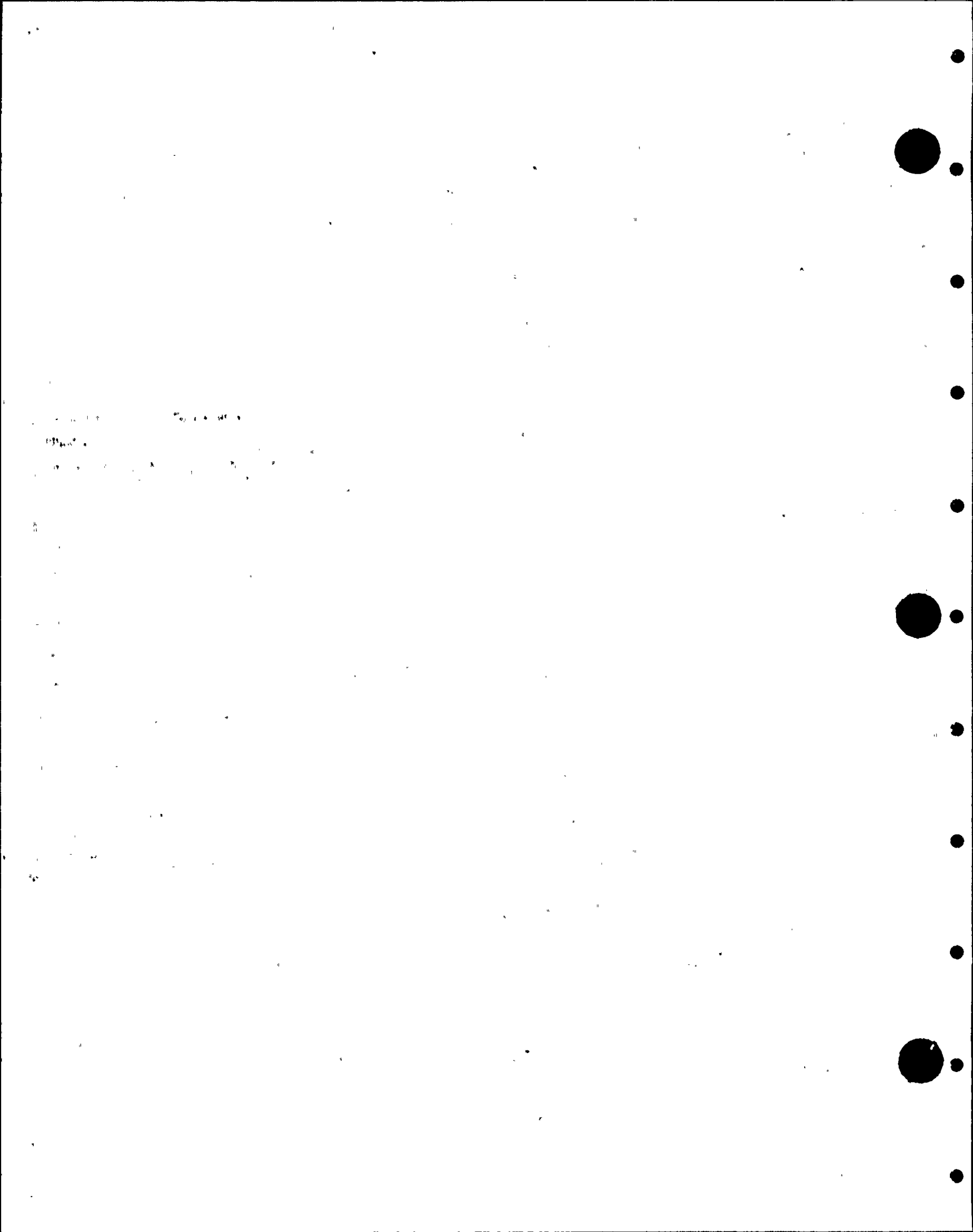
DATE

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is treated as the dependent variable, and because the hydraulic friction losses are taken into account. In each iteration, the last used value for ΔP_{Fric} is used as the first estimate, and the ΔP gets adjusted (as function of the flow) until it changes by less than 0.1 percent, see Line 3640.

The tabulation of pump head versus flow is dense enough that linear interpolation (see Lines 3580, 3600) provides satisfactory accuracy.

The printout of injection flow during the transient is shown in Appendix 4 starting on Page A4-21; column on right hand side.





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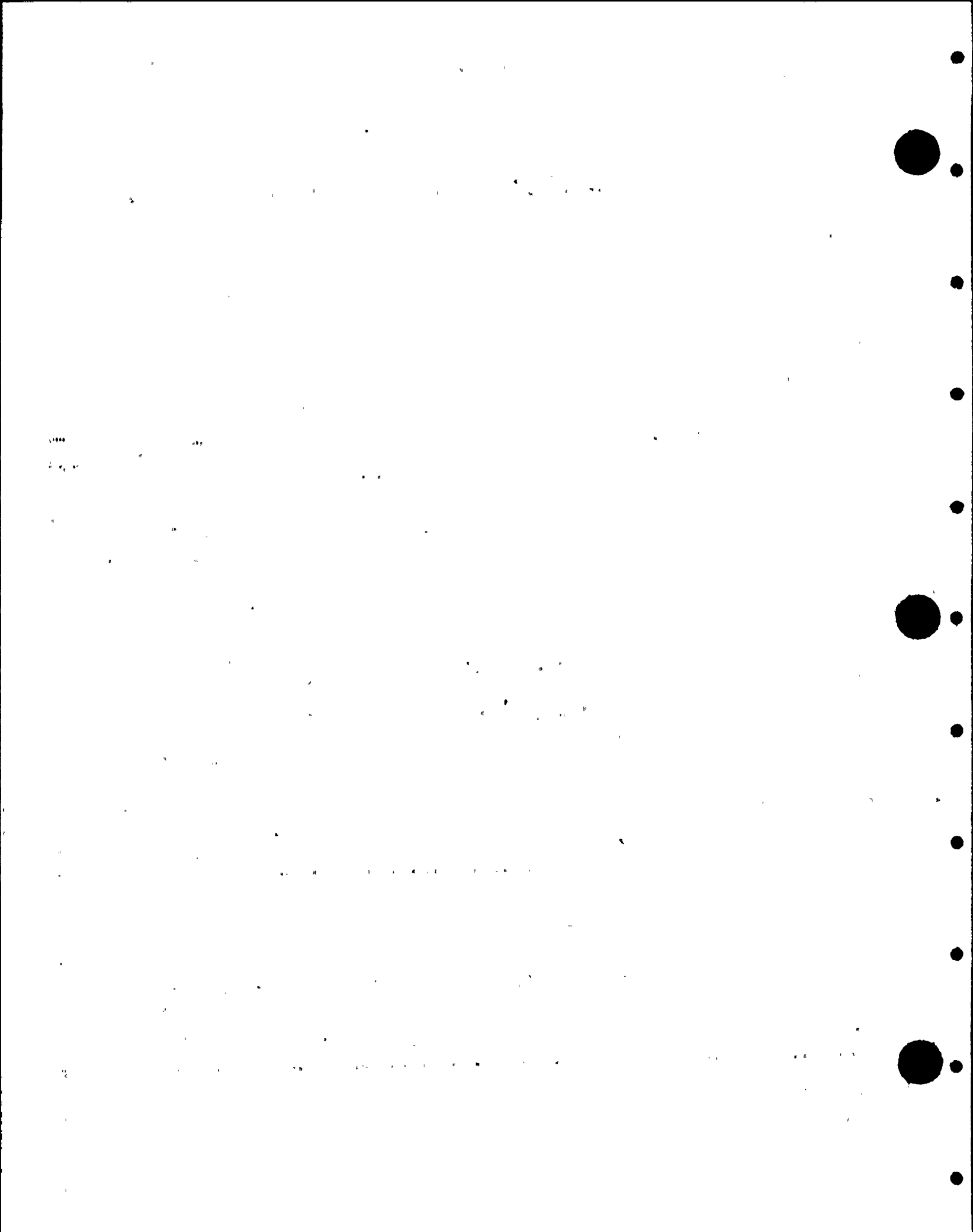
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2.11 Pressure Drop in the LPCI Line

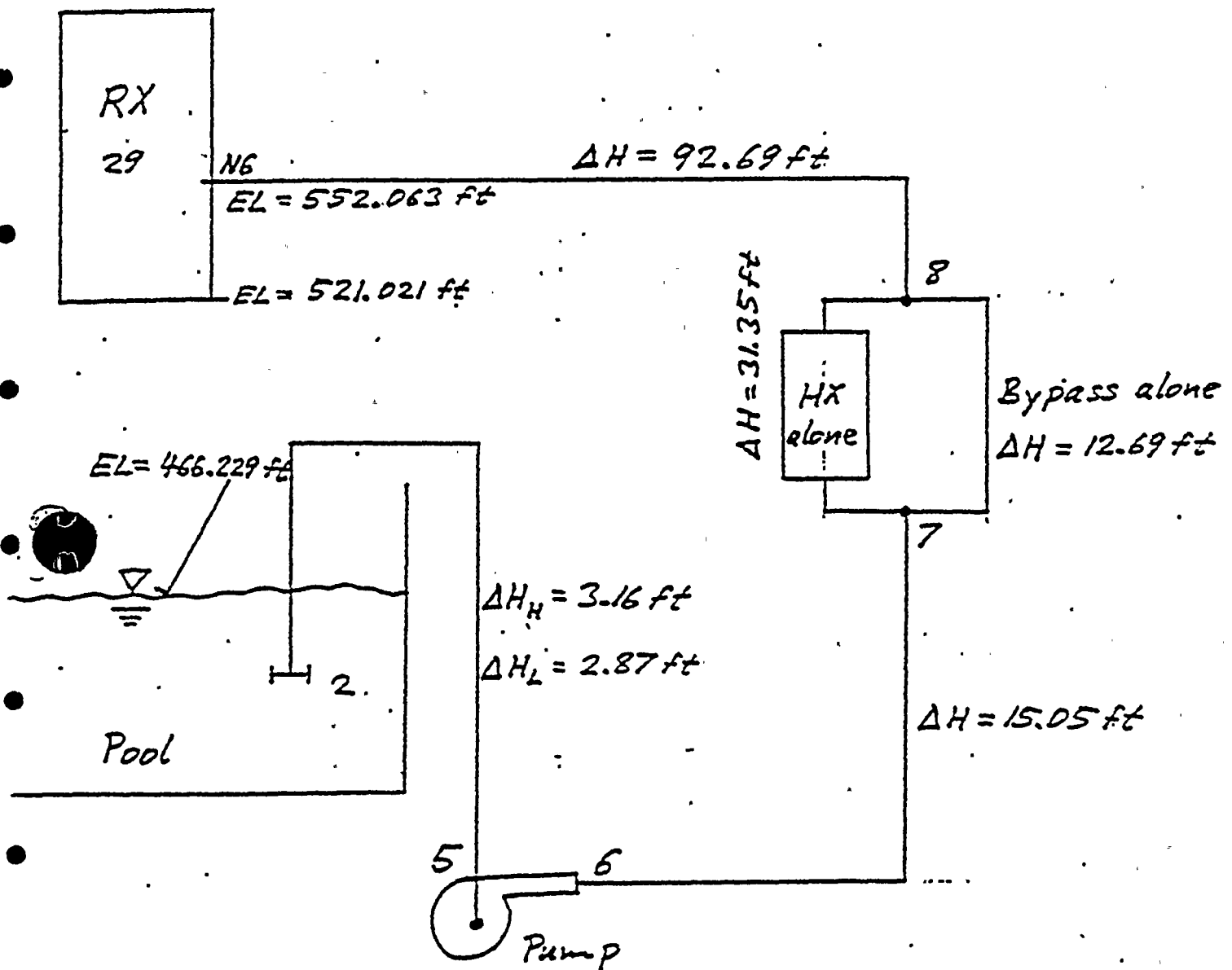
Both kinds of hydraulic pressure drop are modeled in the LPCI line: the piezometric head loss caused by elevation differences and the hydraulic friction drop, which is proportional to the square of the volumetric flow. Calculation NE-02-84-31 (Reference 9) was performed for the purpose of determining a best estimate value for the hydraulic pressure loss along Loop B of the RHR system in the LPCI mode. Page 5.017 of Ref. 9 is reproduced on the next page. Line 1640 on Page A3-11 utilizes the elevation figures from Page 5.050, it calculates DELP as the piezometric pressure difference between pool level and vessel level. Line 1660 (Page A3-11) adds into it the pressure difference between vessel and containment.

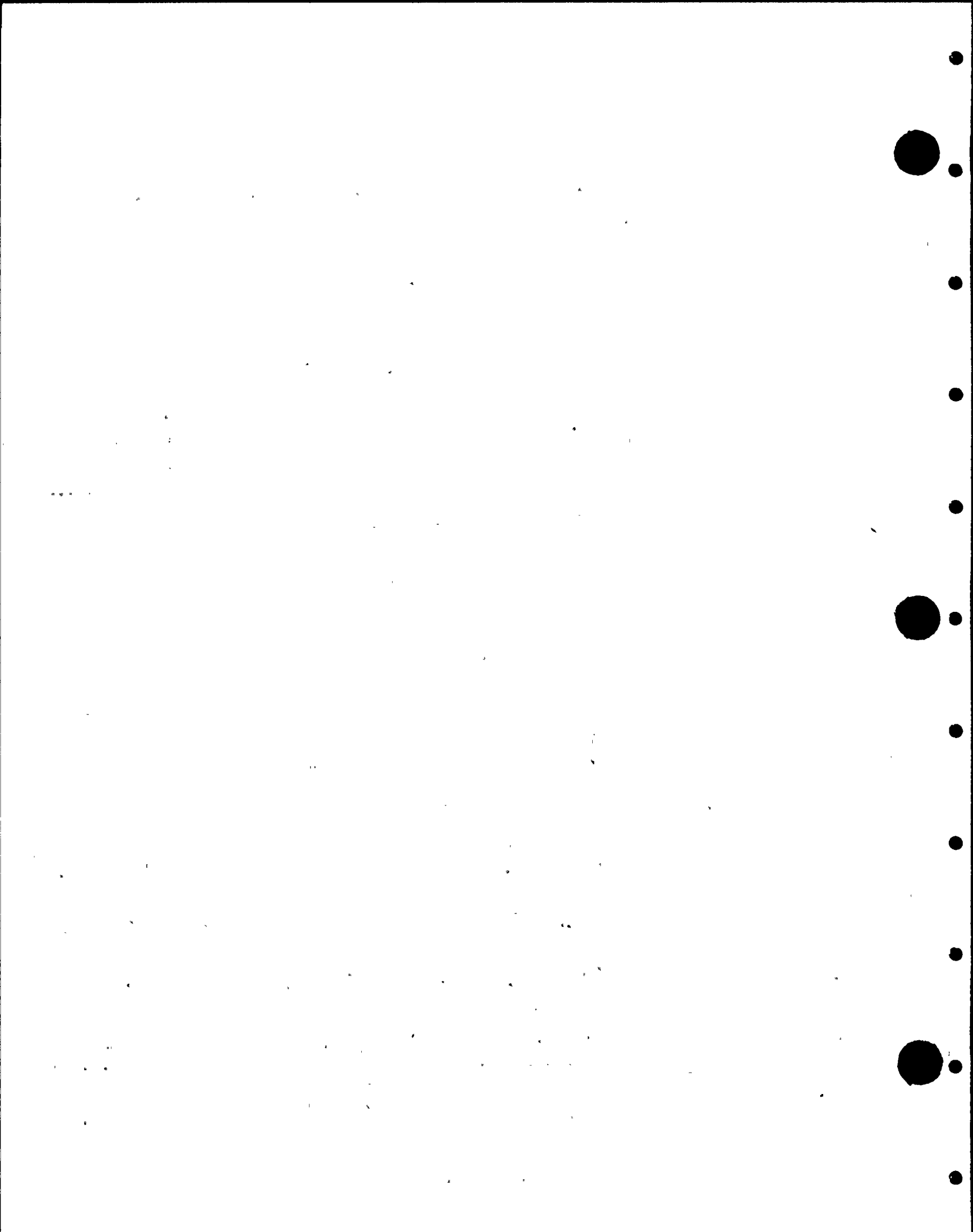


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(From Page 5.017 of NE-02-84-31, Ref. 9)

Hydraulic Friction Losses at Rated Flow(7,450 gal/min) for RHR Loop B in LPCI.Mode





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From Page 5.024 of Reference 9 we use $\Delta H = 115.5$ ft as best estimate value (see the line "Total, HX + Bypass"). This head loss is for rated flow = 7.45 Kgal/min. The friction coefficient in psid is then

$$FR COEF = 115.5 \text{ ft} \left(\frac{14.7 \text{ psid}}{33.93 \text{ ft}} \right)$$

which is initialized in Line 1140 on Page A3-10. The friction loss is then calculated as

$$DPFRIC = 115.5 \left(\frac{14.7}{33.93} \right) \left(\frac{Q}{7.45} \right)^2$$

in Line 3620 on Page A3-12. DPFRIC is in psid and Q is in Kgal/min. Line 3460 (Page A3-12) sums up the total pressure drop, and Line 3600 calculates the flow in Kgal/min.

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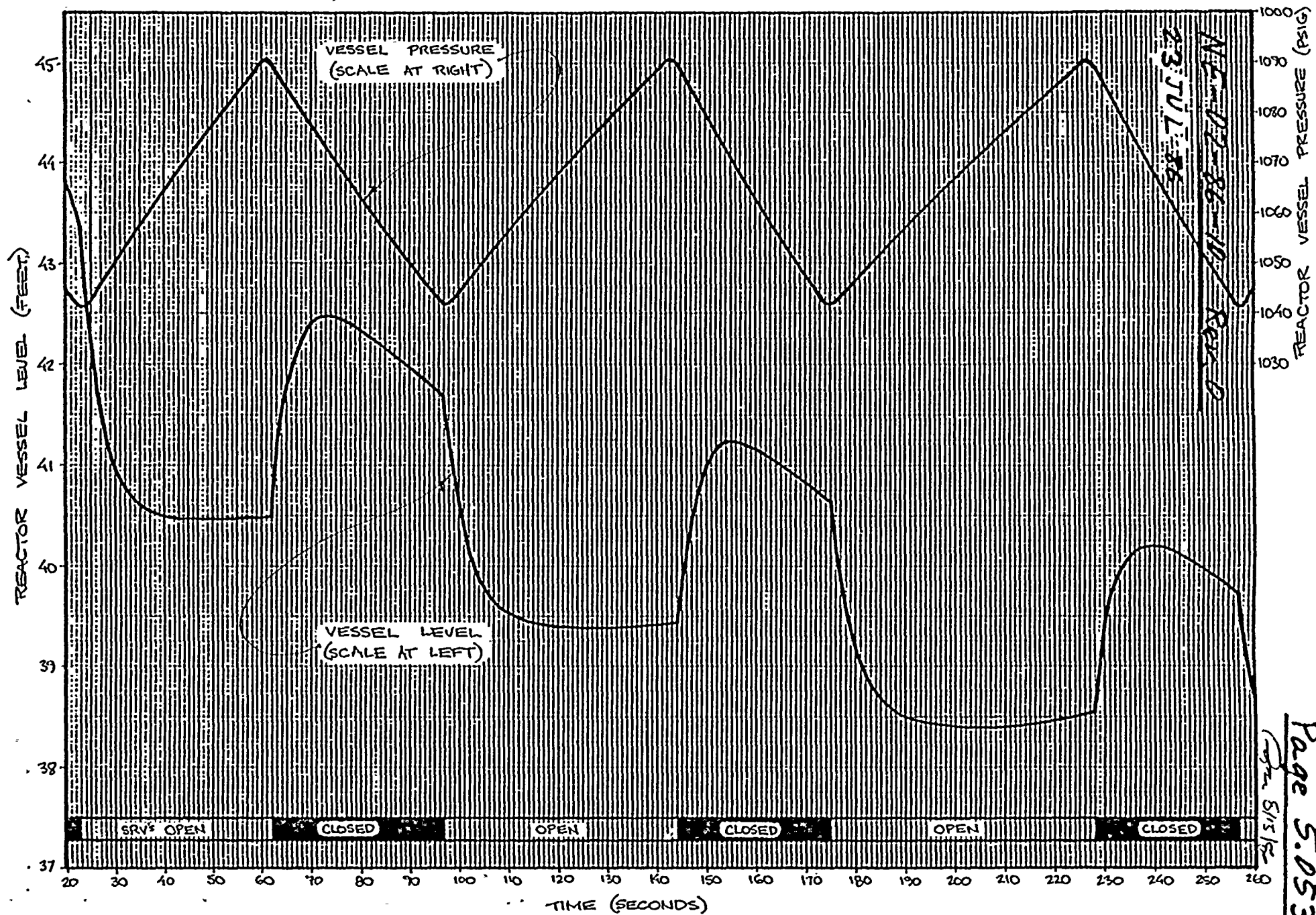
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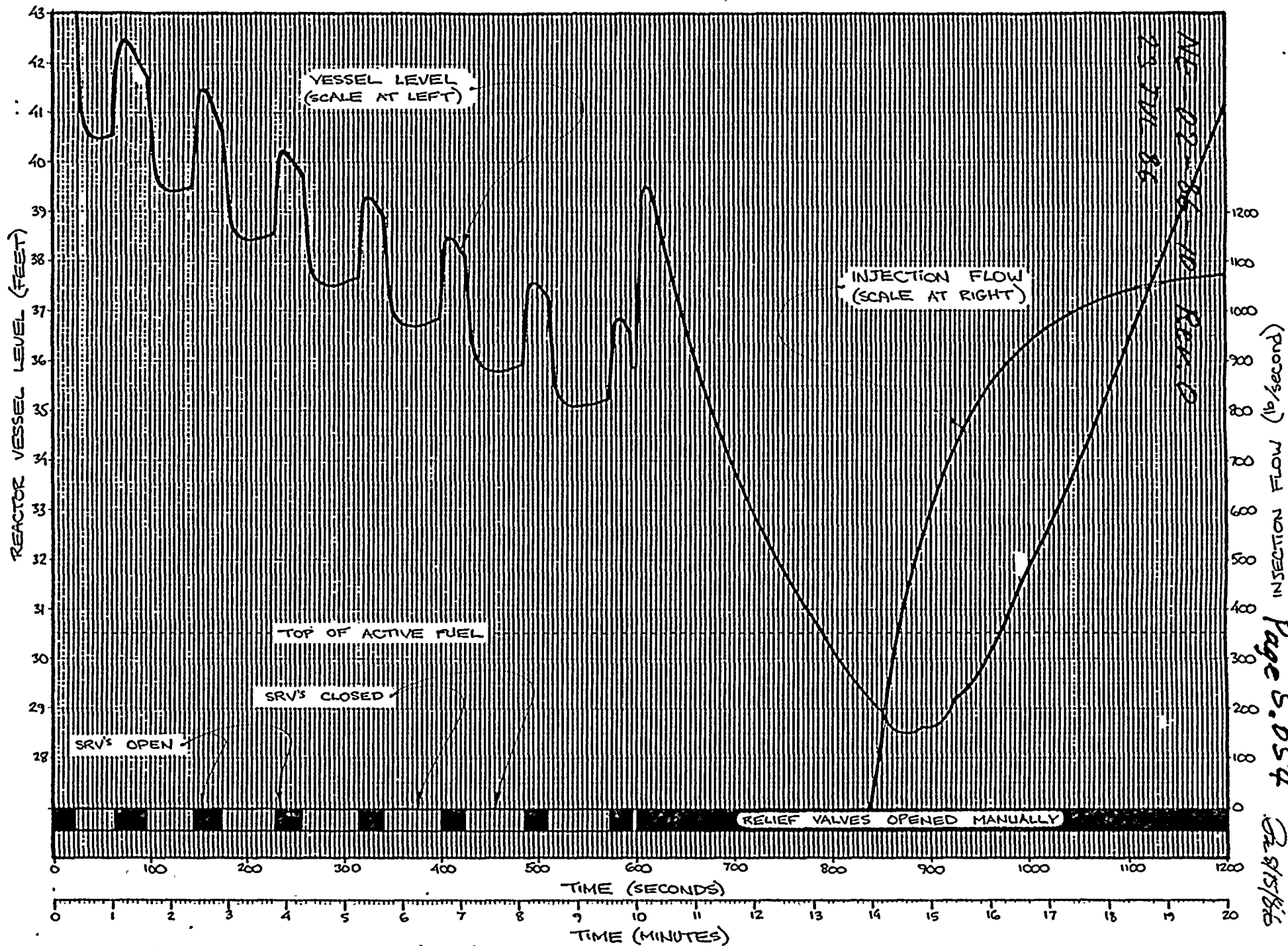
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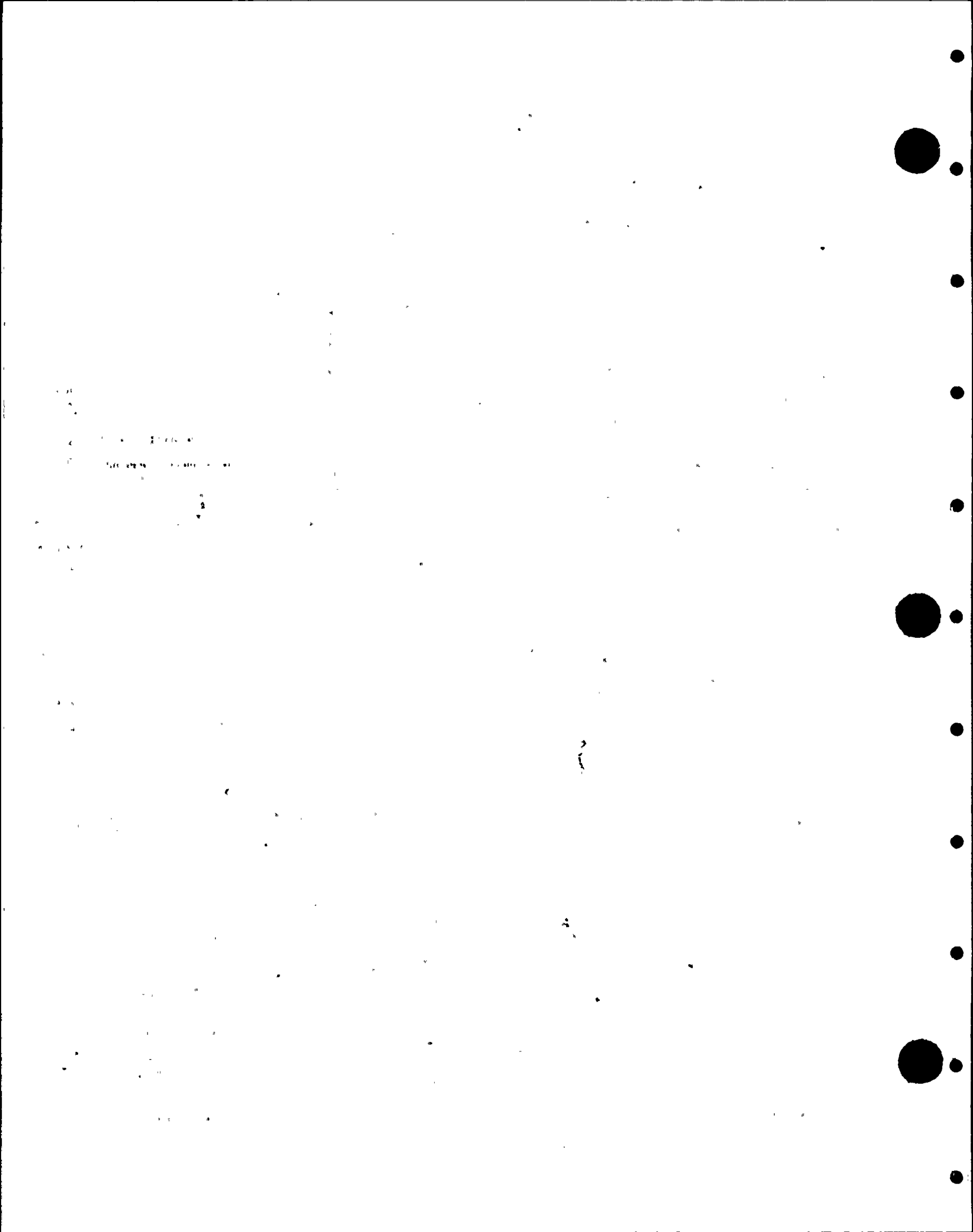
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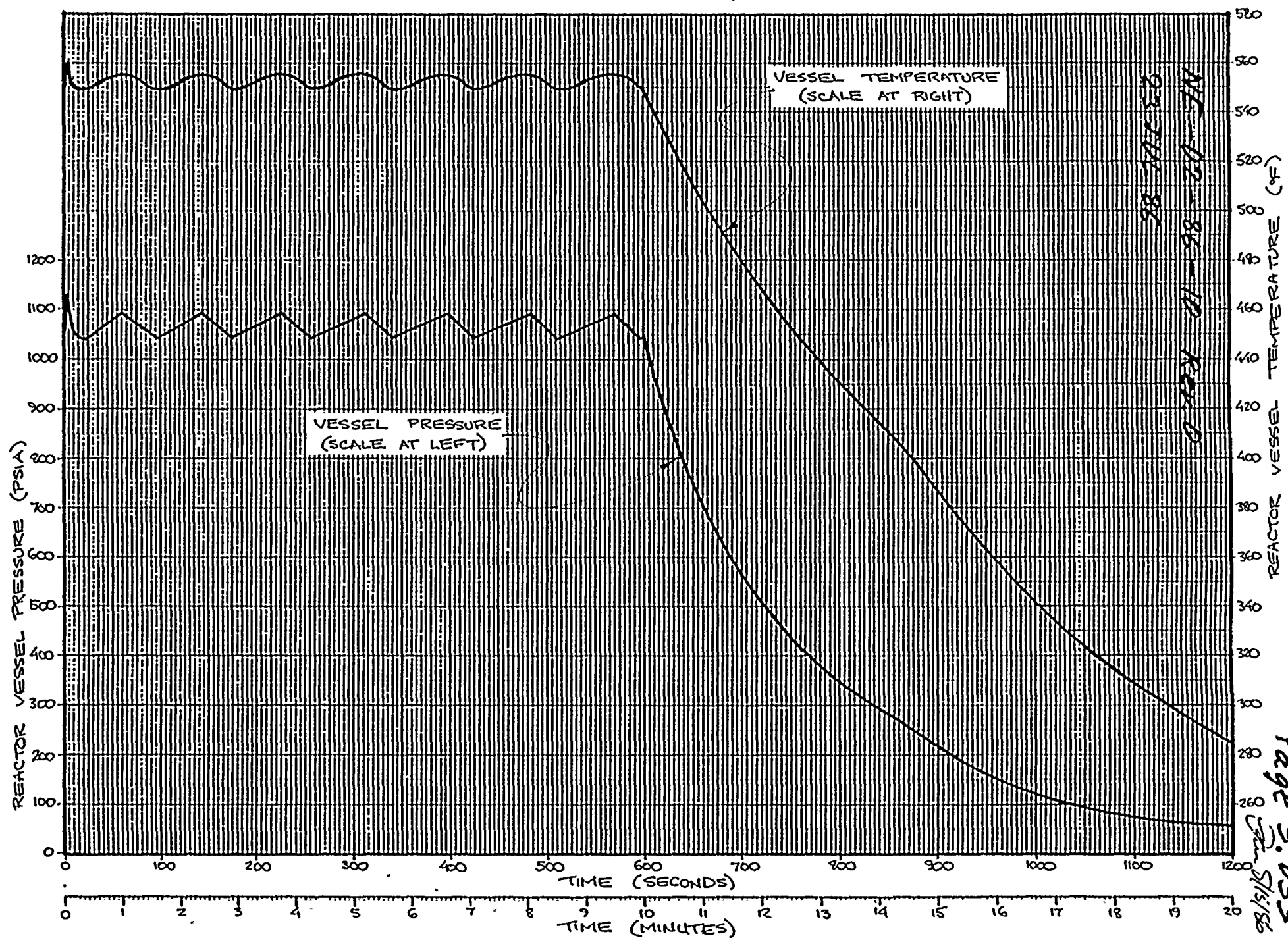
3. Discussion of Plant Transient Response

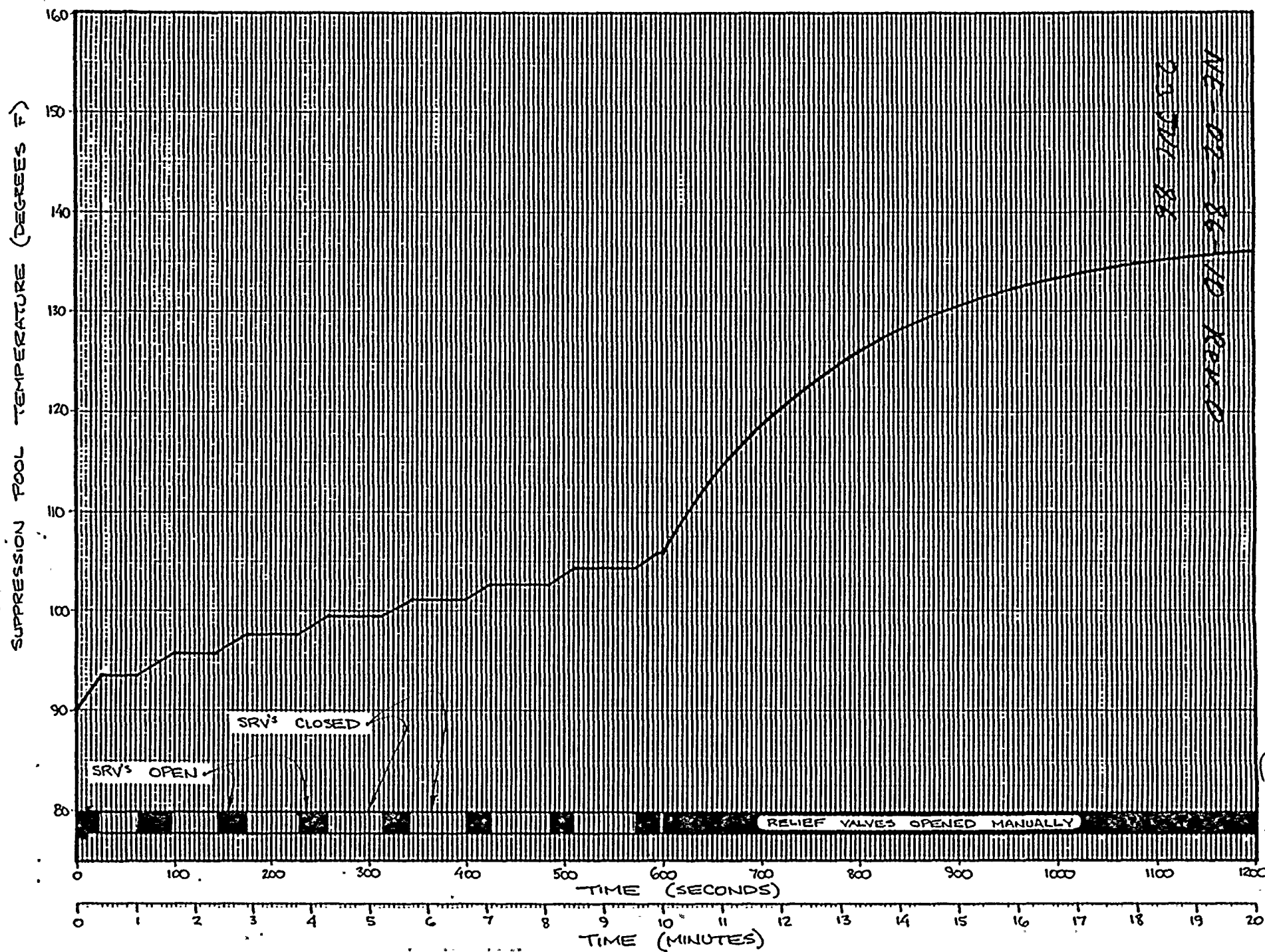
The next page shows a high resolution graph of vessel level and vessel pressure over the time interval of 20 sec thru 140 sec. The graph illustrates how the relief valve opening results in a remarkably linear pressure decrease, and how it just as linearly increases after the valve has closed. The level response is quite nonlinear. A very steep swell occurs right after the valve pops open. As vapor is lost thru the open valve, the level slowly decreases: the steam generation rate, even with flashing, does not keep up with the rate at which steam is lost. After the relief valve has closed, the level decreases just as steeply as it had previously increased.













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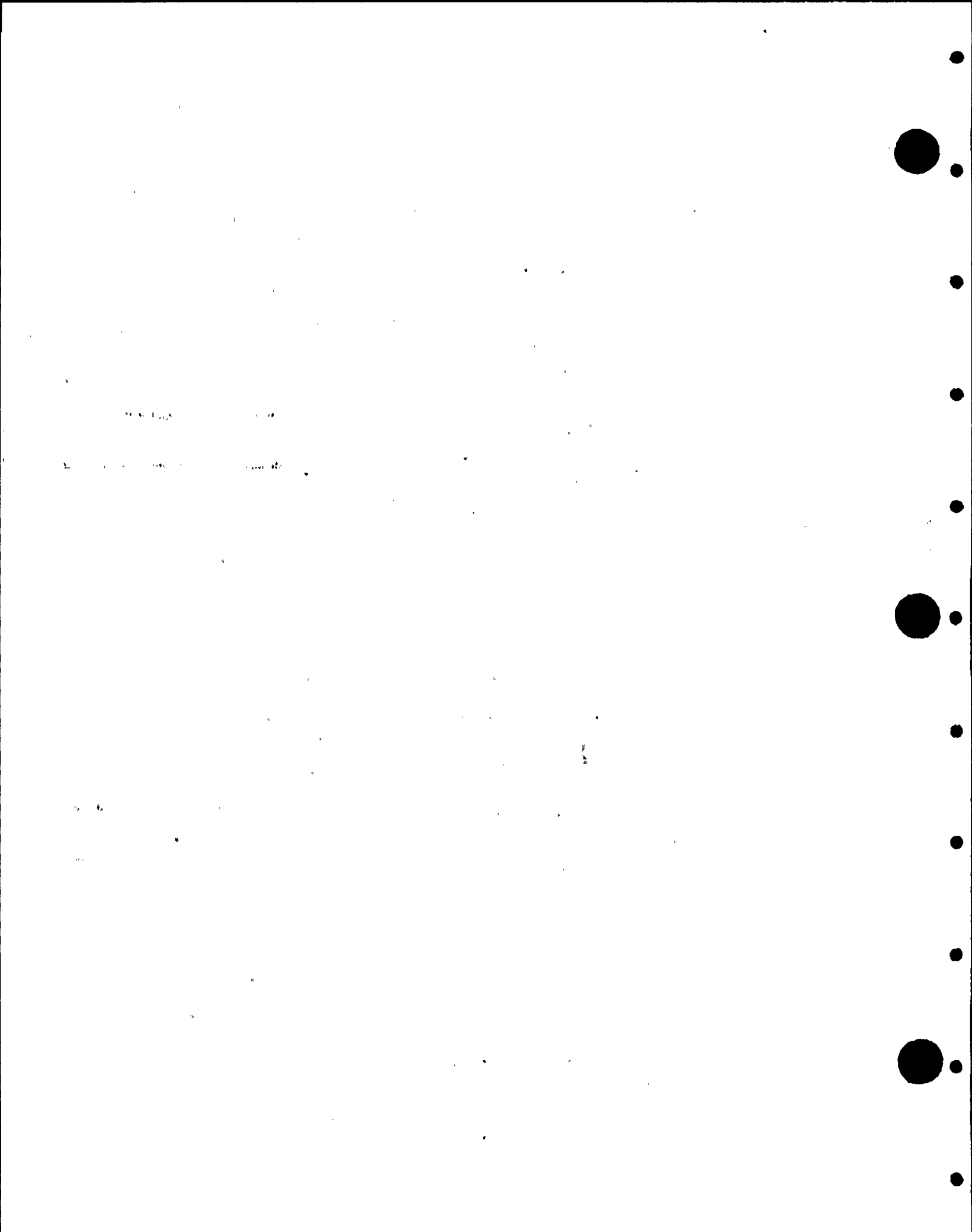
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Page 5.054 shows the reactor liquid level response together with the injection flow. At the bottom is the indicator bar showing the valve openings. During the first 10 minutes the level drifts down from an initial value of 45 ft (see data on Page A4-06) to about 35.0 ft at a slowly decreasing rate. A steep level swell of 1.6 to 1.8 ft occurs every time the valves open. A level swell of 4.8 ft occurs when manual depressurization with 6 valves is initiated. This level swell results from vigorous flashing thruout the fluid region when the pressure starts to decrease exponentially (see Page 5.055, lower trace). From thereon the level decreases steeply, until it reaches the top of the active fuel at $t=791$ sec. Meanwhile the injection flow





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has started at $t = 837$ sec. The liquid level reaches its minimum (about 2.0 ft below the top of the active fuel) at $t = 873$ sec. It goes back up and reaches the top of the active fuel again at $t = 968$ sec, for a total uncover time of $\Delta t = 177$ sec. The slight kink in the level trace at $t = 750$ sec is caused by a sudden change in the level versus volume function as the level drops below the setpoint L1 (at 31.54 ft). After the level turns back up (at $t = 873$ sec), it recovers very quickly as the pressure decreases further and injection flow increases rapidly.

Page 5.055 shows the pressure trace and the temperature trace for the reactor



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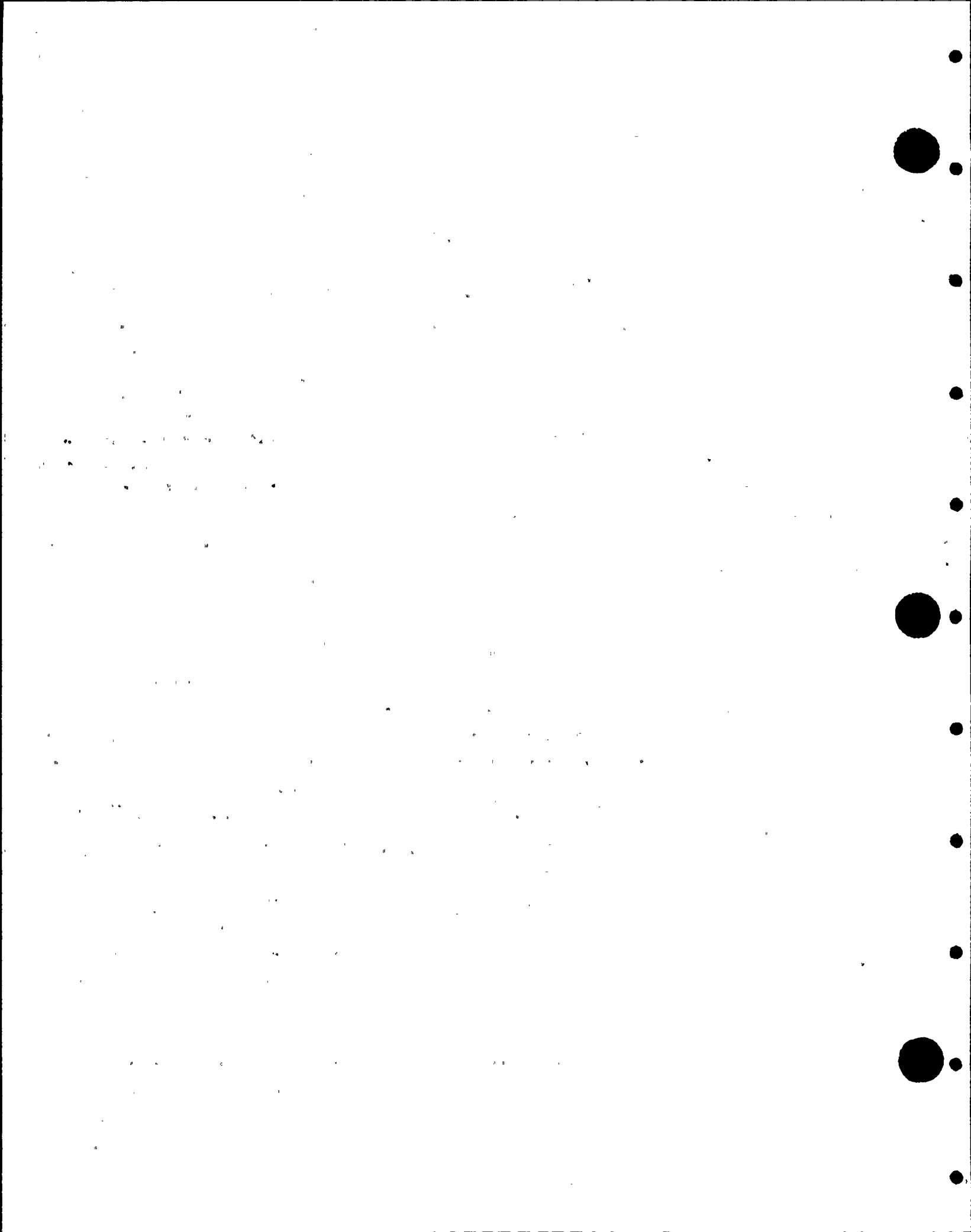
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coolant. The pressure trace shows a total of 8 pressure decreases during the first 10 min, caused by the automatic relief valve openings. The temperature trace is naturally synchronized to the pressure trace. Both traces are rather repetitive until manual depressurization is initiated at $t = 10$ min.

Page 5.056 shows the pool temperature versus time. The trace nicely illustrates how each relief valve actuation causes a temperature increase of slightly less than 2 Fahrenheit. The tail end of the trace shows a drastic decrease in gradient, that is, the temperature increase gets progressively smaller. The trace illustrates that this particular transient is quite easy on the suppression pool.





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4. Results and Conclusions

The key results can be summarized as follows:

Start of Manual Depressurization 10.0 min

Start of LPCI Flow 14.0 min

Time of Minimum Coolant Level 14.6 min

Duration of Partial Fuel Uncovery 3.0 min

Vessel Depressurization Time ~10 min

Recommended Time for Shifting
from LPCI Mode to RSC Mode ~20 min

Minimum Coolant Level 2.0 ft
below TAF

Initial Coolant Temperature 552 F

Upper Bound Estimate for Peak
Cladding Temperature at
Minimum Coolant Level 788 F

Maximum Suppression Pool
Temperature ~140 F



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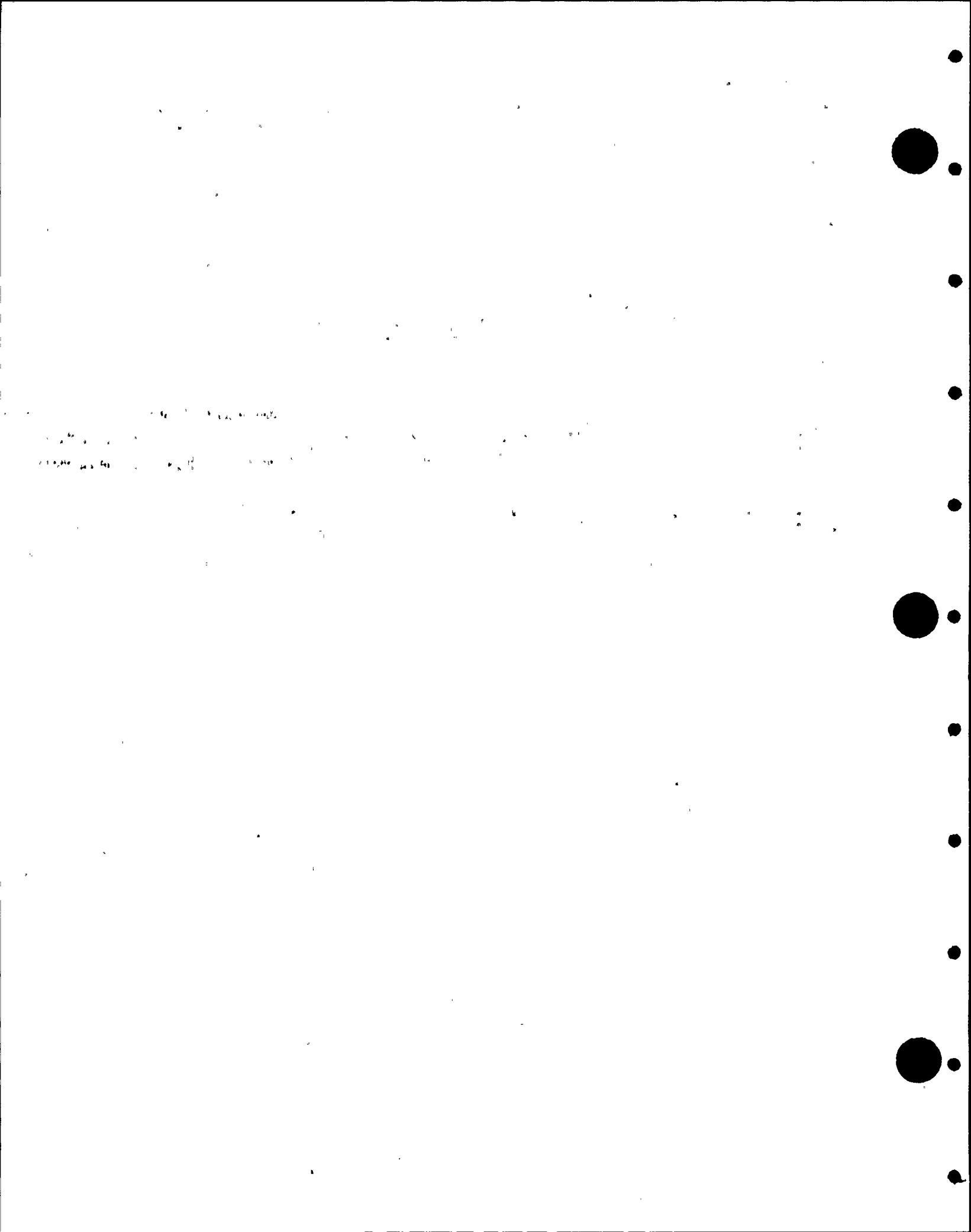
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Although this transient is based on an unusually harsh (nonphysical) scenario, it will not result in any fuel overheating, undercooling and/or potential for cladding damage.

Two recommendations can be derived:

1. Depressurize with all six valves as early as possible
2. After the reactor coolant level has recovered to its normal region, terminate the LPCI (low pressure coolant injection) mode and initiate the RSC (reactor shutdown cooling) mode.



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Page A1-01

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APPENDIX 1

Bounding Estimate for Heatup
Rate of Unsubmerged Fuel



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Appendix 1

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Traces from NEDO-24708A, Ref. 6

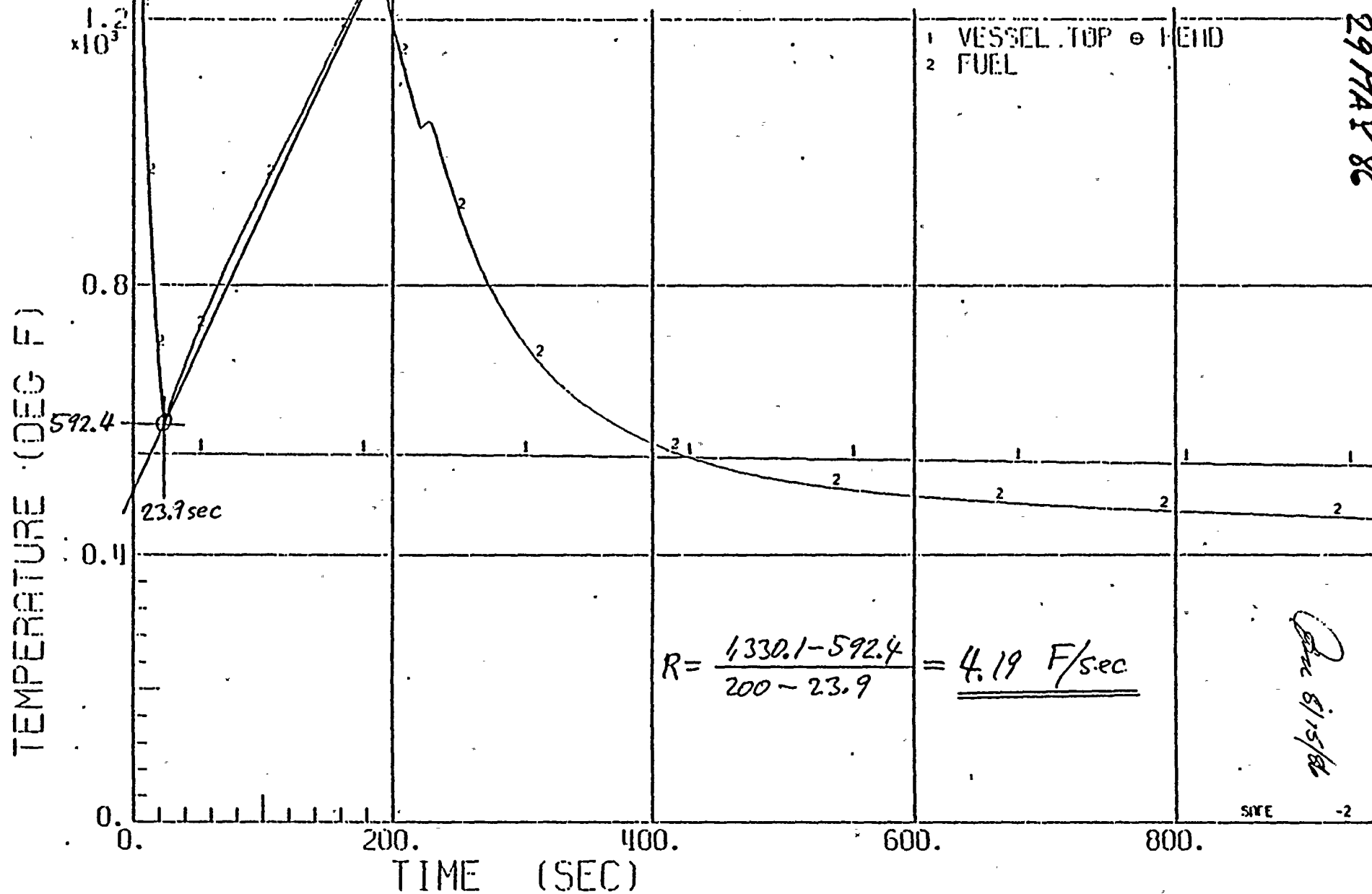
Ref. 6 Page	Temp. Gradient F/sec	Starting Time sec	Decay Power pct	Adjusted Temp. Gradient F/sec
4-1002	4.19	23.9	4.25	1.92
4-1085	1.64	1,394	1.72	1.85
4-1126	1.66	1,394	1.72	1.88
4-1175	1.64	1,383	1.722	1.85
4-1184	1.65	1,394	1.72	1.87
4-1191	1.63	1,392	1.72	1.84

The following 6 pages show fuel temperature traces from Reference 6. These traces are tabulated in the above table. The first column gives the appropriate page number in Reference 6. Column 2 shows the heatup rate as taken from the graph. In the third column, the time at which heatup starts is listed. The fourth column shows the AWS decay power at that time, in percent of full power. The fifth co-



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FIGURE 3.5.2.1 - 15.6 TEMPERATURE VS TIME FOR A DBA SUCTION BREAK WITH ONE LPCI AVAILABLE.

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Page A1-03

SHE -2

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

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1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

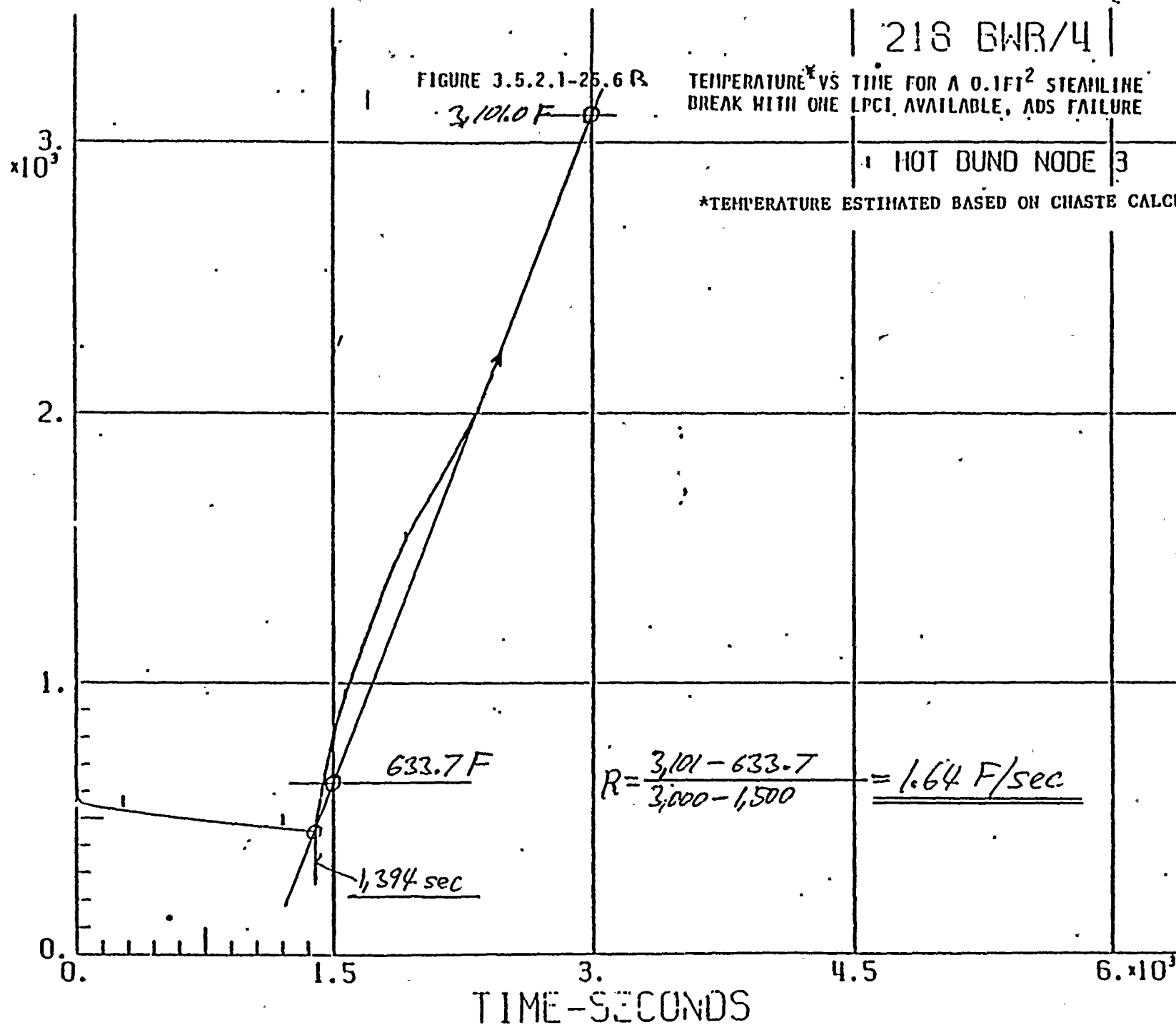
1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

1. 1000 200 100 1000 1000 1000

PEAK CLAD TEMP - DEG F



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Page A1-04

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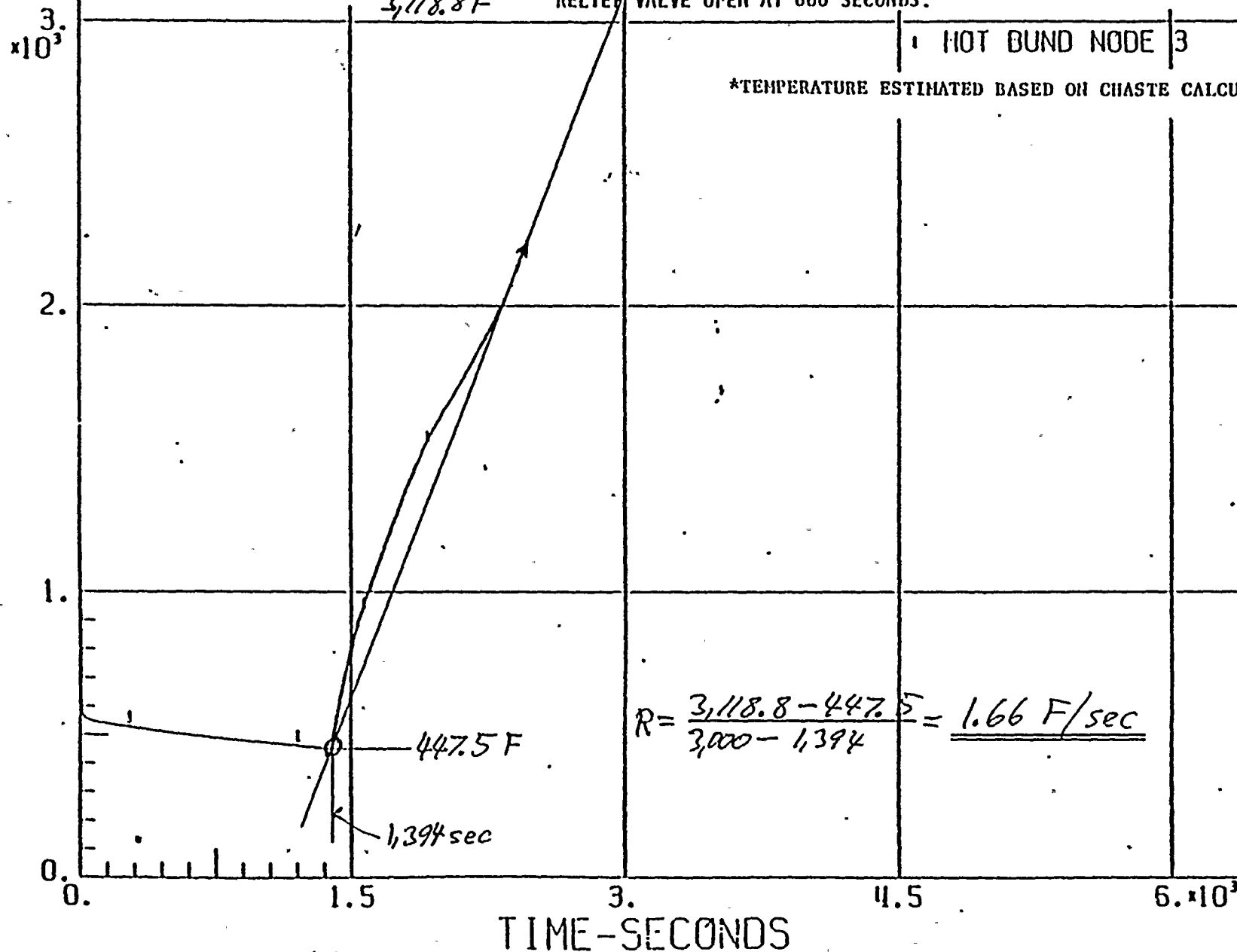
FIGURE 3.5.2.1-30-68 TEMPERATURE VS TIME FOR AN ISOLATION WITH ONE LPCI AVAILABLE, ONE RELIEF VALVE OPEN AT 600 SECONDS.

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HOT BUND NODE 3

*TEMPERATURE ESTIMATED BASED ON CHASTE CALCULATION

PEAK CLAD TEMP - DEG F
4-1126

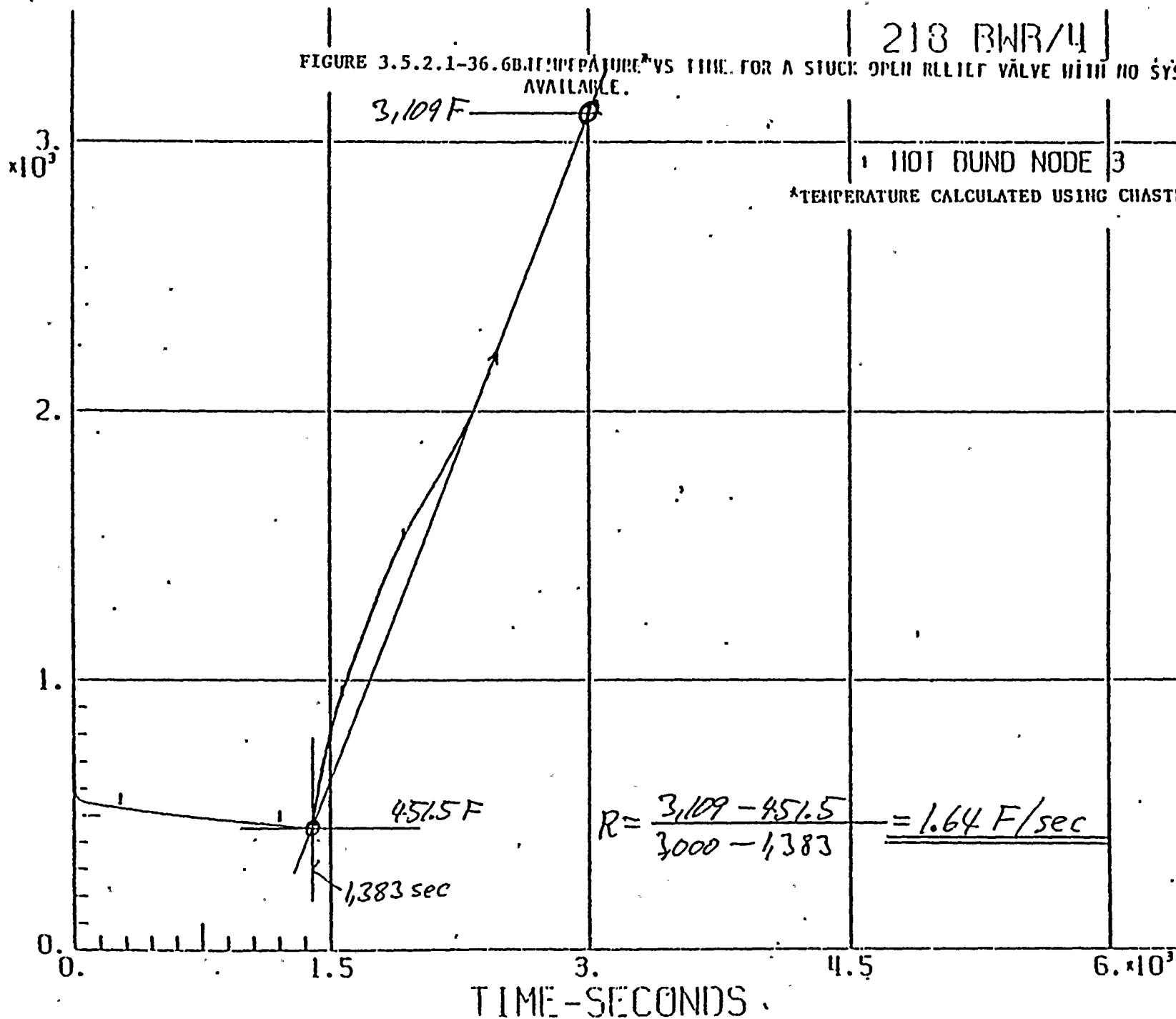


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Page A1-05

*, 100460026656748647'2930 -0.

PEAK CLAD TEMP - DEG F
4-1175



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Page A1-06

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4-1184

PEAK CLAD TEMP - DEG F

$\times 10^3$

3.

2.

1.

0.

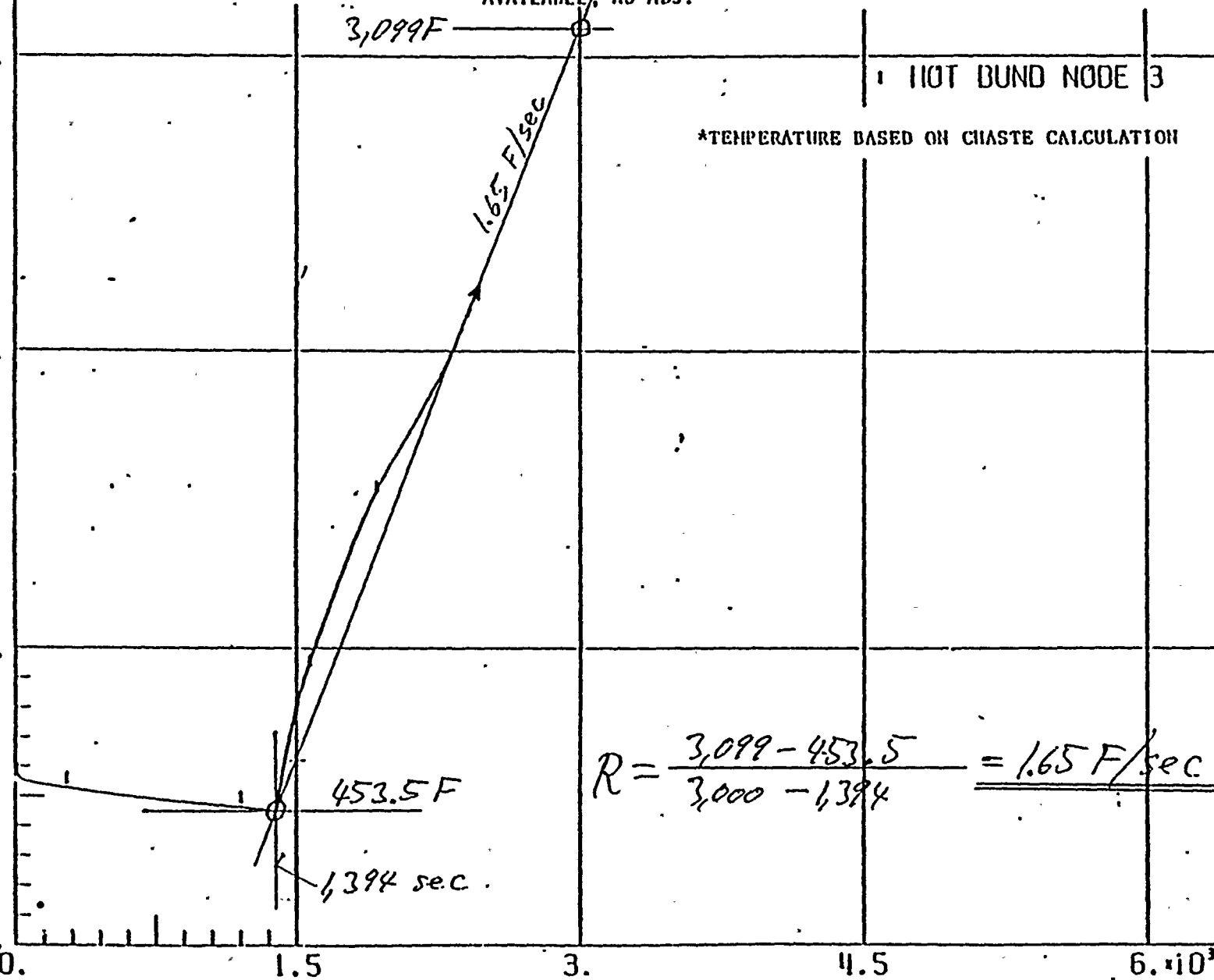


FIGURE 3.5.2.1-37.6B TEMPERATURE VS TIME FOR A STUCK OPEN RELIEF VALVE WITH ONE LPCI AVAILABLE, NO ADS.

BWR/4-218

110T BUND NODE 3

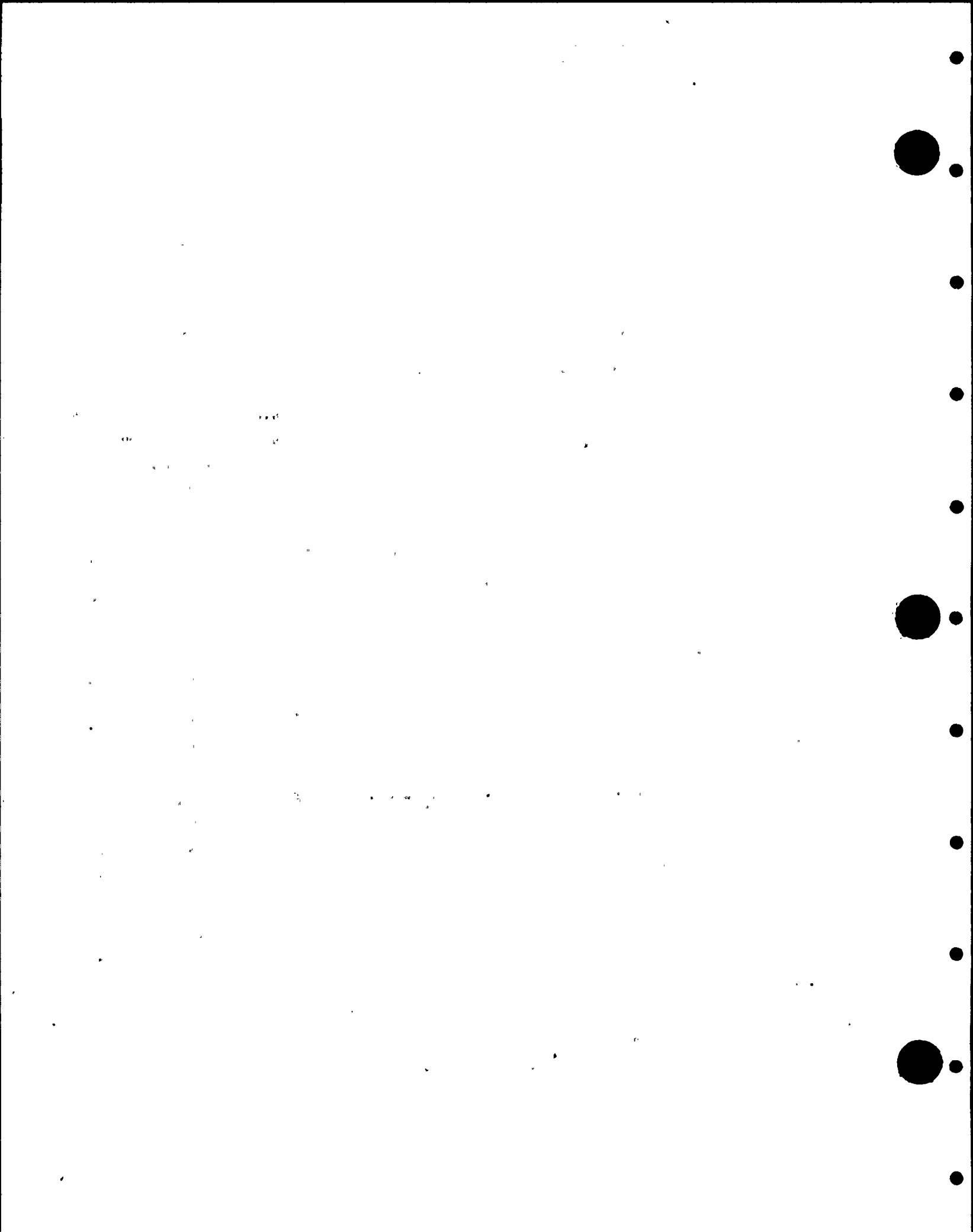
*TEMPERATURE BASED ON CHASTE CALCULATION

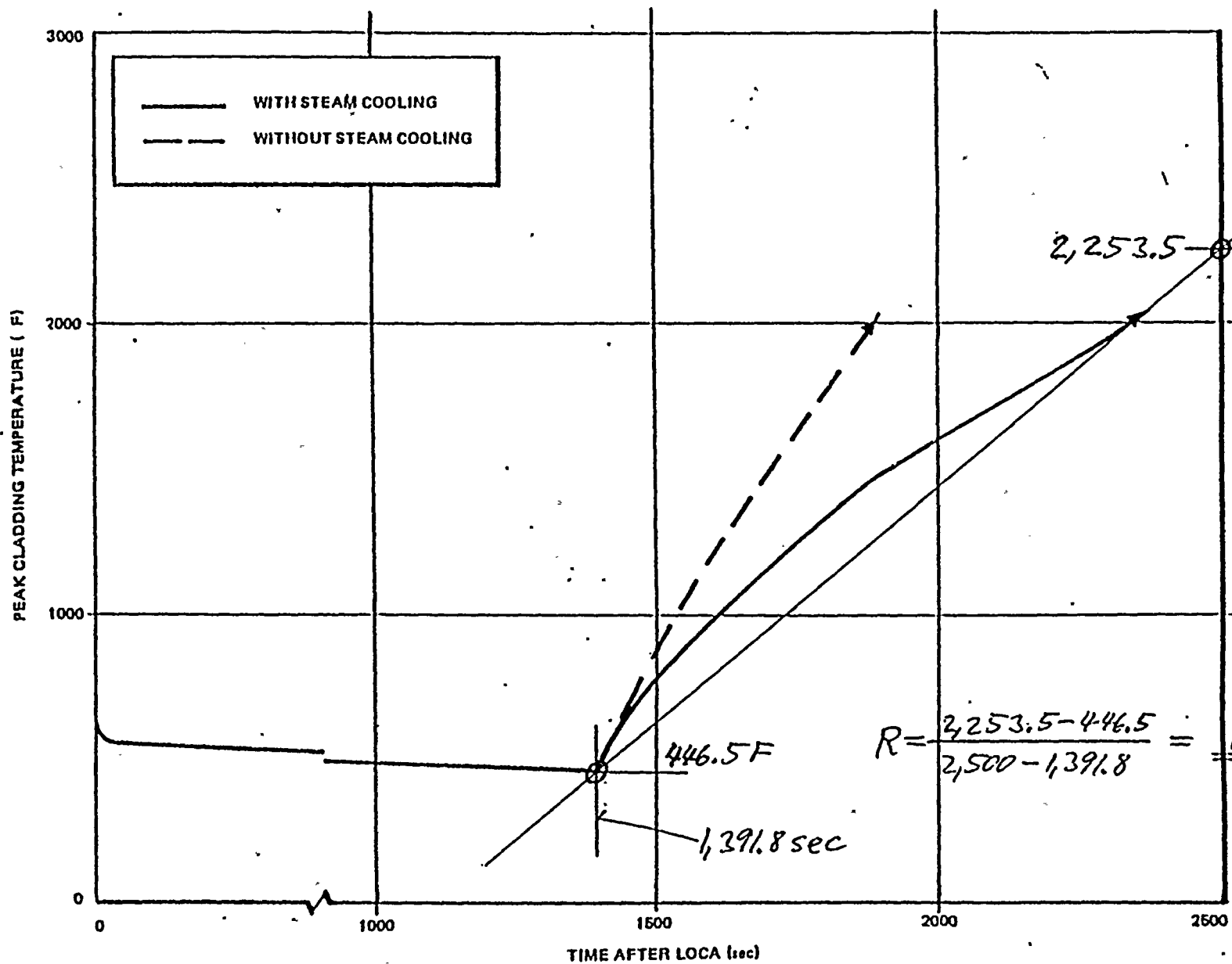
$$R = \frac{3,099 - 453.5}{3,000 - 1,394} = 1.65 \text{ F/sec}$$

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Figure 3.5.2.4-2.. The Effect of Steam Cooling on Peak Cladding Temperature



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lumin shows the adjusted heatup rate for the ANS decay power at $t = 815$ sec. The heatup rate is adjusted by the ratio in decay power:

$$\text{Rate}(t_2) = \text{Rate}(t_1) \left(\frac{\text{Power}(t_2)}{\text{Power}(t_1)} \right)$$

with t_1 = time when heatup starts in GE trace, see Column 3

$$t_2 = 815 \text{ sec}$$

$$\text{Power}(t_2) = \text{Power}(815 \text{ sec}) = 1.945 \text{ pct}$$

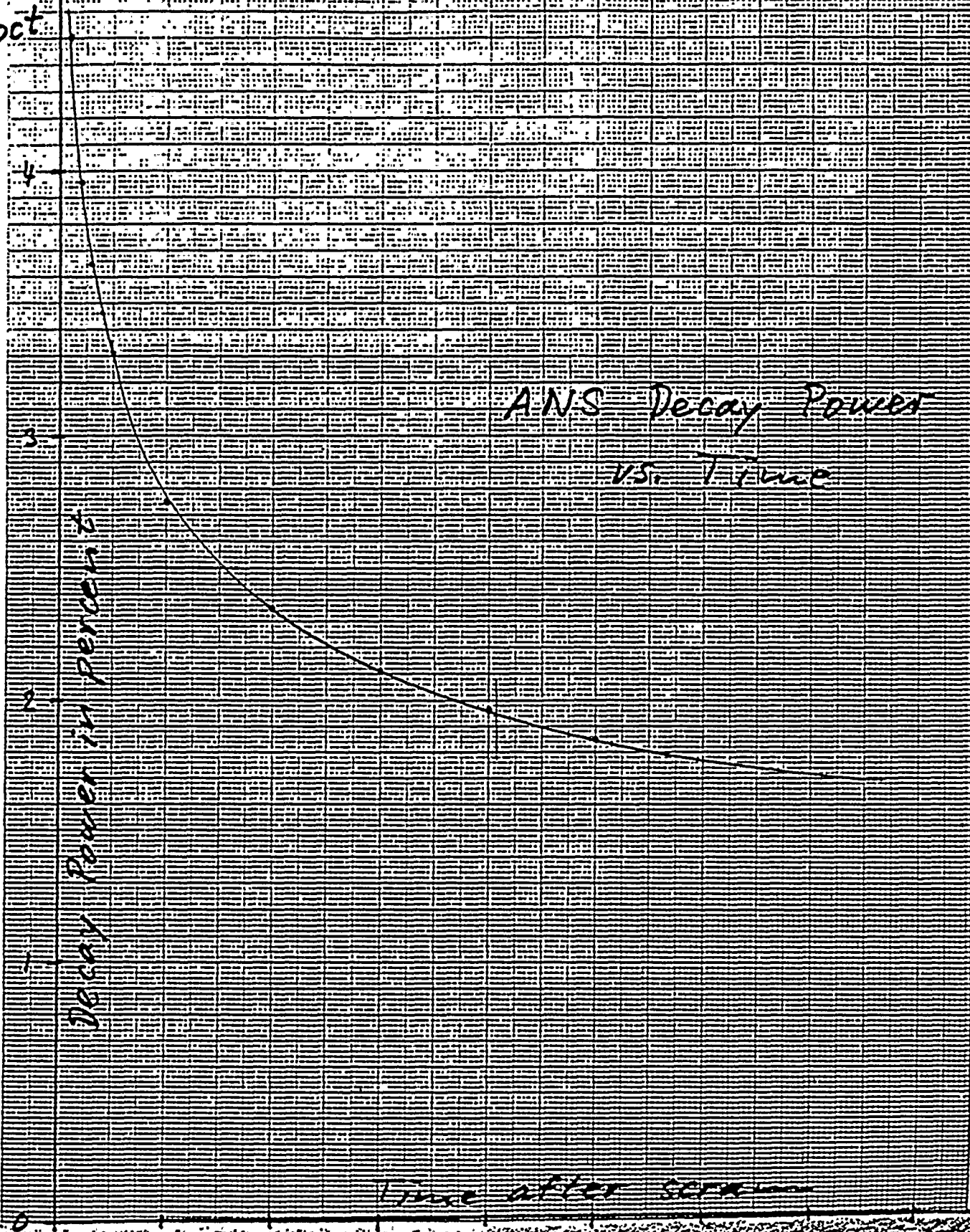
The decay power in percent is read from the graph of ANS decay power on Page A1-10. The underlying rationale for this approach is that the heatup rate is determined by

1. The level of decay power imparted on the fuel rod; this is approximately constant over small intervals of time, especially if the scram occurred more

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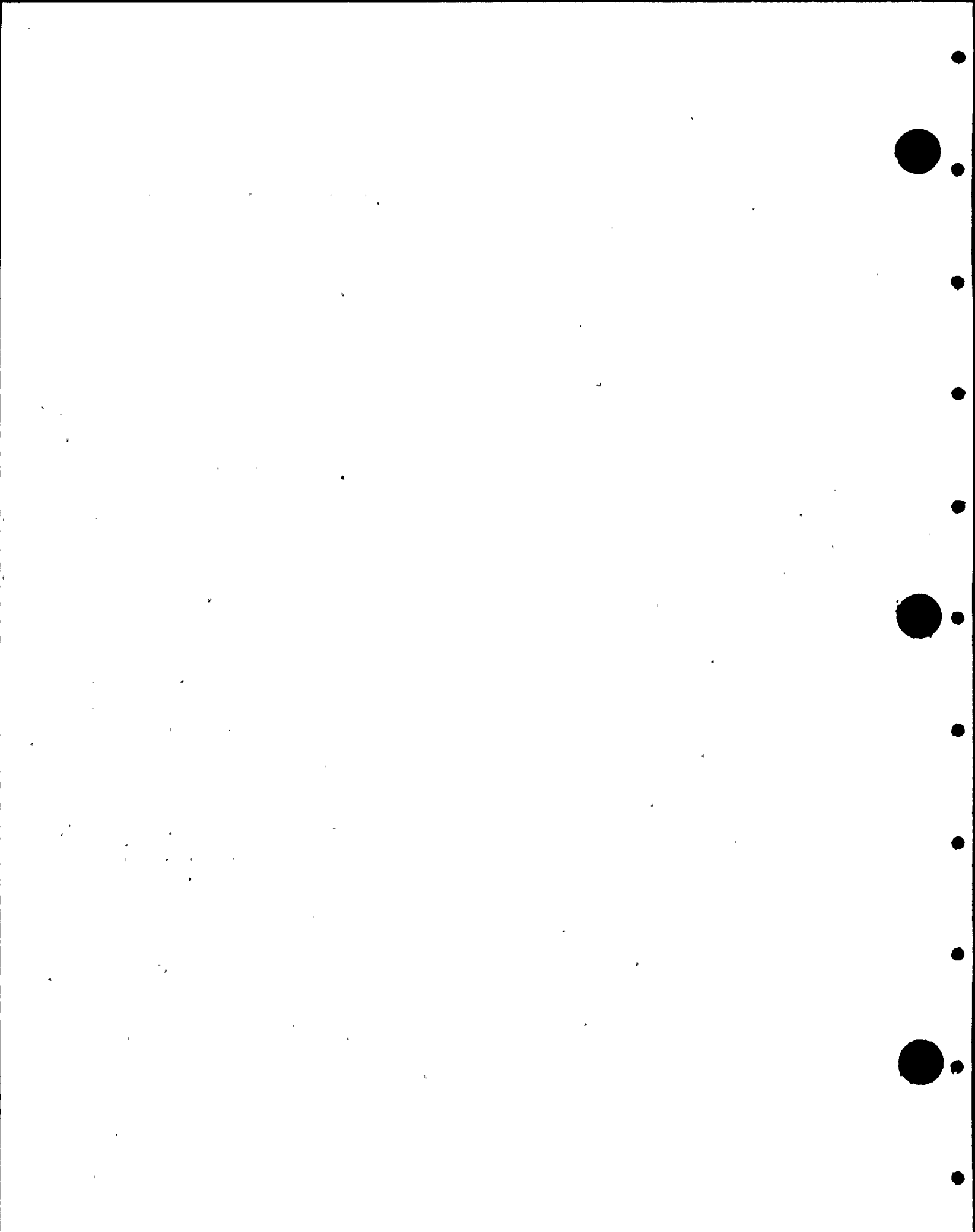
pct



A.N.S. Decay Power
vs. Time

Decay Power in Percent

Time after SCRAM





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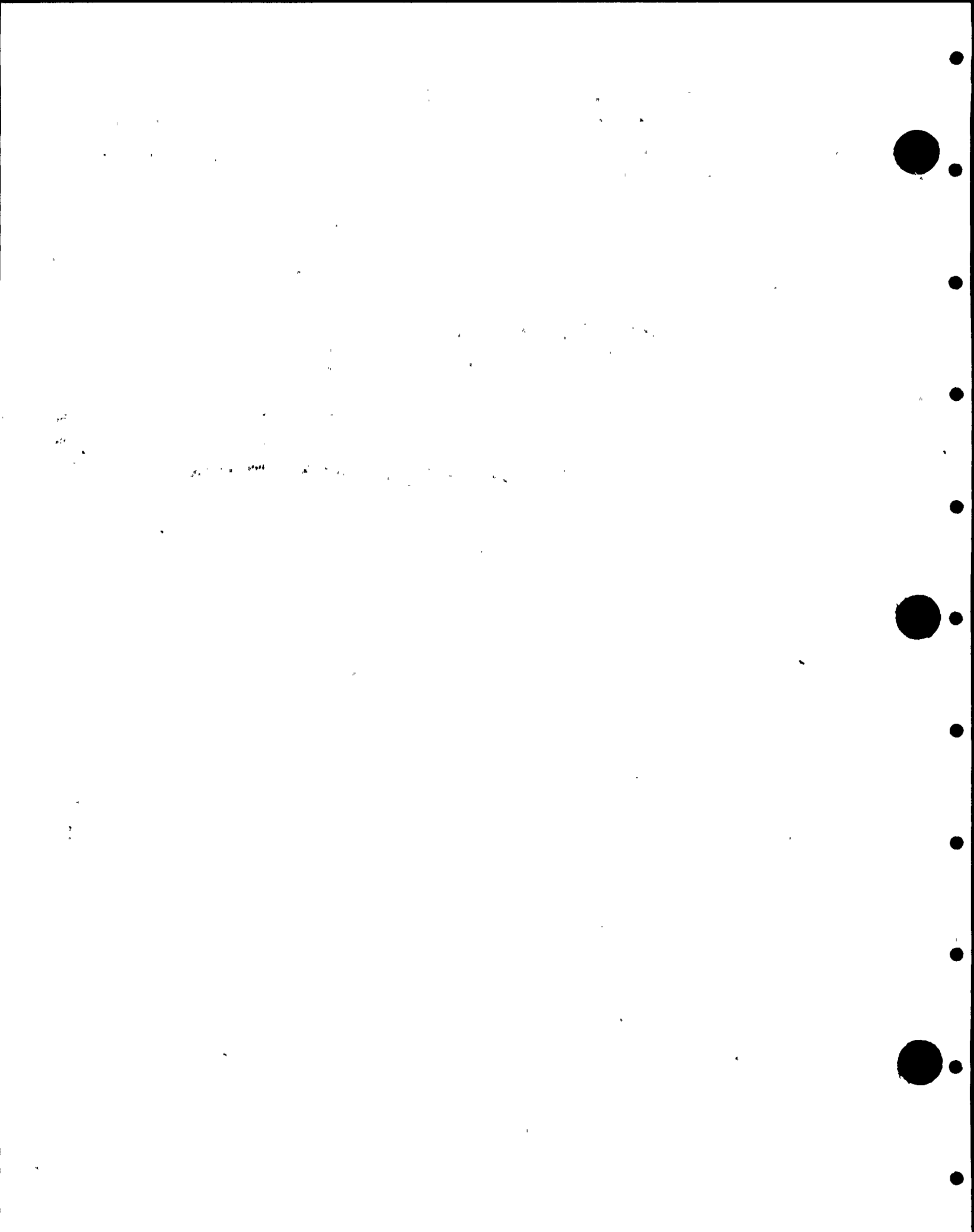
than 10 min ago.

2. The specific heat (thermal capacity) of the fuel rod, which is approximately constant for reasonably limited temperature intervals sufficiently removed from the melting point.

3. The fact that the steam cooling effect, which is modeled in the GE CHASTE code, is not a strong function of the heatup rate.

These three arguments combine to justify using the decay power ratio for adjusting the heatup rate.

For all six traces tabulated on Page A1-02, the adjusted heatup rate (see last column of the table) is below 2.0 Fahrenheit/second. In all of these 6 cases steam cooling is accounted for. Based on these results it can be stated that a heatup rate of 2.0 F/sec is an upper bound estimate.



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APPENDIX 2

Bounding Estimate for Stored
Structural Heat



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Reactor Depressurization Using 6 Relief Valves

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In both previous depressurization calculations (References 7 and 8), the thermal power versus time was taken from FSAR Table 6.2-11 (Page 6.2-106) together with FSAR Figure 6.2-27, see Pages A4-02 and A4-03 of NE-02-84-19 (Reference 7). In this approach the stored structural heat is added in terms of additional thermal power during the first 30 seconds after the scram. This method of accounting for the stored heat is conservative insofar as the additional equivalent thermal power causes a temperature increase which stored heat cannot cause. Also, in terms of thermal energy accounting it is not relevant when the additional thermal power is added. However, as far as correct pressure calculation during depressurization is concerned, adding this additional heat before start of depressurization is non-conservative with respect to low pressure coolant injection. In physical reality,

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the stored heat will somewhat retard the cooldown during depressurization, and consequently the reactor pressure will decrease more slowly, thereby delaying the start of low pressure coolant injection.

For this calculation, we determine the stored heat from the above data by integrating that extra hump on the decay power curve. From the amount of heat calculated, we determine a corresponding amount of steel which is then carried as a point node for steel, see Mass 4 on Page 5.007.

From Page A4-08 of NE-02-84-19 (Ref. 7) we get the integrated thermal power including stored heat for the first 30 sec:

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$$\begin{aligned} L_{30} &= 0.009,1 + 0.846,044 + 1.002,317 \\ &\quad + 1.442,906 + 0.843,602 + 0.706,255 \\ &\quad + 1.429,871 + 1.172,698 \end{aligned}$$

$$\underline{L_{30} = 7.44 \text{ unit}\cdot\text{sec}}$$

To calculate the corresponding full power seconds for the case without stored heat adjustment, we integrate the thermal power during the scram plus the decay power from $t = 3.9 \text{ sec}$ to $t = 30 \text{ sec}$. The tabulation on the next page shows the fission power data (during the scram) from Page 5.011. These data are linearly interpolated, and the power integral amounts to $3.1761 \text{ unit}\cdot\text{sec}$, as the table shows. From $t = 3.9 \text{ sec}$ to $t = 30 \text{ sec}$, we use the logarithmic decay

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<u>time</u> sec	<u>Power</u> frac	<u>Energy</u> unit·sec
0	1.000	—
1	0.987	0.9935
2	0.953	0.9700
2.5	0.903	0.4640
3.0	0.719	0.4055
3.5	0.330	0.2623
3.9	0.074	0.0808
Total		3.1761

power curve from Appendix 4 of Reference 7 (see also log-log graph on Page 5.012, use straight line in that graph). The straight line in the log-log graph is

$$\underline{\pi = A + B\tau} \quad \text{with}$$

$$\pi = \log(\text{Power}), \quad \tau = \log(\text{time})$$

We use 2 data points:

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time	Power	τ	π
3.9	0.074	0.591,065	-1.130,768
30	0.047	1.477,121	-1.327,902

from which we get the equations:

$$\left. \begin{aligned} -1.130,768 &= A + (0.591,065) B \\ -1.327,902 &= A + (1.477,121) B \end{aligned} \right\}$$

Solving for A and B, we have

$A = -0.999,265$

$B = -0.222,485$

and $\pi = -0.999,265 - (0.222,485) \tau$

Using the method from Appendix 4 of Reference 7, we get for the integrated power from $t=3.9$ sec to $t=30$ sec

$$L = \left(\frac{10^A}{B+1} \right) \left\{ 30^{B+1} - 3.9^{B+1} \right\}$$

or with A and B inserted:

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$$L = \left(\frac{10^{-0.999,265}}{0.777,515} \right) \left\{ 30^{0.777,515} - 3.9^{0.777,515} \right\}$$

$$L = 1.4423 \text{ unit} \cdot \text{sec}$$

The integrated thermal power during the first 30 sec without stored heat adjustment is then:

$$L'_{30} = 3.1761 + 1.4423 = 4.6184 \text{ unit} \cdot \text{sec}$$

The adjustment for stored heat then amounts to

$$\Delta L = 7.44 - 4.6184 = 2.82 \text{ unit} \cdot \text{sec}$$

With a full power value of 3,489 MW (see Page A4-02), 2.82 full power seconds amount to:

$$\Delta L = (2.82 \text{ unit} \cdot \text{sec}) \left(3,489 \frac{\text{MW}}{\text{unit}} \right) \left(\frac{1 \text{ hr}}{3,600 \text{ sec}} \right)$$

$$\Delta L = 2.733,05 \text{ MW hr}$$

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$$\Delta L = 2.733,05 (10^3) \text{ KWhr} \left(\frac{3,412 \text{ btu}}{\text{KWhr}} \right)$$

$$\underline{\Delta L = 9.3252 \text{ Mbtu}}$$

From Marks (Reference 10) Page 4-12 we have for the specific heat of steel:

$$\underline{C = 0.127 \frac{\text{btu}}{\text{lb} \cdot \text{F}}}$$

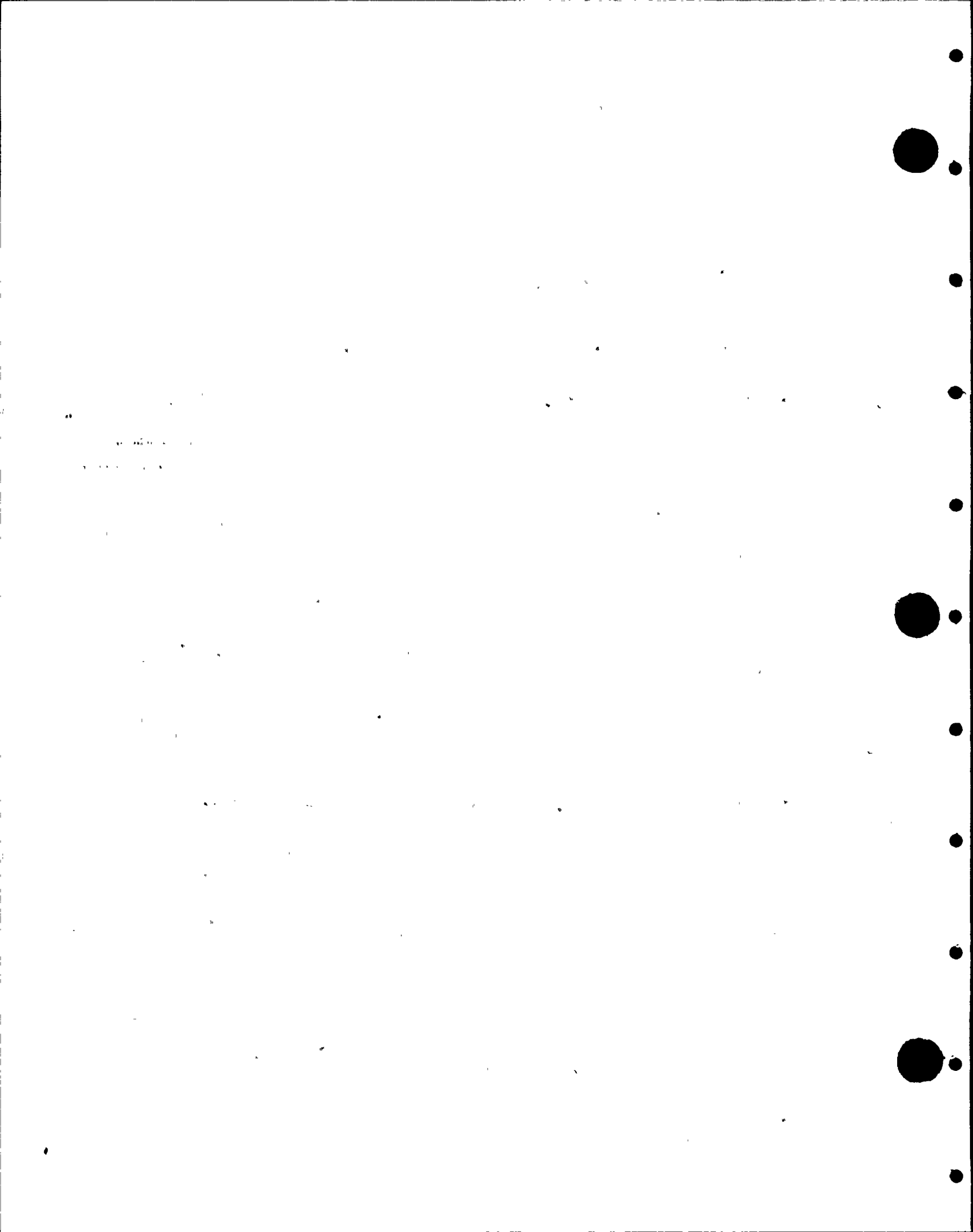
To assure that the estimate for the participating steel mass is not too small, we assume that no stored heat is transacted below 250 F. This results in a temperature range of

$$\Delta T = (550 - 250) \text{ F} = 300 \text{ F}$$

The thermally effective steel structure then amounts to

$$M = \frac{9.3252 (10^6) \text{ btu} \cdot \text{lb} \cdot \text{F}}{0.127 \text{ btu} \quad 300 \text{ F}} = 244.756 \text{ Klb}$$

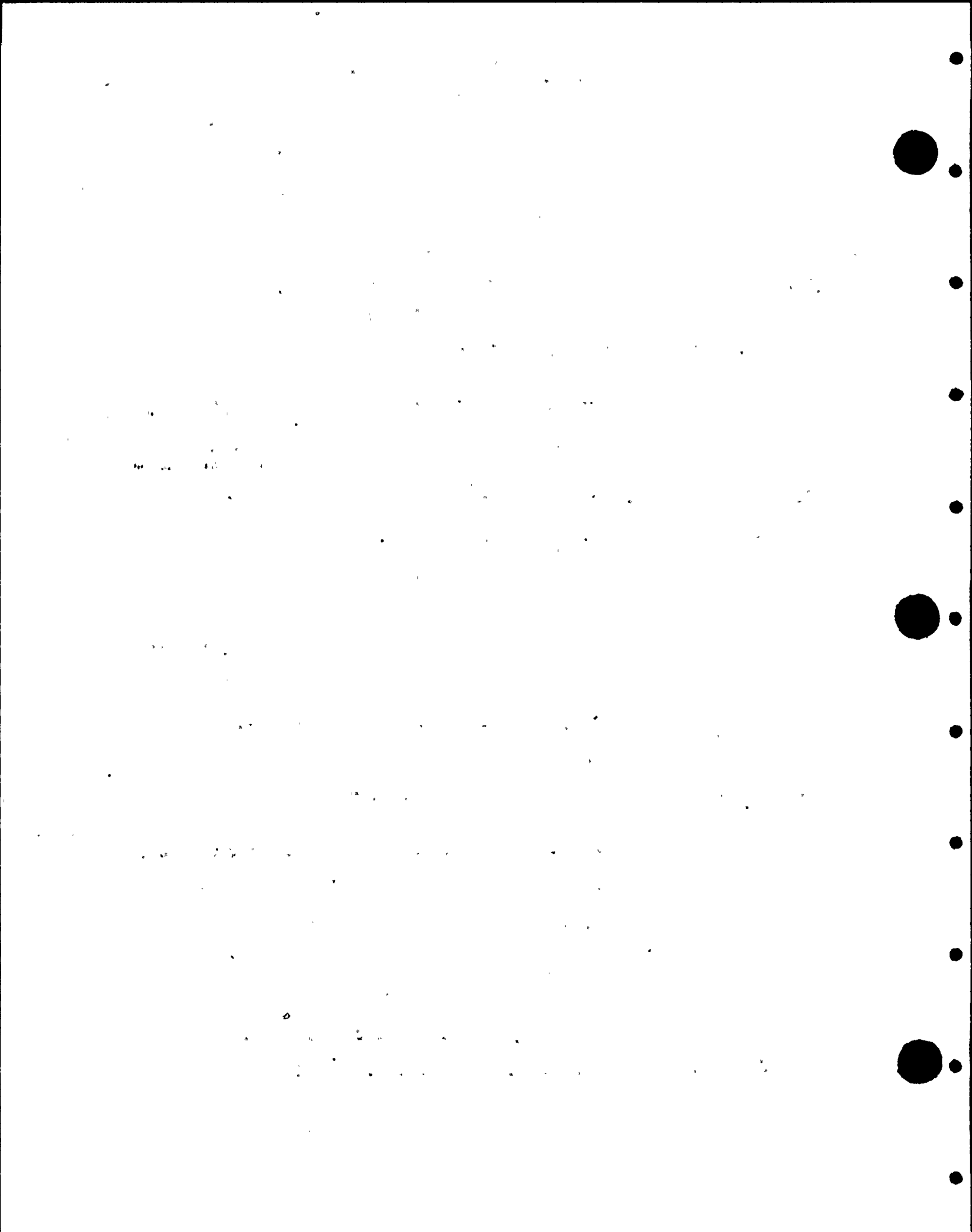
$$\underline{\underline{M \approx 245 \text{ Klb}}}$$



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This value (Item 1 on Page A2-11) was used in the initial computer runs. However, when discussing the data with GE it was found that the figure is too small. The FSAR gives further information on Page 6.2-95 (see next Page). Using the values for D.2.a, D.2.b and D.2.d, we have $Q = (106.1 + 58.6 + 25.2) \text{ Mbtu} = 189.9 \text{ Mbtu}$. This results in the data of Item 2, Page A2-11. The GE plant transient model contains about 2.13 Mlb, see Item 3. The discussion with GE resulted in the figures shown in Item 4,



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TABLE 6.2-3

ACCIDENT ASSUMPTIONS AND INITIAL
CONDITIONS FOR RECIRCULATION LINE BREAK

A.	Effective Accident Break Area (Total), ft ²	3.106	
B.	Components of Effective Break Area:		
1.	Recirculation Line Suction Nozzle Area, ft ²	2.508	
2.	RWCU crosstie line, ft ²	.078	
3.	Jet Pump Nozzles, ft ²	.520	
C.	Break Area/Vent Area Ratio	.0105	
D.	Primary System Energy Distribution ⁽¹⁾		
1.	Steam and Liquid Energy, 10 ⁶ Btu	414	
2.	Sensible Energy, 10 ⁶ Btu		
a.	Reactor Vessel	106.1	←
b.	Reactor Internals (less core)	58.6	←
c.	Primary System Piping	34.6	
d.	Fuel ⁽²⁾	25.2	←
E.	Assumptions Used in Pressure Transient Analysis		
1.	Feedwater Valve Closure Time	Instantaneous	
2.	MSIV Closure Time (sec)	3.5	
3.	Scram Time (sec)	<1	
4.	Liquid Carryover, %	100	
5.	Turbine Stop Valve Closure (sec)	0.2	
1.	All energy values except fuel are based on a 32°F datum.		
2.	Fuel energy is based on a datum of 285°F datum.		

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Thermal Capacity Data

Item	Reference	Thermal Capacity btu/F	Steel Mass (1) Mlb
1	LDCA Data FSAR Page 6.2-106	31,115	0.245
2	FSAR Page 6.2-95	474,750	3.74
3	GE SAFE Model, max. available	270,000	2.13
4	Discussion with GE, Best Estimate	209,550	1.65

(1) Calculated with $c = 0.127 \frac{\text{btu}}{\text{lb} \cdot \text{F}}$ for
steel and $\Delta T = 550 - 150 = 400 \text{ F}$

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which is the best estimate for actively participating thermal capacity for the type of transient analyzed here.

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APPENDIX 3

Listing of Computer Code

BLOWDOWN.01

in BASIC Language

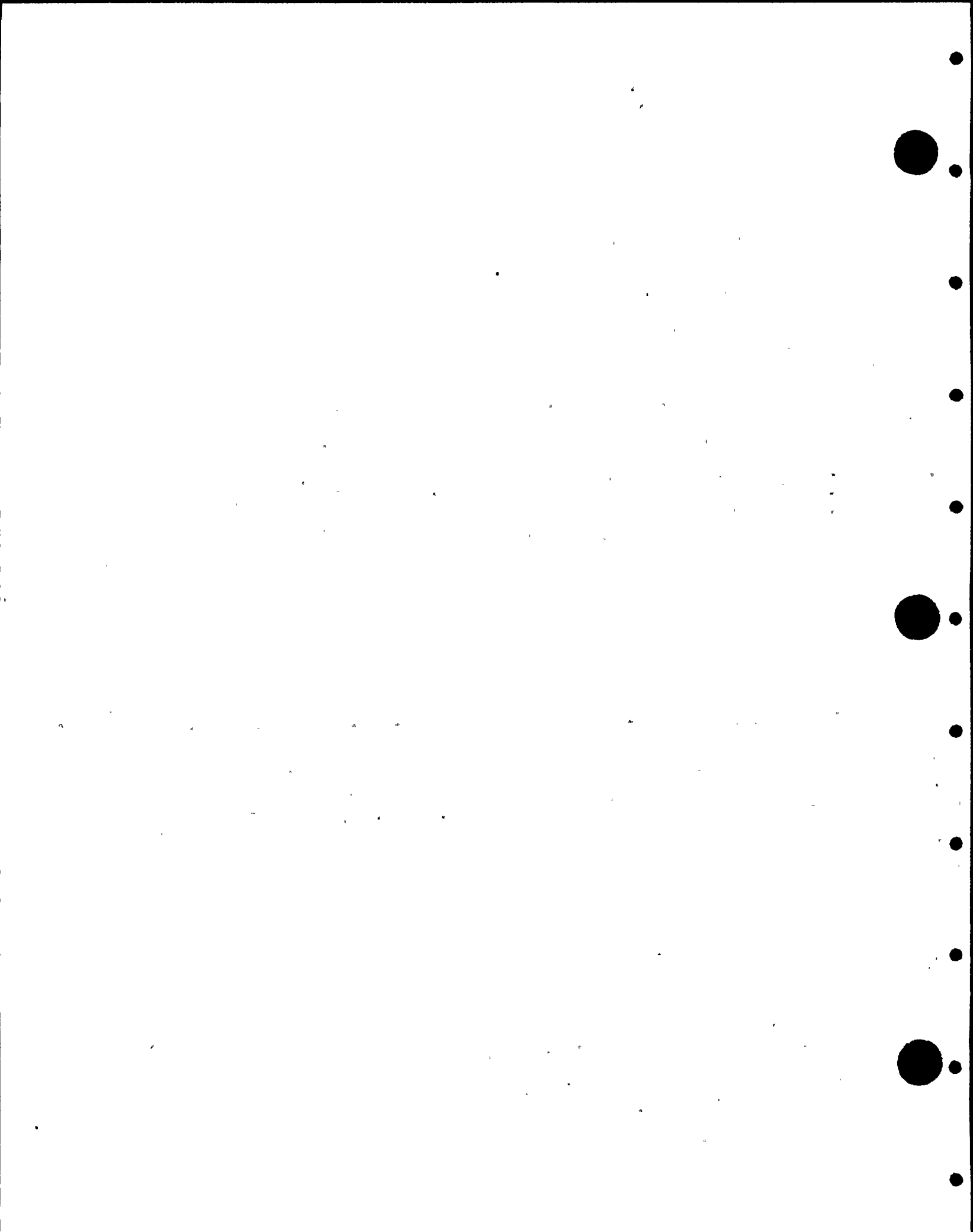
Segment STEDY Pages A3-02 to A3-09

Segment TRANS Pages A3-10 to A3-15

```

20 REM REACTOR VESSEL DEPRESSURIZATION MODEL ***** STEADY STATE *****
40 REM *** PAGE 01, FILE WORKED: *** CODE SLOWDOWN: STEADY STATE ***
60 DIM D(1,10),ELEV(11),VOL(11),SRSET(4,1),FLOW(18),VOPEN(18),FPO(18),ALL(18)
80 DIM POPEN(18),PCLOSE(18),KPRV(18),PUMP(18)
100 DEFINT I,J
120 DATES="THU, 26 MAY 84, 08:00 HR"
140 IO=0 : II=1 : IO=2 : IO=3 : IO=4 : IO=5 : IO=6 : IO=7 : IO=8 : IO=9 : IO=10 : IO=11 : IO=12 : IO=13 : IO=14 : IO=15 : IO=16 : IO=17 : IO=18
160 FO=0.0 : F1=1.0 : F2=2.0 : F100=100.0 : F1000=1000.0 : F10000=10000.0 : F100000=100000.0
180 CLS
200 PRINT:PRINT:PRINT"REACTOR DEPRESSURIZATION CALCULATION":PRINT
220 NVLV = 11 : REM NUMBER OF RELIEF VALVES
240 NDHP = 30 : REM NUMBER OF DECAY HEAT CURVE POINTS
260 NLEV = 11 : REM NUMBER OF CURVE POINTS FOR REACTOR LIQUID VOLUME VS LEVEL
280 NPMP = 11 : REM NUMBER OF CURVE POINTS FOR PUMP HEAD VERSUS VOLUMETRIC FLOW
300 NVCK = 11 : REM NUMBER OF CURVE POINTS FOR CHOILING VELOCITY VERSUS PRESSURE
320 FOR I=11 TO NVLV
340 VOPEN(I) = FO
360 NEXT I
380 REM READ DATA FOR DECAY POWER VERSUS TIME (FIRST TIME, THEN POWER)
400 FOR I=11 TO NDHP
420 READ D(11,I)
440 NEXT I
460 FOR I=11 TO NDHP
480 READ D(12,I)
500 NEXT I
520 FOR I=11 TO 15
540 REM READ 5 VALVE SETPOINTS FOR RELIEF FUNCTION
560 REM SRSET(1,1) = ON, SRSET(2,1) = OFF
580 READ SRSET(11,1), SRSET(12,1)
600 NEXT I
620 FOR I=11 TO 15
640 REM READ 5 VALVE SETPOINTS FOR SAFETY FUNCTION
660 REM SRSET(3,1) = ON, SRSET(4,1) = OFF
680 READ SRSET(13,1), SRSET(14,1)
700 NEXT I
720 REM READ LIQUID VOLUME VERSUS REACTOR LEVEL ELEVATION
740 FOR I=11 TO NLEV
760 READ ELEV(I)
780 NEXT I
800 FOR I=11 TO NLEV
820 READ VOL(I)
840 NEXT I
860 REM READ PUMP HEAD (PSID) VS PUMP FLOW (K GAL/MIN)
880 FOR I=11 TO NPMP
900 READ KPRV(I)
920 NEXT I
940 FOR I=11 TO NPMP
960 READ PUMP(I)
980 NEXT I
1000 FOR I=11 TO NVCK
1020 READ PCK(I)
1040 NEXT I
1060 FOR I=11 TO NVCK
1080 READ VCK(I)
1100 NEXT I
1120 REM LOAD VALVE SETPOINTS, USE RELIEF FUNCTION SETPOINTS (LOWER SETPOINTS)
1140 POPEN(11) = SRSET(11,1)
1160 POPEN(12) = SRSET(12,1)
1180 PCLOSE(11) = SRSET(13,1)
1200 PCLOSE(12) = SRSET(14,1)
1220 FOR N=11 TO 14
1240 REM

```

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```

1000 FILE = "NE02-86-10"
1010 TITLE = "NE-02-86-10"
1020 NEXT M
1030 NEXT P
1040 DEFSET = 50
1050 OPEN INPUT "READ DATA FROM FILE FILE T"
1060 PRINT
1070 IF VS = "N" GO TO 1880 ELSE IF VS < "Y" GO TO 1880
1080 FLX = "FILE"
1090 OPEN #11,"D1:FLX"INPUT
1100 INPUT #11,VL,VOIDFR,SYSVOL,TCOL,PO,VPOOL,TPOOL
1110 INPUT #11,SDEF,PMIN,PMAX,NDP,TMAX,DT,NPROUT
1120 CLOSE #11
1130 GO TO 1840
1140 INPUT "REACTOR LIQUID VOLUME WITHOUT LINES = "VL
1150 INPUT "REACTOR VOID FRACTION AT HOT STARTS = "VOIDFR
1160 INPUT "REACTOR SYSTEM VOLUME = "SYSVOL
1170 INPUT "REACTOR COOLANT TEMPERATURE = "TCOL
1180 INPUT "REACTOR POWER (MW) = "PO
1190 INPUT "SUPPRESSION POOL VOLUME (FT*AT) = "VPOOL
1200 INPUT "INITIAL SUPPRESSION POOL TEMPERATURE = "TPOOL
1210 INPUT "DEPRESSURIZATION START TIME = "SDEF
1220 INPUT "INITIAL CONTAINMENT PRESSURE = "PMIN
1230 INPUT "FINAL CONTAINMENT PRESSURE = "PMAX
1240 INPUT "NUMBER OF VALVES TO BE USED FOR DEPRESSURIZATION = "NDP
1250 INPUT "MAX TIME IN SEC = "TMAX
1260 INPUT "TIME STEP IN SEC = "DT
1270 INPUT "PRINT FREQUENCY (NUMBER OF TIMESTEPS) = "NPROUT
1280 CLS
1290 INPUT "WRITE INPUT TO FILE FILE T (Y/N) " :VS
1300 IF VS = "N" GO TO 2000 ELSE IF VS < "Y" GO TO 1880
1310 OPEN #11,"D1:FLX"OUTPUT
1320 PRINT #11,VL,VOIDFR,SYSVOL,TCOL,PO,VPOOL,TPOOL
1330 PRINT #11,SDEF,PMIN,PMAX,NDP,TMAX,DT,NPROUT
1340 CLOSE #11
1350 CLS
1360 T = TCOL
1370 GOSUB 7480 :REM-----STEAM TABLES
1380 RVP = P
1390 ROPF = PUMP(11)
1400 VLX = VL / (1-VOIDFR)
1410 RKAPA = 1.0 :REM ISENTROPIC EXPANSION COEFFICIENT FOR SATURATED STEAM
1420 RKX = RKAPA / (RKAPA-1) :REM SEE MARKS HANDBOOK, PAGE 4-83
1430 FCHOKE = (FL/(RKAPA-1))**RKX :REM MIN PRESSURE RATIO FOR CHOKED FLOW
1440 VE14 = 4.84 :REM VALVE THROAT DIAMETER
1450 VARA = (4.84/12.0)**2
1460 VARA = (VARA*3.141593)/4.0 :REM VALVE THROAT AREA
1470 ARV = 0.470936 :REM DISCHARGE COEFFICIENT
1480 FOR I=11 TO NVCK
1490 G=1
1500 IF PPRK(I) > RVP GO TO 1540
1510 NEXT I
1520 DVDX = (VLOK(I)-VLOK(I-11))/(PRK(I)-PRK(I-11))
1530 VEL = VLOK(I-11) + DVDX*(RVP-PRK(I-11))
1540 VLVEL = (RKX*VARA*VEL)/VE
1550 STELMB = 145000.0 :REM LB OF THERMALLY ACTIVE STEEL STRUCTURE
1560 OPEN #12,"F1:"OUTPUT
1570 INPUT "PRINT FULL SET OF INPUT DATA? (Y/N) " :YFULS
1580 IF YFULS = "N" GO TO 2500 ELSE IF YFULS < "Y" GO TO 2440
1590 GO TO 2540
1600 INPUT "PRINT THERMAL HYDRAULIC INPUT DATA? (Y/N) " :YH
1610 IF YH = "N" GO TO 2500 ELSE IF YH < "Y" GO TO 2500
1620 PRINT#12," :PRINT #12," REACTOR DEPRESSURIZATION CALCULATION - INPUT 04
1630 " :PRINT#12,"

```


[illegible]

Dm 6/15/86

```

0080 NEXT I
0090 IF VAL#="Y" GO TO 0100
0100 PRINT "STEAM TABLE"
0110 IF VAL#="N" GO TO 0120
0120 PRINT "EXAMPLE FROM STEAM TABLE"
0130 GOTO 0080
0140 PRINT #1, "      TIME      PRESS      VAP      LIQ      HE      HF      UF"
0150 GOTO 0080
0160 GOSUB 0080
0170 T = 40.0 : S = 10
0180 FOR I = 1 TO 10
0190 IF I = 5 GO TO 0190
0200 PRINT #1, " "
0210 C = 0
0220 S = C + 11
0230 T = T + 10.0
0240 GOSUB 0250 : REM ----- STEAM TABLE
0250 PRINT #1, USING "      ###.### : T;"
0260 PRINT #1, USING "      ###.### : P;"
0270 PRINT #1, USING "      #.### : VF;"
0280 PRINT #1, USING "      ###.### : VG;"
0290 PRINT #1, USING "      ###.### : HF;"
0300 PRINT #1, USING "      ###.### : HE;"
0310 PRINT #1, USING "      ###.### : UF;"
0320 PRINT #1, USING "      ###.### : SG;"
0330 NEXT I
0340 GOSUB 0080 : REM ----- 10 BLANK LINES
0350 IF VAL#="Y" GO TO 0400
0400 INPUT "PRINT DATA FOR DECAY POWER CURVE ? (Y/N) " : Y#
0410 IF Y# = "N" GO TO 0440 ELSE IF YES="Y" GO TO 0440
0420 PRINT #1, " " : PRINT #1, "      DECAY POWER CURVE "
0430 GOSUB 0080 : REM ----- 10 BLANK LINES
0440 PRINT #1, "      POINT      TIME.SEC      TIME.MIN      TIME.HF      POWER.FRAC      PD"
0450 PRINT #1, "      WER.MW"
0460 PRINT #1, " "
0470 FOR I = 1 TO 10
0480 FOR J = 0.1 TO 1.0
0490 PRINT #1, USING "      #.### : I;"
0500 TIM = 0.1
0510 PRINT #1, USING "      ###.###.### : TIM;"
0520 TIM = TIM / 60.0
0530 PRINT #1, USING "      ###.###.### : TIM;"
0540 TIM = TIM / 60.0
0550 PRINT #1, USING "      ###.###.### : TIM;"
0560 PRINT #1, USING "      #.### : D(12.1);"
0570 PRINT #1, USING "      ###.### : PDW"
0580 NEXT J
0590 GOSUB 0080 : REM ----- 10 BLANK LINES
0600 IF VAL#="Y" GO TO 0630
0630 INPUT "PRINT RELIEF VALVE DATA ? (Y/N) " : Y#
0640 IF Y# = "N" GO TO 0670 ELSE IF YES="Y" GO TO 0670
0650 PRINT #1, " " : PRINT #1, "      RELIEF VALVE DATA "
0660 GOSUB 0080 : REM ----- 10 BLANK LINES
0670 PRINT #1, "      VALVE THROAT DIAMETER = " :
0680 PRINT #1, USING "      ###.### : VDIA;"
0690 PRINT #1, "      INCH. DISCHARGE COEFFICIENT = " : VDIA : PRINT #1, " "
0700 PRINT #1, "      DISCHARGE COEFFICIENT = " :
0710 PRINT #1, USING "      #.### : CD;" : PRINT #1, " "
0720 PRINT #1, "      VALVE FLOW AT REACTOR PRESSURE OF " : PWR : PRINT #1, " " : "PSIA IS " : "PSIA : "
0730 PRINT #1, "      LB/SEC" : PRINT #1, " "
0740 PRINT #1, "      SONIC VELOCITY AT THAT PRESSURE IS " : VEL : PRINT #1, " " : "FT/SEC"

```


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```

NEXT I
GOSUB 7120 :REM-----: BLANK LINE
PRINT #17, " " "SONIC VELOCITY TABLE (FROM ASME STEAM TABLES, PAGE 399)"
GOSUB 7020 :REM-----: BLANK LINE
PRINT #17, " " "POINT FREELINE VELOCITY" : PRINT #17, " "
FOR I=11 TO NLEV
PRINT #17, LINE# " " "I;"
PRINT #17, USING " " "####,##":FPOW(I);
PRINT #17, USING " " "####,##":VLX(I)
NEXT I
GOSUB 7020 :REM-----: BLANK LINE
RNV = RNV + VARA
REM CALCULATE INITIAL LIQUID LEVEL IN REACTOR
FOR J=11 TO NLEV
IF VL>VOL(J) THEN I=J
NEXT J
DLDV = (ELEV(I+11)-ELEV(I)) / (VOL(I+11)-VOL(I))
EL=ELEV(I) + DLDV * (VL-VOL(I))
FOR J=11 TO NLEV
IF VLX>VOL(J) THEN I=J
NEXT J
DLDV = (ELEV(I+11)-ELEV(I)) / (VOL(I+11)-VOL(I))
ELX=ELEV(I) + DLDV*(VLX-VOL(I))
ELMIN = ELX : VLMIN = VLX
VV = SYSVOL - VL :REM VV=TOTAL VAPOR VOLUME (INCL. VOIDS). FT**3
VVX = VV - (VLX-VL) :REM VVX=VAPOR VOLUME ABOVE ACTUAL LIQUID LEVEL
T = TPOOL
GOSUB 7480 :REM-----:STEAM TABLE
POOLMASS = VPOOL/VF
COOLING=POOLMASS*(TPOOL - 212.0)
T = TCOL
FPOW = F1
GOSUB 7480 :REM-----:STEAM TABLE
LMASS = VL/VF :REM LIQUID MASS IN REACTOR. LB
SMASS = VV/VG :REM TOTAL VAPOR MASS (INCLUDING VOIDS). LB
SMASX = VVX/VG :REM VAPOR MASS ABOVE LIQUID LEVEL, LB
VFO = VF : VGO = VG
HFO = HF : HGO = HG
REM COOLANT QUALITY. SEE ELWAKIL PAGE 117
XFR = F1 + ((F1-VOIDFR)/VOIDFR) * (VGO/VF)
XFR = F1/XFR
CLS
PRINT:PRINT " REACTOR DEPERSSURIZATION CALCULATION "
PRINT " SYSTEM VOLUME = ":SYSVOL
PRINT " VLIG = ":VL
PRINT " VAPOR VOL = ":VV
PRINT " MASS LIQU = ":LMASS
PRINT " MASS STEAM = ":SMASS
PRINT " FLUID MASS = ": (LMASS+SMASS)/F1000:" "VLB"
PRINT #17, " " :PRINT #17, " " "#### INITIAL CONDITIONS ####" " :DATE#:" "####"
GOSUB 7020 :REM-----: BLANK LINE
PRINT #17, " " "TOTAL SYSTEM VOLUME" = "SYSVOL:" "FT**3"
PRINT #17, " " "LIQUID VOLUME WITHOUT VOIDS" = "VL:" "FT**3"
PRINT #17, " " "LIQUID VOLUME WITH VOIDS" = "VLX:" "FT**3"
PRINT #17, " " "VAPOR VOLUME INCLUDING VOIDS" = "VV:" "FT**3" (TOTAL
S)

```


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```

6500 PRINT #12, "MODE OF THERMAL HEAT EXCHANGER"
6510 PRINT #12, " "
6520 PRINT #12, " "
6530 PRINT #12, " "
6540 PRINT #12, "COLLAPSED REACTOR LEVEL" = "EL;" AT ONE VOICE
6550 PRINT #12, "ACTUAL REACTOR LEVEL" = "EL;" AT TWO VOICE
6560 PRINT #12, " "
6570 PRINT #12, "REACTOR PRESSURE" = "PS;" LB/IN2
6580 PRINT #12, "REACTOR COOLANT TEMPERATURE" = "TEMP;" FARENHEIT
6590 PRINT #12, " "
6600 PRINT #12, "AVERAGE COOLANT QUALITY" = " "
6610 PRINT #12, USING "##,###"; YPER*100;
6620 PRINT #12, "PERCENT"
6630 PRINT #12, "AVERAGE COOLANT VOID FRACTION" = " "
6640 PRINT #12, USING "##,###"; VOIDFR*100;
6650 PRINT #12, "PERCENT"
6660 VLYFR = VOL(9)
6670 VOIDFR = (VLYFR-VL)/VULXFR*100
6680 PRINT #12, "CORRESPONDING AVERAGE VOLUMETRIC VOID AT "
6690 PRINT #12, "FULL POWER, WITH LEVEL AT SETPOINT LN" = " "
6700 PRINT #12, USING "##,###"; VOIDFR;
6710 PRINT #12, "PERCENT"
6720 PRINT #12, " "
6730 PRINT #12, " "
6740 PRINT #12, "SPECIFIC VOLUME OF LIQUID" = "VLO;" FT3/LB
6750 PRINT #12, "SPECIFIC VOLUME OF VAPOR" = "VVO;" FT3/LB
6760 PRINT #12, "SPECIFIC ENTHALPY OF LIQUID" = "HLO;" BTU/LB
6770 PRINT #12, "SPECIFIC ENTHALPY OF VAPOR" = "HVO;" BTU/LB
6780 GO = (PO*1000.0*3413.0)/PTE; REM THERMAL POWER IN BTU/SEC
6790 MLPOI = FO : TDO=FO : TYME=FO : LRFLOW=FO : PCOIN=FO
6800 REM
6810 OPEN #12, "D1:SERIES.ATE" OUTPUT
6820 PRINT #12, IC, IL, ID, IT, IA, IB, IX, IIT, IIS, FI, FI, F100, F1000
6830 PRINT #12, NVLV, NDOP, NLEV, NPMP, NVOK, DENSET, NPRINT, VPOOL
6840 FOR I=1 TO NDOP
6850 PRINT #12, D(I1,I), D(I2,I)
6860 NEXT I
6870 FOR I=1 TO NLEV
6880 PRINT #12, ELEV(I), VOL(I)
6890 NEXT I
6900 FOR I=1 TO NPMP
6910 PRINT #12, MPMP(I), PLMP(I)
6920 NEXT I
6930 FOR I=1 TO NVOK
6940 PRINT #12, PROK(I), VLOK(I)
6950 NEXT I
6960 FOR I=1 TO NVLV
6970 PRINT #12, POPEV(I), POLOBE(I)
6980 NEXT I
6990 PRINT #12, VL, VLY, VOIDFR, SYSVOL, FO, GO, VPOOL, TPOOL, ELX, ELMIN, POOLMASS
7000 PRINT #12, SDEF, PMIN, PMAX, NDOP, TMAX, TYME, DT, NPRINT
7010 PRINT #12, ULMIN, PU, VLY, POOLLEN, FLOW, LMASS, EMAS, EMASX, POFF, AKU, STLMASS
7020 PRINT #12, MLPOI, LRFLOW, ADDFLOW, SPFLOW, TDO, PPOKT, T, RVP, FROCKE
7030 CLOSE #12
7040 PRINT #12, " " : PRINT #12, " *** ALL DATA FOR STEADY STATE WRITTEN TO DISK CHANNEL 2 *** "
7050 PRINT #12, " " : PRINT #12, " *** " " : DATE: " "
7060 PRINT #12, " " : PRINT #12, " *** "

```



```

7600 F = -30.108E + 1*( 8.119E9 + 1*(-0.732E29 + 1*(0.022E5B7 + 1*(0.004E5C1D1))
7610 VF = 0.53704E + 1*(-0.1E9 + 1*( 0.017E11E + 1*(-0.001E04E2 + 1*(0.00017E19))
7700 VE = -0.25370E + 1*( 7.11E8E + 1*(-0.4E07E + 1*(0.016E1E5E + 1*(0.004E0117E))
7710 HF = 311E.7E + 1*( -1070.2E + 1*( 170.7E + 1*( -11.2E7E + 1*(7.447E2))
7740 HE = -1048A.E + 1*( 360E.4E + 1*(-1E0.01E + 1*(3E.10E + 1*(7.1E19))
7760 P = EXP(P)
7780 VE = EXP(VF)
7800 IF T > 750 E2 TO 7860
7820 IF T > 410 E2 TO 7860
7840 XTT = (-TFO.01/20.0
7860 F = 320.2E1 + 5E.5E * XTT
7880 VF = 0.01847E + 0.0007E * XTT
7900 VE = 1.087E + 0.41E * XTT
7920 HF = 32E.9E + 12.6E * XTT
7940 HE = 1200.2 + 2.4 * XTT
7960 EF = HF - P * 0.1250E * VF
7980 EG = HE - F * 0.1250E * VE
8000 RETURN
8020 REM-----SMALL PRINTOUT ROUTINE
8040 PRINT #13, " FT*3"
8060 RETURN
8080 REM SMALL PRINTOUT ROUTINE
8100 PRINT #13, " " : PRINT #13, " "
8120 RETURN
8140 REM DECAY POWER (FRACTION) VERSUS TIME (SEC) - SEE FEAR FIG. 1E.C-2
8160 REM FEAR TABLE 6.2-11 (PAGE 6.2-10E) AND FIG. 6.2-17 & 3E MAY BE *
8180 REM TIME IN SEC. 30 VALUES
8200 DATA 0.0,1.0,2.0,3.5,5.0,6.5,8.9
8220 DATA 10.0,20.0,30.0,40.0,50.0,60.0,70.0,80.0,90.0,100.0,110.0,120.0,130.0,140.0,150.0,160.0,170.0,180.0,190.0,200.0,210.0,220.0,230.0,240.0,250.0,260.0,270.0,280.0,290.0,300.0
8240 DATA 700.0,800.0,900.0,1000.0,1100.0,1200.0,1300.0,1400.0,1500.0,1600.0,1700.0,1800.0,1900.0,2000.0,2100.0,2200.0,2300.0,2400.0,2500.0,2600.0,2700.0,2800.0,2900.0,3000.0,3100.0,3200.0,3300.0,3400.0,3500.0,3600.0,3700.0,3800.0,3900.0,4000.0,4100.0,4200.0,4300.0,4400.0,4500.0,4600.0,4700.0,4800.0,4900.0,5000.0,5100.0,5200.0,5300.0,5400.0,5500.0,5600.0,5700.0,5800.0,5900.0,6000.0,6100.0,6200.0,6300.0,6400.0,6500.0,6600.0,6700.0,6800.0,6900.0,7000.0,7100.0,7200.0,7300.0,7400.0,7500.0,7600.0,7700.0,7800.0,7900.0,8000.0,8100.0,8200.0,8300.0,8400.0,8500.0,8600.0,8700.0,8800.0,8900.0,9000.0,9100.0,9200.0,9300.0,9400.0,9500.0,9600.0,9700.0,9800.0,9900.0,10000.0,10100.0,10200.0,10300.0,10400.0,10500.0,10600.0,10700.0,10800.0,10900.0,11000.0,11100.0,11200.0,11300.0,11400.0,11500.0,11600.0,11700.0,11800.0,11900.0,12000.0,12100.0,12200.0,12300.0,12400.0,12500.0,12600.0,12700.0,12800.0,12900.0,13000.0,13100.0,13200.0,13300.0,13400.0,13500.0,13600.0,13700.0,13800.0,13900.0,14000.0,14100.0,14200.0,14300.0,14400.0,14500.0,14600.0,14700.0,14800.0,14900.0,15000.0,15100.0,15200.0,15300.0,15400.0,15500.0,15600.0,15700.0,15800.0,15900.0,16000.0,16100.0,16200.0,16300.0,16400.0,16500.0,16600.0,16700.0,16800.0,16900.0,17000.0,17100.0,17200.0,17300.0,17400.0,17500.0,17600.0,17700.0,17800.0,17900.0,18000.0,18100.0,18200.0,18300.0,18400.0,18500.0,18600.0,18700.0,18800.0,18900.0,19000.0,19100.0,19200.0,19300.0,19400.0,19500.0,19600.0,19700.0,19800.0,19900.0,20000.0,20100.0,20200.0,20300.0,20400.0,20500.0,20600.0,20700.0,20800.0,20900.0,21000.0,21100.0,21200.0,21300.0,21400.0,21500.0,21600.0,21700.0,21800.0,21900.0,22000.0,22100.0,22200.0,22300.0,22400.0,22500.0,22600.0,22700.0,22800.0,22900.0,23000.0,23100.0,23200.0,23300.0,23400.0,23500.0,23600.0,23700.0,23800.0,23900.0,24000.0,24100.0,24200.0,24300.0,24400.0,24500.0,24600.0,24700.0,24800.0,24900.0,25000.0,25100.0,25200.0,25300.0,25400.0,25500.0,25600.0,25700.0,25800.0,25900.0,26000.0,26100.0,26200.0,26300.0,26400.0,26500.0,26600.0,26700.0,26800.0,26900.0,27000.0,27100.0,27200.0,27300.0,27400.0,27500.0,27600.0,27700.0,27800.0,27900.0,28000.0,28100.0,28200.0,28300.0,28400.0,28500.0,28600.0,28700.0,28800.0,28900.0,29000.0,29100.0,29200.0,29300.0,29400.0,29500.0,29600.0,29700.0,29800.0,29900.0,30000.0
8260 REM DECAY POWER IN FRACTION, 30 VALUES
8280 DATA 1.0,0.997,0.993,0.989,0.985,0.981,0.977,0.973,0.969,0.965,0.961,0.957,0.953,0.949,0.945,0.941,0.937,0.933,0.929,0.925,0.921,0.917,0.913,0.909,0.905,0.901,0.897,0.893,0.889,0.885,0.881,0.877,0.873,0.869,0.865,0.861,0.857,0.853,0.849,0.845,0.841,0.837,0.833,0.829,0.825,0.821,0.817,0.813,0.809,0.805,0.801,0.797,0.793,0.789,0.785,0.781,0.777,0.773,0.769,0.765,0.761,0.757,0.753,0.749,0.745,0.741,0.737,0.733,0.729,0.725,0.721,0.717,0.713,0.709,0.705,0.701,0.697,0.693,0.689,0.685,0.681,0.677,0.673,0.669,0.665,0.661,0.657,0.653,0.649,0.645,0.641,0.637,0.633,0.629,0.625,0.621,0.617,0.613,0.609,0.605,0.601,0.597,0.593,0.589,0.585,0.581,0.577,0.573,0.569,0.565,0.561,0.557,0.553,0.549,0.545,0.541,0.537,0.533,0.529,0.525,0.521,0.517,0.513,0.509,0.505,0.501,0.497,0.493,0.489,0.485,0.481,0.477,0.473,0.469,0.465,0.461,0.457,0.453,0.449,0.445,0.441,0.437,0.4
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20 REM REACTOR VESSEL DEPRESSURIZATION MODEL ***** TRANSIENT STATE *****
40 REM * PAGE OF * DRIVE 1 * FILE WORKING * CODE SLOWDOWN. 11 TRANS *
60 REM ***ADJUSTED FOR MONIFLOW LINE*** 14 JUL 71 ***
80 DIM D(12,30),ELEV(12),VOL(12),FLOD(12),FOPEN(12),PCLOSE(12),PUMP(12)
100 DIM FOPEN(12),PCLOSE(12),KEPM(12),PUMP(12)
120 DEFINT I,J
140 DATES="THU. 29 MAY 66. 11:38 H5"
160 CLS
180 PRINT:PRINT:PRINT"REACTOR DEPRESSURIZATION CALCULATION":PRINT
200 OPEN #3,"P1:OUTPUT"
220 PRINT #3," ":PRINT #3," REACTOR DEPRESSURIZATION CALCULATION-TRANSIENT PA
RT-":DATES
240 REM
260 OPEN #2,"D1:DEPRESS.ETE" INPUT
280 REM
300 INPUT #2,I0,I1,I2,I3,I4,I5,I6,I7,I8,I9,F0,F1,F2,F100,F1000
320 INPUT #2,NVLV,NDHF,NLEV,NPMF,NVCK,DEPSET,NPRINT,VPOOL
340 FOR I=I1 TO NDHF
360 INPUT #2,D(I,1),D(I,2),D(I,3)
380 NEXT I
400 FOR I=I1 TO NLEV
420 INPUT #2,ELEV(I),VOL(I)
440 NEXT I
460 FOR I=I1 TO NPMF
480 INPUT #2,KEPM(I),PUMP(I)
500 NEXT I
520 FOR I=I1 TO NVCK
540 INPUT #2,FROCK(I),VLCK(I)
560 NEXT I
580 FOR I=I1 TO NVLV
600 INPUT #2,FOPEN(I),PCLOSE(I)
620 NEXT I
640 INPUT #2,VL,VLX,VOIDFR,SYEVOL,FO,GO,VPOOL,TPCOL,ELX,ELMIN,FOOLMASS
660 INPUT #2,SDEP,PMIN,PMAX,NDF,TMAX,TYME,DT,NPROUT
680 INPUT #2,VLMIN,VV,VVX,FOOLENG,FROW,LMASS,SMASS,SMASX,POFF,PKU,STLMAS
700 INPUT #2,MLFOI,LPIFLOW,ADFLOW,SRVFLOW,TDS,PCONT,T,RWF,FOCKE
720 CLOSE #2
740 LBAF = ELEV(2)
760 LTAF = ELEV(3)
780 VTAF = VOL(3)
800 PRTLVI = F0 - F100
820 PRTLVI2 = F0 - F100
840 GOSUB 3720 :REM-----STEAM TABLES
860 HFOLD=HF
880 SDEP = 600.0
900 NDF = 6
920 HFPOL=TPCOL
940 STMOLD = SMASS - SMASX :REM STEAM MASS INSIDE LIQUID
960 TBUEL = 4.7 :REM PARAMETER FOR LEVEL SWELL EFFECT
980 TAUSTL = 4.0 :REM PARAMETER FOR HEAT TRANSFER FROM STRUCTURAL STEEL
1000 QSTEEL = F0
1020 TSTEEL = T
1040 STLMAS = 1.65 * 1000.0 * 1000.0 :REM LB OF STRUCTURAL STEEL
1060 CSTEEL = 0.127 :REM BTU/(LB*F)
1080 TMAX = 3600.0
1100 FPS = F0
1120 QLPOO = 7.45 :REM AT 7.45-KGAL/MIN. DFFRIC=115.5 FT
1140 DFFRIC = F0
1160 FRODEF = 115.5 * (14.7/33.93) :REM COEFFICIENT IN PSID
1180 OPEN #11,"D1:ADS.DAT" OUTPUT
1200 PRINT #13," ":PRINT #13," DEPRESSURIZATION STARTS AT ":SDEP;
1220 PRINT #17," SET, TRIP TO "

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1280 VTIME = EL
1280 REM MAIN ITERATING LOOP
1300 IF VTIME > 20.0 THEN WPR = 50
1320 REM IF VTIME = 20.0 THEN DT = F1
1340 VTIME = VTIME - DT
1360 PRINT VTIME;" SEC"
1380 GOSUB 5500 :REM----->DECAY POWER
1400 DG = FPDW * QD * DT
1420 TDG = TDG + DG
1440 FPS = FPS - FPDW*DT :REM FULL POWER SECONDS
1460 GOSUB 3700 :REM----->STEAM TABLES
1480 GOSUB 3940 :REM----->MASSFLOW THRU RELIEF VALVES
1500 SRVMAS = (SRVFLOW+ADSFLOW)*DT :REM MASS LOST THRU SRV
1520 SRVENG = SRVMAS * HG :REM ENERGY LOST THRU SRV
1540 TZ = T : TLPCI=TPOOL : T=TLPCI
1560 GOSUB 3700 :REM----->STEAM TABLES
1580 T = TZ
1600 PCONT = PMIN + ((PMAX-PMIN)/1200.0)*TYME
1620 IF PCONT>PMAX THEN PCONT = PMAX
1640 DELP = (521.021 + EL - 466.229) / (VF*144)
1660 DELP = DELP + RVP - PCONT
1680 IF DELP > PDFF GO TO 1740
1700 GOSUB 3440 :REM----->RHP PUMP FLOW
1720 LPIFLOW = CLPCI/(7.4805*60.0*VF)
1740 MLPCI = LPIFLOW * DT
1760 HFPDOL = HF
1780 ELPCI = MLPCI * HFPDOL
1800 POOLMASS = POOLMASS + SRVMAS - MLPCI
1820 POOLENG = POOLENG + SRVENG - ELPCI
1840 TPOOL = (POOLENG/POOLMASS) + 32.0
1860 GOSUB 4980 :REM----->NEW THERMAL STATE
1880 GSTEEL = (CSTEEL*STLMAS*(TSTEEL-T)) * DT/TAUSTL
1900 TSTEEL = TSTEEL - GSTEEL/(CSTEEL*STLMAS)
1920 GOSUB 5620 :REM----->VOID FRACTION
1940 IF VLX => VLMIN GO TO 2000
1960 VLMIN = VLX
1980 TMIN = TYME
2000 TZ = T
2020 T = TPOOL
2040 GOSUB 3700 :REM----->STEAM TABLES
2060 VPOOL = POOLMASS * VF
2080 HFPDOL = HF
2100 T = TZ
2120 NPRINT = NPRINT + 11
2140 IF NPRINT<NPROUT THEN GO TO 1280
2160 GOSUB 6500 :REM----->WRITE ON SCREEN
2180 GOSUB 6320 :REM----->REACTOR LEVEL
2200 IF PRTLVI > F0 GO TO 2300
2220 IF ELX > LTAF GO TO 2300
2240 PRTLVI = F100
2260 TUNC = TYME
2280 PRINT #13," *** TOP OF FUEL UNCOVERED AT ":TYME;" SEC, LEV= ":ELX;" FT"
2300 IF PRTLVI < F1 GO TO 2500
2320 IF PRTLVI > F0 GO TO 2500
2340 IF ELX < LTAF GO TO 2500
2360 PRTLVI = F100
2380 TEXP = TYME - TUNC
2400 DELUND = F100*(LTAF-ELMIN)/(LTAF-LBAF)
2420 PRINT #13," *** TOP OF FUEL RESUBMERGED AT ":TYME;" SEC, LEV= ":ELX;" FT"
2440 PRINT #13," *** TOP OF FUEL WAS UNCOVERED FOR ":TEXP;" SEC "
2460 PRINT #13," *** AT LOWEST LEVEL, ":DELUND;" PCT OF CORE WAS UNCOVERED"
2480 PRINT #13," *** MIN LEVEL WAS ":ELMIN;" FT, AT T= ":TMIN;" SEC"
2500 IF VLINE < VLF GO TO 2560

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3720 IF = 1.0 - 1.0E-10
3780 IF = 1.0 - 1.0E-10
3820 IF = 1.0 - 1.0E-10
3840 IF = 1.0 - 1.0E-10
3860 IF = 1.0 - 1.0E-10
3880 IF = 1.0 - 1.0E-10
3900 IF = 1.0 - 1.0E-10
3920 IF = 1.0 - 1.0E-10
3940 IF = 1.0 - 1.0E-10
3960 IF = 1.0 - 1.0E-10
3980 IF = 1.0 - 1.0E-10
4000 IF = 1.0 - 1.0E-10
4020 IF = 1.0 - 1.0E-10
4040 IF = 1.0 - 1.0E-10
4060 IF = 1.0 - 1.0E-10
4080 IF = 1.0 - 1.0E-10
4100 IF = 1.0 - 1.0E-10
4120 IF = 1.0 - 1.0E-10
4140 IF = 1.0 - 1.0E-10
4160 IF = 1.0 - 1.0E-10
4180 IF = 1.0 - 1.0E-10
4200 IF = 1.0 - 1.0E-10
4220 IF = 1.0 - 1.0E-10
4240 IF = 1.0 - 1.0E-10
4260 IF = 1.0 - 1.0E-10
4280 IF = 1.0 - 1.0E-10
4300 IF = 1.0 - 1.0E-10
4320 IF = 1.0 - 1.0E-10
4340 IF = 1.0 - 1.0E-10
4360 IF = 1.0 - 1.0E-10
4380 IF = 1.0 - 1.0E-10
4400 IF = 1.0 - 1.0E-10
4420 IF = 1.0 - 1.0E-10
4440 IF = 1.0 - 1.0E-10
4460 IF = 1.0 - 1.0E-10
4480 IF = 1.0 - 1.0E-10
4500 IF = 1.0 - 1.0E-10
4520 IF = 1.0 - 1.0E-10
4540 IF = 1.0 - 1.0E-10
4560 IF = 1.0 - 1.0E-10
4580 IF = 1.0 - 1.0E-10
4600 IF = 1.0 - 1.0E-10
4620 IF = 1.0 - 1.0E-10
4640 IF = 1.0 - 1.0E-10
4660 IF = 1.0 - 1.0E-10
4680 IF = 1.0 - 1.0E-10
4700 IF = 1.0 - 1.0E-10
4720 IF = 1.0 - 1.0E-10
4740 IF = 1.0 - 1.0E-10
4760 IF = 1.0 - 1.0E-10
4780 IF = 1.0 - 1.0E-10
4800 IF = 1.0 - 1.0E-10
4820 IF = 1.0 - 1.0E-10
4840 IF = 1.0 - 1.0E-10
4860 IF = 1.0 - 1.0E-10
4880 IF = 1.0 - 1.0E-10
4900 IF = 1.0 - 1.0E-10
4920 IF = 1.0 - 1.0E-10
4940 IF = 1.0 - 1.0E-10
4960 IF = 1.0 - 1.0E-10
4980 IF = 1.0 - 1.0E-10
5000 IF = 1.0 - 1.0E-10

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4020 VG = EXP(VG)
4040 IF = 350 GO TO 4200
4060 IF = 410 GO TO 4200
4080 XTT = (T-350.0) / 20.0
4100 F = 220.461 + 55.31 * XTT
4120 VF = 0.01847 + 0.000037 * XTT
4140 VG = 1.08725 + 0.4154 * XTT
4160 HF = 361.912 + 12.625 * XTT
4180 HG = 1200.2 + 2.4 * XTT
4200 EF = HF - F * 0.12509 * VF
4220 EE = HG - F * 0.12509 * VG
4240 RETURN
4260 REM-----PRINT TWO BLANK LINES
4280 PRINT #13, " " : PRINT #13, " "
4300 RETURN
4320 REM-----PRINT HEADLINES
4340 PRINT #13, " "
4360 PRINT #13, " MODEL READ DECAY READ READ READ READ SUPP MAN
4380 PRINT #13, " TIME TEMP POWER PRES LIQUID LEVEL LIQUID POOL FLOW
4400 PRINT #13, " SEC FAHF FRAD PSIA FT**3 FT KLB TEMP LB/S
4420 PRINT #13, " "
4440 NLINE = 10
4460 RETURN
4480 REM-----PRINT RESULTS TO PAPER
4500 REM PRINT #1, T, F, P, V, L, EL, L, MASS, T, POOL, SRV, MASS, DT, SRV, FLOW, M, DT, DT
4520 PRINT #13, USING " ####.#": T;
4540 PRINT #13, USING " ####.#": F;
4560 REM IF LPIFLOW=FO GO TO 4620
4580 REM PRINT#13, USING " ####.#": LL;
4600 REM GO TO 4640
4620 PRINT #13, USING " ####.#": FLOW;
4640 PRINT #13, USING " ####.#": SRV;
4660 PRINT #13, USING " ####.#": VLX;
4680 PRINT #13, USING " ####.#": ELX;
4700 PRINT #13, USING " ####.#": LMASS/F1000;
4720 PRINT #13, USING " ####.#": TPOOL;
4740 PRINT #13, USING " ####.#": ADEFLOW;
4760 IF LPIFLOW = FO GO TO 4820
4780 PRINT #13, USING " ####.#": VOIDFR*F100;
4800 GO TO 4840
4820 PRINT #13, USING " ####.#": SRVFLOW;
4840 REM IF LPIFLOW < FO GO TO 4900
4860 REM PRINT #13, USING " ####.#": LL;
4880 REM GO TO 4920
4900 IF LPIFLOW < FO GO TO 4900
4920 PRINT #13, USING " ####.#": VOIDFR*F100;
4940 GO TO 4920
4960 PRINT #13, USING " ####.#": SRVFLOW;

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RETURN

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4980 REMK-----CALCULATE NEW THERMAL STATE
5000 L=11: FOLD=F1: FNEW=F1: DTEMP=F1/F2: EPMAX=7071000
5010 LMASS=LMASS+MLPDI
5020 SMASS=SMASS-BFUMASS:REM TOTAL STEAM, INCLUDING VOIDS
5030 TMASS=LMASS-SMASS
5040 SMOLD=SMASS
5050 GDSUB 5720:REM-----STEAM TABLES
5110 HEVAP=HG-HF
5140 QDHF=(HFOLD-HF)*LMASS
5160 QPOOL=(HFPOL-HF)*MLPDI
5180 DONET=DQ+QDHF+QPOOL-QSTEEL
5200 STMNEW=DONET/HEVAP:REM NEW STEAM INSIDE LIQUID
5220 SMASS=SMOLD+STMNEW:REM TOTAL STEAM, INSIDE LIQUID AND ABOVE LIQUID
5240 LMASS=TMASS-SMASS
5260 VOL=SMASS*VG+LMASS*VF
5280 VOLERR=VOL-SYSVOL
5300 IF ABS(VOLERR/SYSVOL)<EPMAX GO TO 5500
5320 FNEW=SGN(VOLERR)
5340 IF SGN(FNEW)=SGN(FOLD) GO TO 5380
5360 DTEMP=DTEMP/F2
5380 T=T+FNEW*DTEMP
5400 IF L>29 THEN PRINT #13,"CONVERGENCE - L,SMOLD,SMASS,VOL,VOLERR ":L;SMOLD;SMASS;VOL;VOLERR
5420 IF L>29 THEN PRINT #13,"CONVERGENCE - L,T,P,DQ,DONET,STMNEW,HEVAP ":L;T;P;DQ;DONET;STMNEW;HEVAP
5440 FOLD=FNEW
5460 L=L+11
5480 IF L<36 GO TO 5100
5500 HFOLD=HF
5520 RVP=P
5540 IF RVP<PCONT THEN RVP=PCONT
5560 VL=LMASS*VF
5580 LL=L
5600 RETURN
5620 REMK-----CALCULATE VOID FRACTION
5640 FACTOR=F1-3T/TBUBL
5660 IF FACTOR<F0 THEN FACTOR=F0
5680 STMTOT=STMOLD*FACTOR+STMNEW:REM STEAM MASS INSIDE LIQUID
5700 VOID=STMTOT*VG
5720 VLX=VL+VOID
5740 VOIDFR=VOID/VLX
5760 REM PRINT #3," "
5780 REM PRINT #3,"STMTOT,VOID,VOIDFR,HFEXC,STMNEW ":STMTOT;VOID;VOIDFR;HFEXC;STMNEW:PRINT #3," "
5800 REM PRINT #3,"STMOLD,SMASS,DGTOT,DONET,HEVAP":STMOLD;SMASS;DGTOT;DONET;HEVAP:PRINT #3," "
5820 REM PRINT #3,"TIME,PRES,TEMP,HFOLD,HFLMAS";TYME;P;T;HFOLD;HF;LMASS:PRINT #3," "
5840 STMOLD=STMTOT
5860 RETURN
5880 REMK-----CALCULATE DECAY POWER
5900 FOR J=11 TO NDHP
5920 K=J
5940 IF D(I1,J)>TYME GO TO 5980
5960 NEXT J
5980 IF K=>8 GO TO 6160
6000 XX=TYME-DT/F2
6020 X1=D(I1,K-11)
6040 X2=D(I1,K)
6060 Z1=D(I2,K-11)
6080 Z2=D(I2,K)
6100 DPDT=(Z2-Z1)/(X2-X1)
6120 FROM=11-DT/F2+X1

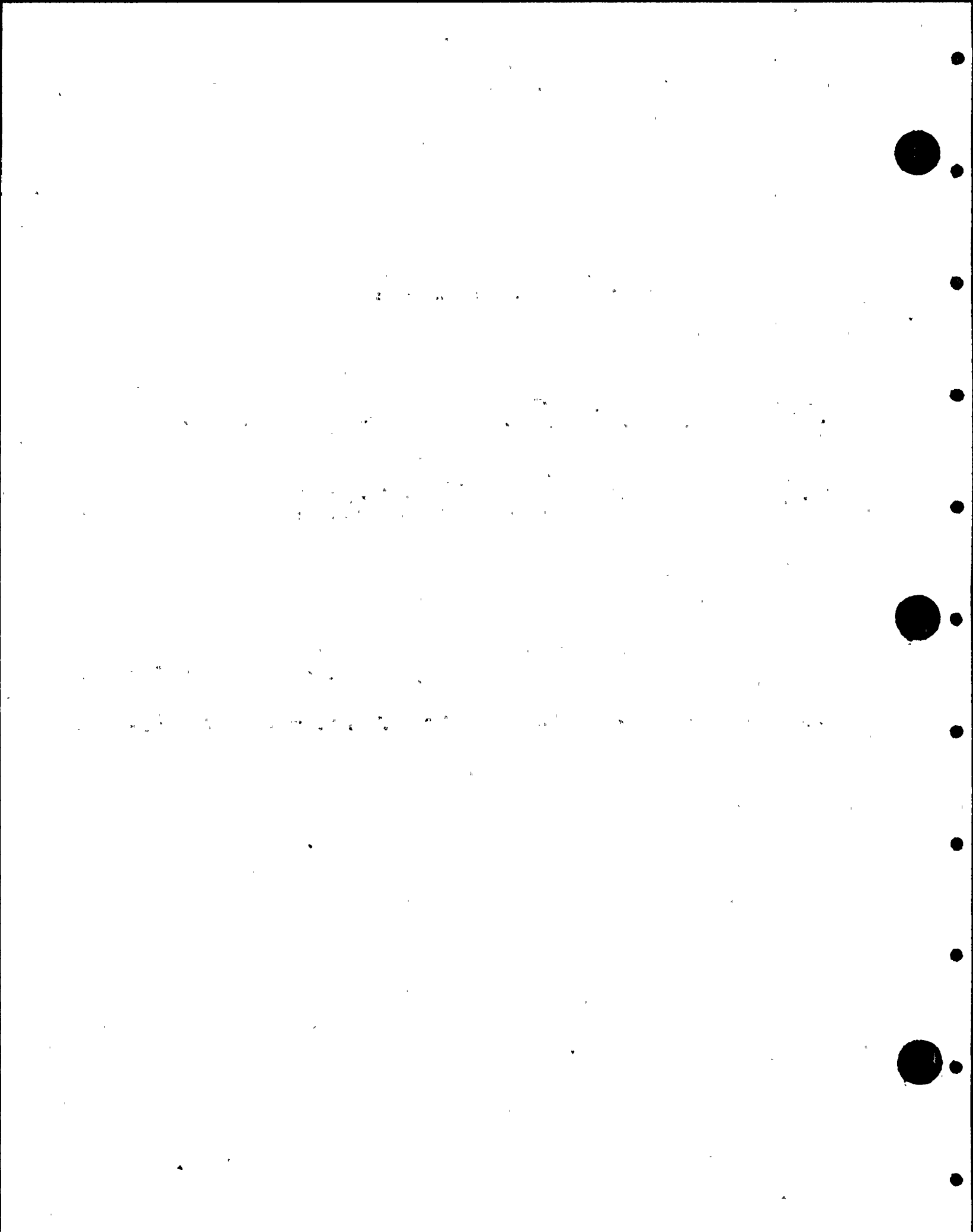
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6200 X1 = LOG(D(11,X))
6220 Y1 = LOG(D(11,Y-1))
6240 Z1 = LOG(D(11,Z))
6260 DPOW = Z1 - Y1 * (X2 - X1)
6280 FPOW = EXP(Z1 - DPOW * (XY - Y1))
6300 RETURN
6320 REM-----CALCULATE REACTOR LEVEL
6340 FOR J=11 TO NLEV
6360 IF VLX > VOL(J) THEN I=J
6380 NEXT J
6400 OLDV = (ELEV(I-1) - ELEV(I)) / (VOL(I-1) - VOL(I)) : REM GRADIENT, FT/FT**2
6420 SLX = ELEV(I) - OLDV * (VLY - VOL(I))
6440 REM PRINT #3, "VOID, VOIDFR, HPEXC, STMNEW " : VOID : VOIDFR : HPEXC : STMNEW : PRINT #
3, " "
6460 IF ELY < ELMIN THEN ELMIN = ELY
6480 RETURN
6500 REM-----WRITE ON CRT SCREEN
6520 CLS : PRINT "*****"
6540 PRINT "WNP-2 DEPRESSURIZATION CALCULATION "
6560 PRINT: PRINT "    TIME = ": TYNE : "SEC": PRINT: PRINT
6580 PRINT "TEMPERATURE      = ": T
6600 PRINT "PRESSURE          = ": RVP
6620 PRINT "DECAY HEAT          = ": FPS : " FULL POWER SECONDS"
6640 GO TO 6900
6660 PRINT "TOTAL DECAY HEAT (BTU) = ": TEG
6680 PRINT "FRACTIONAL POWER = ": FPOW
6700 PRINT "LIQUID MASS          = ": LMASS
6720 PRINT "STEAM MASS           = ": SMASS
6740 PRINT "SRV FLOW (LB/SEC)    = ": SRVMASSEDOT
6760 PRINT "SRV ENERGY REMOVAL (BTU/SEC) = ": SRVENSDOT
6780 PRINT "DECAY HEAT (BTU/SEC) = ": DDOTDOT
6800 PRINT "LPCI FLOW (LB/SEC)    = ": MLPCIDOTDOT : "    LPCI TEMP = ": TLPCI
6820 PRINT "POOL MASS            = ": POOLMASS : "    POOL VOLUME = ": VPOOL
6840 PRINT "POOL TEMPERATURE     = ": TPOOL
6860 PRINT "MINIMUM VESSEL VOL    = ": VLMIN : "    AT ": TMIN : " SEC"
6880 PRINT "VESSEL LIQUID VOLUME = ": VL
6900 PRINT "WATER LEVEL ELEVATION = ": ELX
6920 PRINT "MIN WATER LEVEL     = ": ELMIN : "AT TIME ": TMIN
6940 RETURN

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APPENDIX 4

Printouts of Computer Results
from Code BLOWDOWN.01

Segment STEDY Pages A4-02 to A4-06

Segment TRANS Pages A4-07 to A4-25

REACTOR DEPRESSURIZATION CALCULATION - INPUT DATA - CONT. OF A4-01 EL. 10.11-15
INITIALIZATION ROUTINE, CALCULATED STEADY STATE

On 8/15/86

REACTOR LIQUID VOLUME WITHOUT VESSEL = 11146.0 FT**3
REACTOR VOID FRACTION AT HOT STANDBY = .025
REACTOR SYSTEM VOLUME = 21005.0 FT**3
REACTOR COOLANT TEMPERATURE = 281.5 FAHRENHEIT
REACTOR POWER (THERMAL) = 3487 MW

SUPPRESSION POOL VOLUME = 117007.0 FT**3
INITIAL POOL TEMPERATURE = 91 FAHRENHEIT
RHP PUMP SHUTTER HEAD = 707.5 PSID

DEPRESSURIZATION START TIME = 600 SEC
INITIAL CONTAINMENT PRESSURE = 15 LB/IN**2 ABS
FINAL CONTAINMENT PRESSURE = 10 LB/IN**2 ABS

NUMBER OF VALVES TO BE USED FOR DEPRESSURIZATION = 2

MAXIMUM TIME = 1800 SEC, TIMESTEP = .5 SEC
PRINT FREQUENCY (NR OF TIMESTEPS) = 2

LIQUID VOLUME VERSUS REACTOR LEVEL

	LEVEL, FT	VOLUME, FT**3	
1	0.000	0.0	BOT OF VES
2	18.024	4658.0	BOT ACT FUEL
3	30.524	8387.0	TOP ACT FUEL
4	31.540	8478.0	LEVEL L1
5	40.792	11748.0	LEVEL L2
6	45.000	13034.0	LEVEL L3
7	46.593	13535.0	LEVEL L4
8	46.593	13535.0	LEVEL LN (NORMAL)
9	47.033	13636.0	LEVEL L7
10	48.587	14201.0	LEVEL L8
11	49.920	14572.0	4 IN ABOVE L8
12	54.000	15837.0	CENTERLINE STEAMLINE
13	72.540	21005.0	TOP OF VESSEL

RHP PUMP CURVE

	PUMP FLOW, KGAL/MIN	PUMP HEAD, PSID	PUMP HEAD, FT
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1	0.00	307.20	700.01
2	0.50	298.30	681.77
3	1.00	287.40	663.54
4	1.50	279.50	645.30
5	2.00	271.70	627.09
6	2.50	263.30	608.05
7	3.00	255.00	589.21
8	3.50	247.00	570.00

Dec 6/15/50

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NAME IN FROM STEAK TABLE

TEMP	WSPD	VS	VS	WS	WS	WS	WS
50	0.17382	0.018000	1116.58000	111.571	1104.1	111.571	1077.8
60	0.17382	0.018000	1109.58000	111.571	1097.1	111.571	1070.8
70	0.17382	0.018000	1102.58000	111.571	1090.1	111.571	1063.8
80	0.17382	0.018000	1095.58000	111.571	1083.1	111.571	1056.8
90	0.17382	0.018000	1088.58000	111.571	1076.1	111.571	1049.7
100	0.17382	0.018000	1081.58000	111.571	1069.1	111.571	1042.7
110	0.17382	0.018000	1074.58000	111.571	1062.1	111.571	1035.7
120	0.17382	0.018000	1067.58000	111.571	1055.1	111.571	1028.7
130	0.17382	0.018000	1060.58000	111.571	1048.1	111.571	1021.7
140	0.17382	0.018000	1053.58000	111.571	1041.1	111.571	1014.7
150	0.17382	0.018000	1046.58000	111.571	1034.1	111.571	1007.7
160	0.17382	0.018000	1039.58000	111.571	1027.1	111.571	1000.7
170	0.17382	0.018000	1032.58000	111.571	1020.1	111.571	993.7
180	0.17382	0.018000	1025.58000	111.571	1013.1	111.571	986.7
190	0.17382	0.018000	1018.58000	111.571	1006.1	111.571	979.7
200	0.17382	0.018000	1011.58000	111.571	999.1	111.571	972.7
210	0.17382	0.018000	1004.58000	111.571	992.1	111.571	965.7
220	0.17382	0.018000	997.58000	111.571	985.1	111.571	958.7
230	0.17382	0.018000	990.58000	111.571	978.1	111.571	951.7
240	0.17382	0.018000	983.58000	111.571	971.1	111.571	944.7
250	0.17382	0.018000	976.58000	111.571	964.1	111.571	937.7
260	0.17382	0.018000	969.58000	111.571	957.1	111.571	930.7
270	0.17382	0.018000	962.58000	111.571	950.1	111.571	923.7
280	0.17382	0.018000	955.58000	111.571	943.1	111.571	916.7
290	0.17382	0.018000	948.58000	111.571	936.1	111.571	909.7
300	0.17382	0.018000	941.58000	111.571	929.1	111.571	902.7
310	0.17382	0.018000	934.58000	111.571	922.1	111.571	895.7
320	0.17382	0.018000	927.58000	111.571	915.1	111.571	888.7
330	0.17382	0.018000	920.58000	111.571	908.1	111.571	881.7
340	0.17382	0.018000	913.58000	111.571	901.1	111.571	874.7
350	0.17382	0.018000	906.58000	111.571	894.1	111.571	867.7
360	0.17382	0.018000	899.58000	111.571	887.1	111.571	860.7
370	0.17382	0.018000	892.58000	111.571	880.1	111.571	853.7
380	0.17382	0.018000	885.58000	111.571	873.1	111.571	846.7
390	0.17382	0.018000	878.58000	111.571	866.1	111.571	839.7
400	0.17382	0.018000	871.58000	111.571	859.1	111.571	832.7
410	0.17382	0.018000	864.58000	111.571	852.1	111.571	825.7
420	0.17382	0.018000	857.58000	111.571	845.1	111.571	818.7
430	0.17382	0.018000	850.58000	111.571	838.1	111.571	811.7
440	0.17382	0.018000	843.58000	111.571	831.1	111.571	804.7

Jm 9/15/81

RELIEF VALVE DATA

VALVE INLET DIAMETER = 4.000 IN. DISCHARGE = 1.000 IN.

DISCHARGE COEFFICIENT = 0.97036

VALVE FLOW AT REACTOR PRESSURE OF 1000.0 PSIA IS 111.24 LB PER

SONIC VELOCITY AT THAT PRESSURE IS 1471.84 FT/SEC

VALVE SETPOINTS IN LIST ARE FOR AUTOMATIC PRESSURE RELIEF
(FROM REAR PAGE E.1-49)

VALVE NUMBER	OPENING PRESSURE	CLOSING PRESSURE
1	1051.0	1041.0
2	1051.0	1041.0
3	1101.0	1051.0
4	1101.0	1051.0
5	1101.0	1051.0
6	1101.0	1051.0
7	1111.0	1061.0
8	1111.0	1061.0
9	1111.0	1061.0
10	1111.0	1061.0
11	1121.0	1071.0
12	1121.0	1071.0
13	1121.0	1071.0
14	1121.0	1071.0
15	1131.0	1081.0
16	1131.0	1081.0
17	1131.0	1081.0
18	1131.0	1081.0

SONIC VELOCITY TABLE (FROM ASME STEAM TABLES, PAGE 277)

POINT	PRESSURE	VELOCITY
1	5.0	1385.0
2	10.0	1419.0
3	20.0	1455.0
4	50.0	1502.0
5	80.0	1511.0
6	100.0	1520.0
7	200.0	1550.0
8	300.0	1555.0
9	400.0	1551.0
10	500.0	1543.0
11	600.0	1531.0
12	700.0	1520.0
13	800.0	1507.0
14	1000.0	1491.0
15	1200.0	1475.0
16	1400.0	1458.0
17	1600.0	1438.0

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5/1/86

*** INITIAL CONDITIONS *** THU. 29 MAY 86. 02:00 HR ***

TOTAL SYSTEM VOLUME = 11000 FT**3
 LIQUID VOLUME WITHOUT VOIDS = 10145.1 FT**3
 LIQUID VOLUME WITH VOIDS = 10145.1 FT**3
 VAPOR VOLUME INCLUDING VOIDS = 854.9 FT**3 (TOTAL VAPOR)
 VAPOR VOLUME ABOVE ACTUAL LIQUID LEVEL = 854.9 FT**3

MASS OF LIQUID ONLY (VOIDS EXCLUDED) = 585.75 LBS
 MASS OF VAPOR ABOVE LIQUID = 17.0749 LBS
 TOTAL MASS OF VAPOR (VOIDS INCLUDED) = 17.0749 LBS
 TOTAL FLUID MASS (LIQUID+TOTAL VAPOR) = 602.82 LBS
 MASS OF THERMALLY ACTIVE STEEL STRUCTURE = 4650 KLB

(Page A3-10, Line 1020)

COLLAPSED REACTOR LEVEL = 40.0488 FT (NO VOIDS)
 ACTUAL REACTOR LEVEL = 42.8841 FT (WITH VOIDS)
 REACTOR PRESSURE = 1087.17 LB/IN**2
 REACTOR COOLANT TEMPERATURE = 551.5 FAHRENHEIT
 AVERAGE COOLANT QUALITY = 0.722 PERCENT
 AVERAGE COOLANT VOID FRACTION = 6.800 PERCENT
 CORRESPONDING AVERAGE VOLUMETRIC VOID AT FULL POWER, WITH LEVEL AT SETPOINT LN = 11.063 PERCENT

SPECIFIC VOLUME OF LIQUID = 2.18711E-02 FT**3/LB
 SPECIFIC VOLUME OF VAPOR = .41666E FT**3/LB
 SPECIFIC ENTHALPY OF LIQUID = 552.047 BTU/LB
 SPECIFIC ENTHALPY OF VAPOR = 1198.41 BTU/LB

*** ALL DATA FOR STEADY STATE WRITTEN TO DISK, CHANNEL 2 ***

*** THU. 29 MAY 86. 02:00 HR ***

28 JUL 86

full RHR pump curve (Fig. 5).

TAVSTL = 4.0

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[illegible]

172.0 549.5 0.0733 1041.3 11494.3 19.98 499.884 97.6 0.0 0.0
 175.0 549.6 0.0733 1041.7 11494.3 19.98 499.884 97.6 0.0 0.0
 178.0 549.6 0.0734 1041.7 11494.3 19.98 499.884 97.6 0.0 0.0

179.0 550.1 0.0733 1042.0 11494.3 19.98 499.884 97.6 0.0 0.0
 180.0 550.3 0.0733 1042.2 11494.3 19.98 499.884 97.6 0.0 0.0
 181.0 550.4 0.0733 1042.7 11494.3 19.98 499.884 97.6 0.0 0.0

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MODEL TIME SEC	HEAD TEMP FAHF	DECAY POWER PPAC	HEAD PRESS PSIA	HEAD FLOW FT*30	HEAD TEMP F	HEAD FLOW LBS/SEC	HEAD TEMP F	HEAD FLOW LBS/SEC	HEAD TEMP F	HEAD FLOW LBS/SEC	HEAD TEMP F	HEAD FLOW LBS/SEC	HEAD TEMP F	HEAD FLOW LBS/SEC
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177.0	549.5	0.0734	1042.0	11494.3	19.98	499.884	97.6	0.0	0.0				
178.0	549.6	0.0733	1042.0	11494.3	19.98	499.884	97.6	0.0	0.0				
179.0	550.1	0.0733	1042.0	11494.3	19.98	499.884	97.6	0.0	0.0				
180.0	550.3	0.0733	1042.2	11494.3	19.98	499.884	97.6	0.0	0.0				
181.0	550.4	0.0733	1042.7	11494.3	19.98	499.884	97.6	0.0	0.0				
182.0	550.4	0.0733	1042.2	11494.3	19.98	499.884	97.6	0.0	0.0				
183.0	550.6	0.0733	1042.7	11494.3	19.98	499.884	97.6	0.0	0.0				
184.0	550.7	0.0733	1043.0	11494.3	19.98	499.884	97.6	0.0	0.0				
185.0	550.8	0.0733	1043.4	11494.3	19.98	499.884	97.6	0.0	0.0				
186.0	550.9	0.0733	1043.5	11494.3	19.98	499.884	97.6	0.0	0.0				
187.0	551.1	0.0733	1043.5	11494.3	19.98	499.884	97.6	0.0	0.0				
188.0	551.2	0.0733	1043.6	11494.3	19.98	499.884	97.6	0.0	0.0				
189.0	551.3	0.0733	1043.1	11494.3	19.98	499.884	97.6	0.0	0.0				
190.0	551.4	0.0733	1043.2	11494.3	19.98	499.884	97.6	0.0	0.0				
191.0	551.5	0.0733	1043.3	10990.4	38.47	499.884	97.6	0.0	0.0				
192.0	551.6	0.0733	1043.3	10980.9	38.44	499.884	97.6	0.0	0.0				
193.0	551.8	0.0733	1043.4	10973.2	38.41	499.884	97.6	0.0	0.0				
194.0	551.9	0.0733	1043.5	10965.3	38.38	499.884	97.6	0.0	0.0				
195.0	551.9	0.0733	1043.0	10957.7	38.44	499.884	97.6	0.0	0.0				
196.0	552.1	0.0733	1043.1	10947.7	38.42	499.884	97.6	0.0	0.0				
197.0	552.2	0.0733	1043.2	10939.9	38.40	499.884	97.6	0.0	0.0				
198.0	552.2	0.0733	1043.2	10933.2	38.38	499.884	97.6	0.0	0.0				
199.0	552.4	0.0733	1043.3	10928.1	38.37	499.884	97.6	0.0	0.0				
200.0	552.5	0.0733	1043.8	10927.5	38.43	499.884	97.6	0.0	0.0				
201.0	552.6	0.0733	1043.9	10922.3	38.41	499.884	97.6	0.0	0.0				
202.0	552.8	0.0733	1043.0	10916.3	38.39	499.884	97.6	0.0	0.0				
203.0	552.9	0.0733	1043.1	10911.2	38.38	499.884	97.6	0.0	0.0				
204.0	552.9	0.0733	1043.6	10908.8	38.44	499.884	97.6	0.0	0.0				
205.0	553.1	0.0733	1070.7	10975.8	38.42	499.884	97.6	0.0	0.0				
206.0	553.2	0.0733	1071.8	10969.8	38.40	499.884	97.6	0.0	0.0				
207.0	553.3	0.0733	1071.6	10963.6	38.46	499.884	97.6	0.0	0.0				
208.0	553.4	0.0733	1073.4	10952.8	38.43	499.884	97.6	0.0	0.0				
209.0	553.5	0.0733	1074.5	10947.9	38.43	499.884	97.6	0.0	0.0				
210.0	553.6	0.0733	1075.6	10942.1	38.41	499.884	97.6	0.0	0.0				
211.0	553.7	0.0733	1076.1	10931.1	38.43	499.884	97.6	0.0	0.0				
212.0	553.8	0.0733	1077.1	10925.4	38.43	499.884	97.6	0.0	0.0				
213.0	553.9	0.0733	1077.7	11004.4	38.31	499.884	97.6	0.0	0.0				
214.0	554.0	0.0733	1078.8	10998.9	38.29	499.884	97.6	0.0	0.0				
215.0	554.1	0.0733	1079.9	10993.5	38.27	499.884	97.6	0.0	0.0				
216.0	554.3	0.0733	1081.0	10987.7	38.26	499.884	97.6	0.0	0.0				
217.0	554.4	0.0733	1082.1	10981.4	38.24	499.884	97.6	0.0	0.0				
218.0	554.4	0.0733	1082.6	10979.8	38.26	499.884	97.6	0.0	0.0				
219.0	554.6	0.0733	1083.7	10973.5	38.23	499.884	97.6	0.0	0.0				
220.0	554.6	0.0733	1084.1	11012.2	38.23	499.884	97.6	0.0	0.0				
221.0	554.8	0.0733	1085.0	11006.2	38.21	499.884	97.6	0.0	0.0				
222.0	554.9	0.0733	1085.4	10998.9	38.19	499.884	97.6	0.0	0.0				
223.0	555.0	0.0733	1085.5	10994.4	38.18	499.884	97.6	0.0	0.0				
224.0	555.1	0.0733	1085.6	10988.2	38.16	499.884	97.6	0.0	0.0				
225.0	555.2	0.0733	1089.2	11006.2	38.21	499.884	97.6	0.0	0.0				
226.0	555.3	0.0733	1090.1	11000.4	38.20	499.884	97.6	0.0	0.0				
227.0	555.4	0.0733	1090.8	11013.9	38.22	499.884	97.6	0.0	0.0				
228.0	555.5	0.0733	1090.1	11027.0	38.24	499.884	97.6	0.0	0.0				
229.0	555.6	0.0733	1092.1	11031.2	38.27	499.884	97.6	0.0	0.0				
230.0	555.8	0.0733	1092.1	11031.2	38.27	499.884	97.6	0.0	0.0				

void fraction in pct

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void fraction in pct

470.0 554.1 0.0155 1079.7 10144.4 103.91
 471.0 554.2 0.0155 1079.8 10144.5 103.91

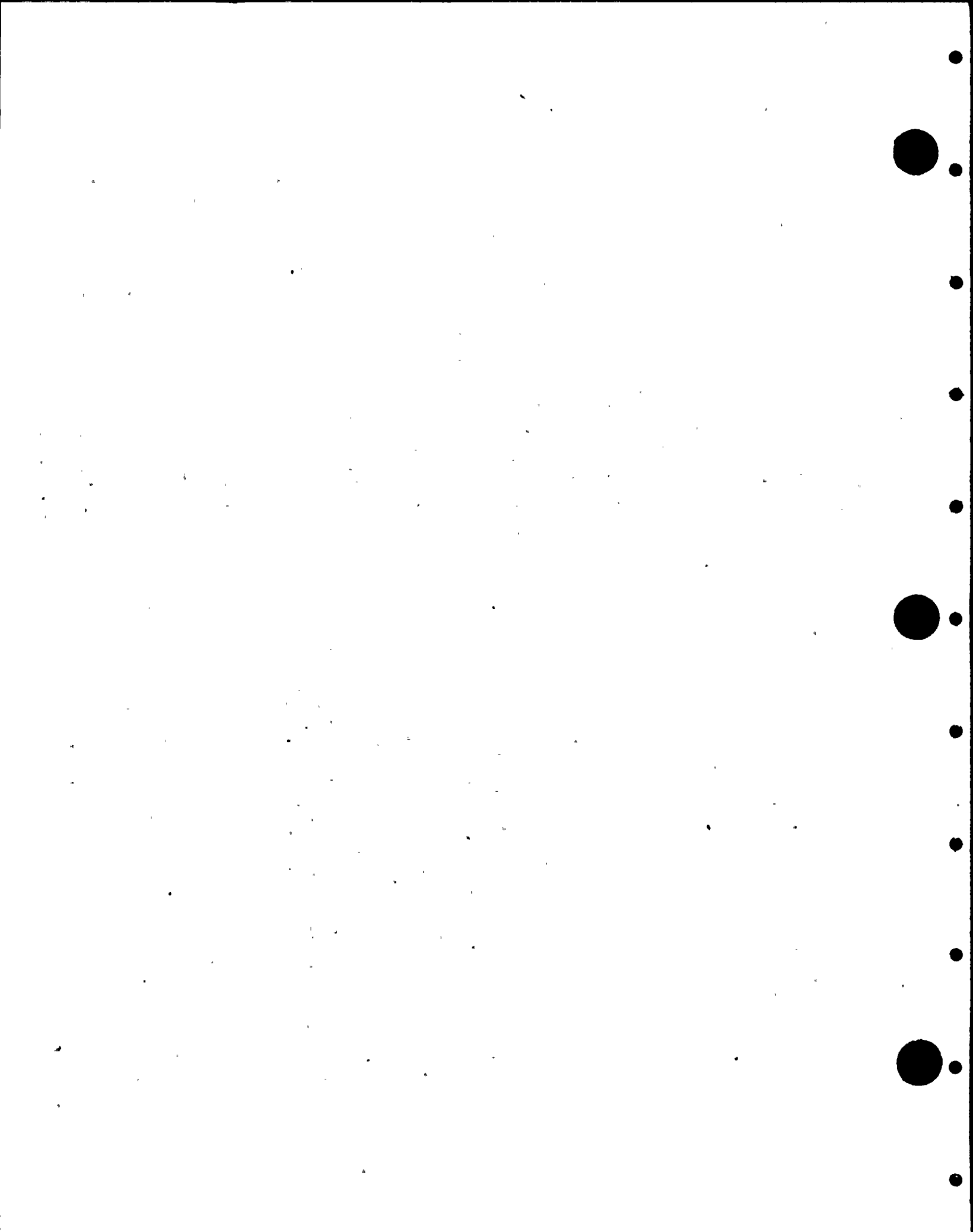
472.0 554.3 0.0155 1079.9 10144.6 103.91
 473.0 554.4 0.0155 1080.0 10144.7 103.91
 474.0 554.5 0.0155 1080.1 10144.8 103.91

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MODEL TIME SEC	SEAL TEMP FAH	SEAL POWER FAH	SEAL PRESS PSIA	SEAL FLOW GPM	SEAL LEVEL FT	SEAL FLOW KLB	SEAL TEMP FAH	SEAL FLOW LB/SEC	SEAL FLOW LB/SEC	SEAL FLOW LB/SEC
472.0	554.3	0.0155	1081.0	10145.0	103.96	450.1590	102.9	0.0	0.0	0.4
473.0	554.4	0.0155	1081.1	10145.1	103.96	450.1590	102.9	0.0	0.0	0.4
474.0	554.5	0.0155	1081.2	10145.2	103.96	450.1590	102.9	0.0	0.0	0.4
475.0	554.6	0.0155	1081.3	10145.3	103.96	450.1590	102.9	0.0	0.0	0.4
476.0	554.7	0.0155	1081.4	10145.4	103.96	450.1590	102.9	0.0	0.0	0.4
477.0	554.8	0.0155	1081.5	10145.5	103.96	450.1590	102.9	0.0	0.0	0.4
478.0	554.9	0.0155	1081.6	10145.6	103.96	450.1590	102.9	0.0	0.0	0.4
479.0	555.0	0.0155	1081.7	10145.7	103.96	450.1590	102.9	0.0	0.0	0.4
480.0	555.1	0.0155	1081.8	10145.8	103.96	450.1590	102.9	0.0	0.0	0.4
481.0	555.2	0.0155	1081.9	10145.9	103.96	450.1590	102.9	0.0	0.0	0.4
482.0	555.3	0.0155	1082.0	10146.0	103.96	450.1590	102.9	0.0	0.0	0.4
483.0	555.4	0.0155	1082.1	10146.1	103.96	450.1590	102.9	0.0	0.0	0.4
484.0	555.5	0.0155	1082.2	10146.2	103.96	450.1590	102.9	0.0	0.0	0.4
485.0	555.6	0.0155	1082.3	10146.3	103.96	450.1590	102.9	0.0	0.0	0.4
486.0	555.7	0.0155	1082.4	10146.4	103.96	450.1590	102.9	0.0	0.0	0.4
487.0	555.8	0.0155	1082.5	10146.5	103.96	450.1590	102.9	0.0	0.0	0.4
488.0	555.9	0.0155	1082.6	10146.6	103.96	450.1590	102.9	0.0	0.0	0.4
489.0	556.0	0.0155	1082.7	10146.7	103.96	450.1590	102.9	0.0	0.0	0.4
490.0	556.1	0.0155	1082.8	10146.8	103.96	450.1590	102.9	0.0	0.0	0.4
491.0	556.2	0.0155	1082.9	10146.9	103.96	450.1590	102.9	0.0	0.0	0.4
492.0	556.3	0.0155	1083.0	10147.0	103.96	450.1590	102.9	0.0	0.0	0.4
493.0	556.4	0.0155	1083.1	10147.1	103.96	450.1590	102.9	0.0	0.0	0.4
494.0	556.5	0.0155	1083.2	10147.2	103.96	450.1590	102.9	0.0	0.0	0.4
495.0	556.6	0.0155	1083.3	10147.3	103.96	450.1590	102.9	0.0	0.0	0.4
496.0	556.7	0.0155	1083.4	10147.4	103.96	450.1590	102.9	0.0	0.0	0.4
497.0	556.8	0.0155	1083.5	10147.5	103.96	450.1590	102.9	0.0	0.0	0.4
498.0	556.9	0.0155	1083.6	10147.6	103.96	450.1590	102.9	0.0	0.0	0.4
499.0	557.0	0.0155	1083.7	10147.7	103.96	450.1590	102.9	0.0	0.0	0.4
500.0	557.1	0.0155	1083.8	10147.8	103.96	450.1590	102.9	0.0	0.0	0.4
501.0	557.2	0.0155	1083.9	10147.9	103.96	450.1590	102.9	0.0	0.0	0.4
502.0	557.3	0.0155	1084.0	10148.0	103.96	450.1590	102.9	0.0	0.0	0.4
503.0	557.4	0.0155	1084.1	10148.1	103.96	450.1590	102.9	0.0	0.0	0.4
504.0	557.5	0.0155	1084.2	10148.2	103.96	450.1590	102.9	0.0	0.0	0.4
505.0	557.6	0.0155	1084.3	10148.3	103.96	450.1590	102.9	0.0	0.0	0.4
506.0	557.7	0.0155	1084.4	10148.4	103.96	450.1590	102.9	0.0	0.0	0.4
507.0	557.8	0.0155	1084.5	10148.5	103.96	450.1590	102.9	0.0	0.0	0.4
508.0	557.9	0.0155	1084.6	10148.6	103.96	450.1590	102.9	0.0	0.0	0.4
509.0	558.0	0.0155	1084.7	10148.7	103.96	450.1590	102.9	0.0	0.0	0.4
510.0	558.1	0.0155	1084.8	10148.8	103.96	450.1590	102.9	0.0	0.0	0.4
511.0	558.2	0.0155	1084.9	10148.9	103.96	450.1590	102.9	0.0	0.0	0.4
512.0	558.3	0.0155	1085.0	10149.0	103.96	450.1590	102.9	0.0	0.0	0.4
513.0	558.4	0.0155	1085.1	10149.1	103.96	450.1590	102.9	0.0	0.0	0.4
514.0	558.5	0.0155	1085.2	10149.2	103.96	450.1590	102.9	0.0	0.0	0.4
515.0	558.6	0.0155	1085.3	10149.3	103.96	450.1590	102.9	0.0	0.0	0.4
516.0	558.7	0.0155	1085.4	10149.4	103.96	450.1590	102.9	0.0	0.0	0.4
517.0	558.8	0.0155	1085.5	10149.5	103.96	450.1590	102.9	0.0	0.0	0.4
518.0	558.9	0.0155	1085.6	10149.6	103.96	450.1590	102.9	0.0	0.0	0.4
519.0	559.0	0.0155	1085.7	10149.7	103.96	450.1590	102.9	0.0	0.0	0.4
520.0	559.1	0.0155	1085.8	10149.8	103.96	450.1590	102.9	0.0	0.0	0.4
521.0	559.2	0.0155	1085.9	10149.9	103.96	450.1590	102.9	0.0	0.0	0.4
522.0	559.3	0.0155	1086.0	10150.0	103.96	450.1590	102.9	0.0	0.0	0.4
523.0	559.4	0.0155	1086.1	10150.1	103.96	450.1590	102.9	0.0	0.0	0.4
524.0	559.5	0.0155	1086.2	10150.2	103.96	450.1590	102.9	0.0	0.0	0.4
525.0	559.6	0.0155	1086.3	10150.3	103.96	450.1590	102.9	0.0	0.0	0.4

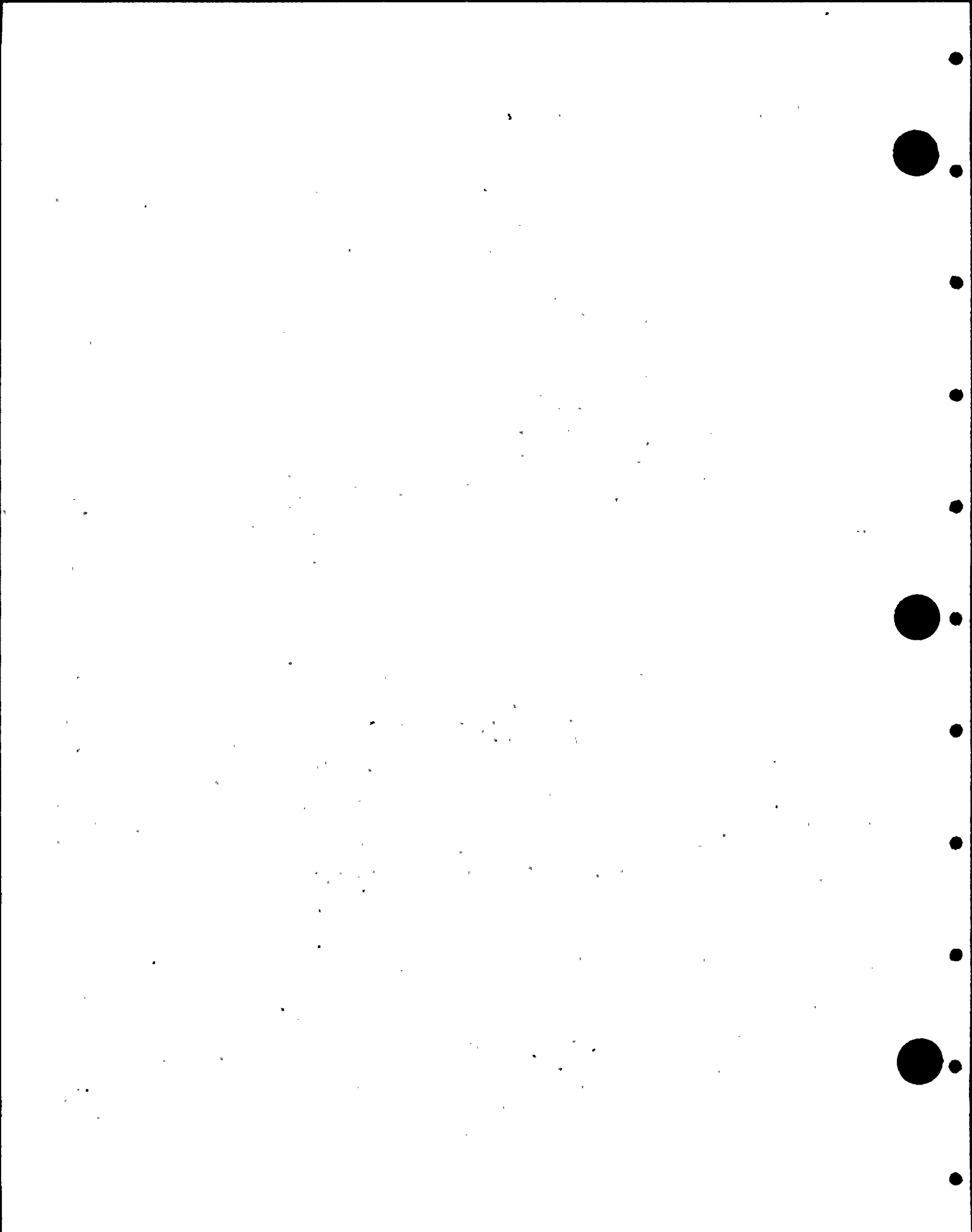
void fraction in pct



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[illegible]



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[illegible]

Page A4-18^{1/12 3/13/50}

void fraction in percent

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MODE	TIME	SEC	705.0	706.0	710.0	711.0	712.0	713.0	714.0	715.0	716.0	717.0	718.0	719.0	720.0	721.0	722.0	723.0	724.0	725.0	726.0	727.0	728.0	729.0	730.0	731.0	732.0	733.0	734.0	735.0	736.0	737.0	738.0	739.0	740.0	741.0	742.0	743.0	744.0	745.0	746.0	747.0	748.0	749.0	750.0	751.0	752.0	753.0	754.0	755.0	756.0	757.0	758.0	759.0	760.0	761.0	762.0	763.0	764.0	765.0	766.0	767.0	768.0	769.0	770.0	771.0	772.0	773.0	774.0	775.0	776.0	777.0	778.0	779.0	780.0	781.0	782.0	783.0	784.0	785.0	786.0	787.0	788.0	789.0	790.0	791.0	792.0	793.0	794.0	795.0	796.0	797.0	798.0	799.0	800.0	801.0	802.0	803.0	804.0	805.0	806.0	807.0	808.0	809.0	810.0	811.0	812.0	813.0	814.0	815.0	816.0	817.0	818.0	819.0	820.0	821.0	822.0	823.0	824.0	825.0	826.0	827.0	828.0	829.0	830.0	831.0	832.0	833.0	834.0	835.0	836.0	837.0	838.0	839.0	840.0	841.0	842.0	843.0	844.0	845.0	846.0	847.0	848.0	849.0	850.0	851.0	852.0	853.0	854.0	855.0	856.0	857.0	858.0	859.0	860.0	861.0	862.0	863.0	864.0	865.0	866.0	867.0	868.0	869.0	870.0	871.0	872.0	873.0	874.0	875.0	876.0	877.0	878.0	879.0	880.0	881.0	882.0	883.0	884.0	885.0	886.0	887.0	888.0	889.0	890.0	891.0	892.0	893.0	894.0	895.0	896.0	897.0	898.0	899.0	900.0	901.0	902.0	903.0	904.0	905.0	906.0	907.0	908.0	909.0	910.0	911.0	912.0	913.0	914.0	915.0	916.0	917.0	918.0	919.0	920.0	921.0	922.0	923.0	924.0	925.0	926.0	927.0	928.0	929.0	930.0	931.0	932.0	933.0	934.0	935.0	936.0	937.0	938.0	939.0	940.0	941.0	942.0	943.0	944.0	945.0	946.0	947.0	948.0	949.0	950.0	951.0	952.0	953.0	954.0	955.0	956.0	957.0	958.0	959.0	960.0	961.0	962.0	963.0	964.0	965.0	966.0	967.0	968.0	969.0	970.0	971.0	972.0	973.0	974.0	975.0	976.0	977.0	978.0	979.0	980.0	981.0	982.0	983.0	984.0	985.0	986.0	987.0	988.0	989.0	990.0	991.0	992.0	993.0	994.0	995.0	996.0	997.0	998.0	999.0	1000.0																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																																	
472.6	472.4	472.9	472.5	472.7	472.1	471.6	471.1	470.8	469.9	469.4	469.9	468.8	467.8	467.3	466.9	466.2	465.7	465.1	464.6	464.1	463.6	463.1	462.6	462.0	461.5	460.9	460.4	459.9	459.4	458.9	458.4	457.9	457.4	456.9	456.4	455.9	455.4	454.9	454.4	453.9	453.4	452.9	452.4	451.9	451.4	450.9	450.4	449.9	449.4	448.9	448.4	447.9	447.4	446.9	446.4	445.9	445.4	444.9	444.4	443.9	443.4	442.9	442.4	441.9	441.4	440.9	440.4	439.9	439.4	438.9	438.4	437.9	437.4	436.9	436.4	435.9	435.4	434.9	434.4	433.9	433.4	432.9	432.4	431.9	431.4	430.9	430.4	429.9	429.4	428.9	428.4	427.9	427.4	426.9	426.4	425.9	425.4	424.9	424.4	423.9	423.4	422.9	422.4	421.9	421.4	420.9	420.4	419.9	419.4	418.9	418.4	417.9	417.4	416.9	416.4	415.9	415.4	414.9	414.4	413.9	413.4	412.9	412.4	411.9	411.4	410.9	410.4	409.9	409.4	408.9	408.4	407.9	407.4	406.9	406.4	405.9	405.4	404.9	404.4	403.9	403.4	402.9	402.4	401.9	401.4	400.9	400.4	399.9	399.4	398.9	398.4	397.9	397.4	396.9	396.4	395.9	395.4	394.9	394.4	393.9	393.4	392.9	392.4	391.9	391.4	390.9	390.4	389.9	389.4	388.9	388.4	387.9	387.4	386.9	386.4	385.9	385.4	384.9	384.4	383.9	383.4	382.9	382.4	381.9	381.4	380.9	380.4	379.9	379.4	378.9	378.4	377.9	377.4	376.9	376.4	375.9	375.4	374.9	374.4	373.9	373.4	372.9	372.4	371.9	371.4	370.9	370.4	369.9	369.4	368.9	368.4	367.9	367.4	366.9	366.4	365.9	365.4	364.9	364.4	363.9	363.4	362.9	362.4	361.9	361.4	360.9	360.4	359.9	359.4	358.9	358.4	357.9	357.4	356.9	356.4	355.9	355.4	354.9	354.4	353.9	353.4	352.9	352.4	351.9	351.4	350.9	350.4	349.9	349.4	348.9	348.4	347.9	347.4	346.9	346.4	345.9	345.4	344.9	344.4	343.9	343.4	342.9	342.4	341.9	341.4	340.9	340.4	339.9	339.4	338.9	338.4	337.9	337.4	336.9	336.4	335.9	335.4	334.9	334.4	333.9	333.4	332.9	332.4	331.9	331.4	330.9	330.4	329.9	329.4	328.9	328.4	327.9	327.4	326.9	326.4	325.9	325.4	324.9	324.4	323.9	323.4	322.9	322.4	321.9	321.4	320.9	320.4	319.9	319.4	318.9	318.4	317.9	317.4	316.9	316.4	315.9	315.4	314.9	314.4	313.9	313.4	312.9	312.4	311.9	311.4	310.9	310.4	309.9	309.4	308.9	308.4	307.9	307.4	306.9	306.4	305.9	305.4	304.9	304.4	303.9	303.4	302.9	302.4	301.9	301.4	300.9	300.4	299.9	299.4	298.9	298.4	297.9	297.4	296.9	296.4	295.9	295.4	294.9	294.4	293.9	293.4	292.9	292.4	291.9	291.4	290.9	290.4	289.9	289.4	288.9	288.4	287.9	287.4	286.9	286.4	285.9	285.4	284.9	284.4	283.9	283.4	282.9	282.4	281.9	281.4	280.9	280.4	279.9	279.4	278.9	278.4	277.9	277.4	276.9	276.4	275.9	275.4	274.9	274.4	273.9	273.4	272.9	272.4	271.9	271.4	270.9	270.4	269.9	269.4	268.9	268.4	267.9	267.4	266.9	266.4	265.9	265.4	264.9	264.4	263.9	263.4	262.9	262.4	261.9	261.4	260.9	260.4	259.9	259.4	258.9	258.4	257.9	257.4	256.9	256.4	255.9	255.4	254.9	254.4	253.9	253.4	252.9	252.4	251.9	251.4	250.9	250.4	249.9	249.4	248.9	248.4	247.9	247.4	246.9	246.4	245.9	245.4	244.9	244.4	243.9	243.4	242.9	242.4	241.9	241.4	240.9	240.4	239.9	239.4	238.9	238.4	237.9	237.4	236.9	236.4	235.9	235.4	234.9	234.4	233.9	233.4	232.9	232.4	231.9	231.4	230.9	230.4	229.9	229.4	228.9	228.4	227.9	227.4	226.9	226.4	225.9	225.4	224.9	224.4	223.9	223.4	222.9	222.4	221.9	221.4	220.9	220.4	219.9	219.4	218.9	218.4	217.9	217.4	216.9	216.4	215.9	215.4	214.9	214.4	213.9	213.4	212.9	212.4	211.9	211.4	210.9	210.4	209.9	209.4	208.9	208.4	207.9	207.4	206.9	206.4	205.9	205.4	204.9	204.4	203.9	203.4	202.9	202.4	201.9	201.4	200.9	200.4	199.9	199.4	198.9	198.4	197.9	197.4	196.9	196.4	195.9	195.4	194.9	194.4	193.9	193.4	192.9	192.4	191.9	191.4	190.9	190.4	189.9	189.4	188.9	188.4	187.9	187.4	186.9	186.4	185.9	185.4	184.9	184.4	183.9	183.4	182.9	182.4	181.9	181.4	180.9	180.4	179.9	179.4	178.9	178.4	177.9	177.4	176.9	176.4	175.9	175.4	174.9	174.4	173.9	173.4	172.9	172.4	171.9	171.4	170.9	170.4	169.9	169.4	168.9	168.4	167.9	167.4	166.9	166.4	165.9	165.4	164.9	164.4	163.9	163.4	162.9	162.4	161.9	161.4	160.9	160.4	159.9	159.4	158.9	158.4	157.9	157.4	156.9	156.4	155.9	155.4	154.9	154.4	153.9	153.4	152.9	152.4	151.9	151.4	150.9	150.4	149.9	149.4	148.9	148.4	147.9	147.4	146.9	146.4	145.9	145.4	144.9	144.4	143.9	143.4	142.9	142.4	141.9	141.4	140.9	140.4	139.9	139.4	138.9	138.4	137.9	137.4	136.9	136.4	135.9	135.4	134.9	134.4	133.9	133.4	132.9	132.4	131.9	131.4	130.9	130.4	129.9	129.4	128.9	128.4	127.9	127.4	126.9	126.4	125.9	125.4	124.9	124.4	123.9	123.4	122.9	122.4	121.9	121.4	120.9	120.4	119.9	119.4	118.9	118.4	117.9	117.4	116.9	116.4	115.9	115.4	114.9	114.4	113.9	113.4	112.9	112.4	111.9	111.4	110.9	110.4	109.9	109.4	108.9	108.4	107.9	107.4	106.9	106.4	105.9	105.4	104.9	104.4	103.9	103.4	102.9	102.4	101.9	101.4	100.9	100.4	99.9	99.4	98.9	98.4	97.9	97.4	96.9	96.4	95.9	95.4	94.9	94.4	93.9	93.4	92.9	92.4	91.9	91.4	90.9	90.4	89.9	89.4	88.9	88.4	87.9	87.4	86.9	86.4	85.9	85.4	84.9	84.4	83.9	83.4	82.9	82.4	81.9	81.4	80.9	80.4	79.9	79.4	78.9	78.4	77.9	77.4	76.9	76.4	75.9	75.4	74.9	74.4	73.9	73.4	72.9	72.4	71.9	71.4	70.9	70.4	69.9	69.4	68.9	68.4	67.9	67.4	66.9	66.4	65.9	65.4	64.9	64.4	63.9	63.4	62.9	62.4	61.9	61.4	60.9	60.4	59.9	59.4	58.9	58.4	57.9	57.4	56.9	56.4	55.9	55.4	54.9	54.4	53.9	53.4	52.9	52.4	51.9	51.4	50.9	50.4	49.9	49.4	48.9	48.4	47.9	47.4	46.9	46.4	45.9	45.4	44.9	44.4	43.9	43.4	42.9	42.4	41.9	41.4	40.9	40.4	39.9	39.4	38.9	38.4	37.9	37.4	36.9	36.4	35.9	35.4	34.9	34.4	33.9	33.4	32.9	32.4	31.9	31.4	30.9	30.4	29.9	29.4	28.9	28.4	27.9	27.4	26.9	26.4	25.9	25.4	24.9	24.4	23.9	23.4	22.9	22.4	21.9	21.4	20.9	20.4	19.9	19.4	18.9	18.4	17.9	17.4	16.9	16.4	15.9	15.4	14.9	14.4	13.9	13.4	12.9	12.4	11.9	11.4	10.9	10.4	9.9	9.4	8.9	8.4	7.9	7.4	6.9	6.4	5.9	5.4	4.9	4.4	3.9	3.4	2.9	2.4	1.9	1.4	0.9	0.4	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

765.0 445.8 0.02177 438.5 808.0 30.43 311.653 125.9 433.3 0.0 13.1
 766.0 445.8 0.02177 438.5 808.0 30.43 311.653 125.9 433.3 0.0 13.1
 767.0 445.8 0.02177 438.5 808.0 30.43 311.653 125.9 433.3 0.0 13.1

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METE- TIME SEC	FEED TEMP F/400	SECORV FLOW F/400	SEED TEMP F/400	SEED FLOW F/400	SEED TEMP F/400	SEED FLOW F/400	SEED TEMP F/400	SEED FLOW F/400	SEED TEMP F/400	SEED FLOW F/400	SEED TEMP F/400	SEED FLOW F/400
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767.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
768.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
769.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
770.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
771.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
772.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
773.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
774.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
775.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
776.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
777.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
778.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
779.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
780.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
781.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
782.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
783.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
784.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
785.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
786.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
787.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
788.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
789.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1
790.0	445.8	0.02177	438.5	808.0	30.43	311.653	125.9	433.3	0.0	13.1

Void fraction in pct

*** TOP OF FUEL UNCOVERED AT 791 SEC. LEV= 10.4392 FT

791.0	432.4	0.02356	359.7	8353.9	30.43	311.653	125.9	433.3	0.0	13.6
792.0	433.9	0.02356	358.0	8352.9	30.43	311.653	125.9	433.3	0.0	13.9
793.0	433.5	0.02356	356.4	8352.4	30.43	311.653	125.9	433.3	0.0	13.0
794.0	433.1	0.02356	354.7	8354.4	30.43	311.653	125.9	433.3	0.0	13.2
795.0	432.7	0.02356	353.3	8350.6	30.43	311.653	125.9	433.3	0.0	13.0
796.0	432.3	0.02356	351.7	8352.0	30.43	311.653	125.9	433.3	0.0	13.1
797.0	431.8	0.02356	350.1	8352.4	30.43	311.653	125.9	433.3	0.0	13.2
798.0	431.4	0.02356	348.7	8353.0	30.43	311.653	125.9	433.3	0.0	13.1
799.0	431.0	0.02356	347.0	8359.7	30.43	311.653	125.9	433.3	0.0	13.1
800.0	430.6	0.02356	345.7	8370.2	30.43	311.653	125.9	433.3	0.0	13.1
801.0	430.2	0.02356	344.1	8377.2	30.43	311.653	125.9	433.3	0.0	13.2
802.0	429.8	0.02356	342.5	8385.2	30.43	311.653	125.9	433.3	0.0	13.4
803.0	429.4	0.02356	341.1	8387.7	30.43	311.653	125.9	433.3	0.0	13.5
804.0	428.9	0.02356	339.5	8387.4	30.43	311.653	125.9	433.3	0.0	13.5
805.0	428.6	0.02356	338.2	8381.5	30.43	311.653	125.9	433.3	0.0	13.7
806.0	428.1	0.02356	336.6	8382.8	30.43	311.653	125.9	433.3	0.0	13.6
807.0	427.8	0.02356	335.2	8388.4	30.43	311.653	125.9	433.3	0.0	13.6
808.0	427.3	0.02356	333.8	8383.2	30.43	311.653	125.9	433.3	0.0	13.5
809.0	427.0	0.02356	332.4	8380.9	30.43	311.653	125.9	433.3	0.0	13.4
810.0	426.5	0.02356	330.9	8385.3	30.43	311.653	125.9	433.3	0.0	13.6
811.0	426.2	0.02356	329.5	8393.9	30.43	311.653	125.9	433.3	0.0	13.6
812.0	425.7	0.02356	328.0	8390.0	30.43	311.653	125.9	433.3	0.0	13.8
813.0	425.3	0.02356	326.7	8389.9	30.43	311.653	125.9	433.3	0.0	13.7
814.0	425.0	0.02356	325.4	8370.6	30.43	311.653	125.9	433.3	0.0	13.7
815.0	424.6	0.02356	324.0	8372.8	30.43	311.653	125.9	433.3	0.0	13.6
816.0	424.2	0.02356	322.5	8377.0	30.43	311.653	125.9	433.3	0.0	13.9
817.0	423.8	0.02356	321.1	8378.0	30.43	311.653	125.9	433.3	0.0	13.8
818.0	423.4	0.02356	319.9	8379.0	30.43	311.653	125.9	433.3	0.0	13.8
819.0	423.0	0.02356	318.5	8379.0	30.43	311.653	125.9	433.3	0.0	13.8

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968	967	966	965	964	963	962	961	960	959	958	957	956	955	954	953	952	951	950	949	948	947	946	945	944	943	942	941	940	939	938	937	936	935	934	933	932	931	930	929	928	927	926	925	924	923	922	921	920	919	918	917	916	915	914	913	912	911	910	909	908	907	906	905	904	903	902	901	900	899	898	897	896	895	894	893	892	891	890	889	888	887	886	885	884	883	882	881	880	879	878	877	876	875	874	873	872	871	870	869	868	867	866	865	864	863	862	861	860	859	858	857	856	855	854	853	852	851	850	849	848	847	846	845	844	843	842	841	840	839	838	837	836	835	834	833	832	831	830	829	828	827	826	825	824	823	822	821	820	819	818	817	816	815	814	813	812	811	810	809	808	807	806	805	804	803	802	801	800	799	798	797	796	795	794	793	792	791	790	789	788	787	786	785	784	783	782	781	780	779	778	777	776	775	774	773	772	771	770	769	768	767	766	765	764	763	762	761	760	759	758	757	756	755	754	753	752	751	750	749	748	747	746	745	744	743	742	741	740	739	738	737	736	735	734	733	732	731	730	729	728	727	726	725	724	723	722	721	720	719	718	717	716	715	714	713	712	711	710	709	708	707	706	705	704	703	702	701	700	699	698	697	696	695	694	693	692	691	690	689	688	687	686	685	684	683	682	681	680	679	678	677	676	675	674	673	672	671	670	669	668	667	666	665	664	663	662	661	660	659	658	657	656	655	654	653	652	651	650	649	648	647	646	645	644	643	642	641	640	639	638	637	636	635	634	633	632	631	630	629	628	627	626	625	624	623	622	621	620	619	618	617	616	615	614	613	612	611	610	609	608	607	606	605	604	603	602	601	600	599	598	597	596	595	594	593	592	591	590	589	588	587	586	585	584	583	582	581	580	579	578	577	576	575	574	573	572	571	570	569	568	567	566	565	564	563	562	561	560	559	558	557	556	555	554	553	552	551	550	549	548	547	546	545	544	543	542	541	540	539	538	537	536	535	534	533	532	531	530	529	528	527	526	525	524	523	522	521	520	519	518	517	516	515	514	513	512	511	510	509	508	507	506	505	504	503	502	501	500	499	498	497	496	495	494	493	492	491	490	489	488	487	486	485	484	483	482	481	480	479	478	477	476	475	474	473	472	471	470	469	468	467	466	465	464	463	462	461	460	459	458	457	456	455	454	453	452	451	450	449	448	447	446	445	444	443	442	441	440	439	438	437	436	435	434	433	432	431	430	429	428	427	426	425	424	423	422	421	420	419	418	417	416	415	414	413	412	411	410	409	408	407	406	405	404	403	402	401	400	399	398	397	396	395	394	393	392	391	390	389	388	387	386	385	384	383	382	381	380	379	378	377	376	375	374	373	372	371	370	369	368	367	366	365	364	363	362	361	360	359	358	357	356	355	354	353	352	351	350	349	348	347	346	345	344	343	342	341	340	339	338	337	336	335	334	333	332	331	330	329	328	327	326	325	324	323	322	321	320	319	318	317	316	315	314	313	312	311	310	309	308	307	306	305	304	303	302	301	300	299	298	297	296	295	294	293	292	291	290	289	288	287	286	285	284	283	282	281	280	279	278	277	276	275	274	273	272	271	270	269	268	267	266	265	264	263	262	261	260	259	258	257	256	255	254	253	252	251	250	249	248	247	246	245	244	243	242	241	240	239	238	237	236	235	234	233	232	231	230	229	228	227	226	225	224	223	222	221	220	219	218	217	216	215	214	213	212	211	210	209	208	207	206	205	204	203	202	201	200	199	198	197	196	195	194	193	192	191	190	189	188	187	186	185	184	183	182	181	180	179	178	177	176	175	174	173	172	171	170	169	168	167	166	165	164	163	162	161	160	159	158	157	156	155	154	153	152	151	150	149	148	147	146	145	144	143	142	141	140	139	138	137	136	135	134	133	132	131	130	129	128	127	126	125	124	123	122	121	120	119	118	117	116	115	114	113	112	111	110	109	108	107	106	105	104	103	102	101	100	99	98	97	96	95	94	93	92	91	90	89	88	87	86	85	84	83	82	81	80	79	78	77	76	75	74	73	72	71	70	69	68	67	66	65	64	63	62	61	60	59	58	57	56	55	54	53	52	51	50	49	48	47	46	45	44	43	42	41	40	39	38	37	36	35	34	33	32	31	30	29	28	27	26	25	24	23	22	21	20	19	18	17	16	15	14	13	12	11	10	9	8	7	6	5	4	3	2	1	0
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void fraction in ext.

100-443886-100

968.0	354.9	0.02235	143.5	3737.0	70.32	340.3200	143.5	177.1	25.8	373.5
969.0	354.4	0.02235	143.6	3740.2	70.33	340.3400	143.6	178.2	25.8	375.1
970.0	354.0	0.02235	144.3	3745.1	70.37	340.3600	144.3	179.2	25.7	377.5
971.0	353.5	0.02235	144.0	3750.0	70.36	341.3400	144.0	178.2	25.6	380.1
972.0	353.1	0.02234	140.2	3744.0	70.35	340.3400	140.2	175.5	25.2	383.5
973.0	352.7	0.02234	135.4	3736.0	70.33	340.3400	135.4	172.4	25.0	385.0
974.0	352.5	0.02234	138.6	3740.1	70.33	340.3400	138.7	171.4	25.0	387.5
975.0	351.5	0.02234	137.0	3735.3	70.33	340.3400	137.0	170.5	25.0	389.9
976.0	351.4	0.02234	137.1	3734.0	70.32	340.3400	137.1	169.6	25.0	392.1
977.0	351.0	0.02234	136.5	3731.7	70.32	340.3400	136.5	168.7	25.4	394.0
978.0	350.5	0.02234	135.6	3729.5	70.31	340.3400	135.6	167.7	25.4	396.2
979.0	350.1	0.02234	134.9	3727.7	70.30	340.3400	134.9	166.8	25.4	398.2
980.0	349.7	0.02234	134.1	3725.1	70.30	340.3400	134.1	165.0	25.1	400.1
981.0	349.3	0.02234	133.5	3722.6	70.30	340.3400	133.5	163.4	25.4	402.2
982.0	348.8	0.02234	132.6	3720.1	70.30	340.3400	132.6	162.4	25.3	404.1
983.0	348.4	0.02234	131.4	3717.0	70.30	340.3400	131.4	160.5	25.0	406.2
984.0	348.0	0.02234	131.1	3715.3	70.30	340.3400	131.1	159.4	25.3	408.1
985.0	347.5	0.02234	130.4	3713.1	70.30	340.3400	130.4	158.4	25.3	410.1
986.0	347.1	0.02234	129.7	3710.4	70.30	340.3400	129.7	157.4	25.3	411.0
987.0	346.7	0.02234	129.0	3708.0	70.30	340.3400	129.0	156.5	25.4	413.9
988.0	346.5	0.02234	128.0	3705.0	70.30	340.3400	128.0	155.0	25.9	415.4
989.0	346.3	0.02234	127.7	3703.4	70.30	340.3400	127.7	154.0	25.0	417.7
990.0	346.0	0.02234	126.5	3701.0	70.30	340.3400	126.5	153.7	25.9	419.7
991.0	345.4	0.02234	125.7	3698.1	70.30	340.3400	125.7	152.7	25.0	421.7
992.0	345.1	0.02234	124.8	3695.0	70.30	340.3400	124.8	151.7	25.0	423.7

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MODEL	READ	DELAY	SEAC	READ	SEAC	READ	SEAC	READ	SEAC	READ	SEAC	READ	SEAC
TIME	TEMP	POWER	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP
SEC	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP	TEMP

1180.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1181.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1182.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1183.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1184.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1185.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1186.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1187.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1188.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1189.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1190.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1191.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1192.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1
1193.0	1289.1	0.0211	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1	1289.1

*** END OF CHOKED FLOW REGIME FOR VALVES ***

*** RVP. POINT 54.6814 29.9125 ***

$P_{\text{REACTOR}} = 54.68 \text{ psia}$

$P_{\text{CONTAINMENT}} = 29.91 \text{ psia} = 15.21 \text{ psig}$

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17 JUL 86

Dr 8/15/86

APPENDIX 5

Independent Calculation G-KK-6-140
by General Electric Company

17 JUL 86

GENERAL  ELECTRIC

Don 8/15/86

NUCLEAR ENERGY BUSINESS OPERATIONS
GENERAL ELECTRIC COMPANY • 175 CURTNER AVENUE • SAN JOSE, CALIFORNIA 95125

G-KK-6-140
July 17, 1986

Mr. L.T. Harrold, MD 994E
Assistant Director, Generation Engineering
Washington Public Power Supply System
P.O. Box 968
Richland, Washington 99352

Subject: SUPPLY SYSTEM NUCLEAR PLANT NO. 2
FIRE PROTECTION ANALYSIS FOR HANFORD 2

Reference: WRO-C0875-GE-86-P2-18

Attention: G.L. Gelhaus, MD 981A

Attached is the subject report as requested by the referenced WRO. This analysis assumed that six relief valves are manually opened and only one LPCI pump (Loop B) is available. The analysis clearly demonstrates that adequate shutdown capability exists to meet the performance requirement of no fuel cladding damage as specified in 10CFR50 Appendix R for the event analyzed.

If you have any questions or require additional information, please contact G.L. Hayes.

Very truly yours,

J. Armenta

J. Armenta, Manager
Customer Services - Western Region

cc: F. Markowski/SS, 981A
WG Edmonds/SS, 982A
JG Tellefson/SS, 994E
GL Hayes/GE, 981C
San Jose GE Files/394

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Em 5/15/86

FIRE PROTECTION ANALYSIS

FOR

HANFORD 2

PREPARED BY:

JMPL for Ram Seetharaman
Ram Seetharaman
Plant Performance Engineering

APPROVED BY:

DA Hamon acting for AE Rogers 6/27/86
A.E. Rogers, Manager
Plant Performance Engineering

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1. INTRODUCTION

This report presents an analysis performed to demonstrate adequate safe shutdown capability for a hypothetical fire event at Hanford 2. This analysis considers the performance of the minimum safe shutdown systems to the event scenario supplied by Washington Public Power Supply System (WPPSS). The analysis was performed to assist WPPSS in showing compliance with the requirements of 10CFR50.48 and 10CFR50 Appendix R.

Section III.L of 10CFR50 Appendix R requires that fuel cladding damage shall not occur during a plant fire event using the alternative or dedicated shutdown capability. Fuel cladding damage will not occur if the fuel heatup response, and in particular the peak cladding temperature (PCT), are sufficiently below the cladding perforation limits.

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2. SUMMARY AND CONCLUSIONS

Analyses were performed to demonstrate that the minimum set of safe shutdown systems defined for Hanford 2 have the capability to comply with the requirements of 10CFR50.48 and 10CFR50 Appendix R. The analyses considered the effects of loss of offsite power and loss of feedwater with only the minimum safe shutdown systems operable.

The analyses clearly demonstrate that adequate shutdown capability exists to meet the performance requirement of no fuel cladding damage as specified in 10CFR50 Appendix R for the event analyzed.

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3. ANALYSIS

3.1 ANALYSIS METHOD

The General Electric evaluation model (SAFE) was used to perform the analysis. The SAFE model predicts the reactor pressure, water level and steam flow response. If core uncover occurs, this information is then used to determine the steam cooling heat transfer coefficients for the fuel rods during the event as described in Section 3.5.2.4.2 of Reference 1. The General Electric evaluation model (CHASTE) is then used to calculate the fuel rod heat-up and the resulting Peak Cladding Temperature (PCT). The assumptions related to initial conditions, operator actions, and other event parameters are discussed in the following sections.

3.2 GENERAL EVENT DESCRIPTION

The initial conditions for the postulated fire event are described as follows. At the start of the postulated fire event, the reactor is assumed to be operating at full power, normal water level, and steady state conditions. The design basis fire event is assumed to occur at time zero with instantaneous loss of all unprotected safe shutdown systems. It is conservatively assumed that the loss of offsite power occurs at the same time which leads to events such as reactor scram, turbine trip, loss of feedwater and isolation.

Immediately after scram and isolation, the reactor pressure increase is limited by operation of the SRVs. After the SRVs open, the vessel pressure immediately drops.

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Normally the high pressure makeup systems, i.e., High Pressure Core Spray (HPCS) and Reactor Core Isolation Cooling (RCIC), will operate to supply coolant inventory upon a sufficient drop in reactor water level. For this event, however, the RCIC and the HPCS systems are assumed unavailable since they are not operable from the remote shutdown panel or will be disabled by the postulated fire.

It is assumed that after 10 minutes 6 relief valves will be manually opened to depressurize the vessel. As the reactor depressurizes, RHR loop "B" only comes on (automatically) for LPCI. Only one LPCI pump is assumed available. The LPCI pump is then able to deliver water into the vessel and rapidly restore the water level to near normal.

3.3 ASSUMPTIONS

The analysis of this loss-of-coolant event is based on the following assumptions:

- a. No credit is assumed for offsite power. For analysis purposes, this assumption is simulated by the loss of offsite power and reactor isolation at time zero to maximize the primary system stored energy.
- b. Six safety relief valves and one LPCI system are assumed available. The relief valve depressurization is assumed to be initiated 10 minutes into the event.
- c. The fire event does not occur simultaneously or coincident with any other abnormal conditions except the loss of offsite power. No other challenges to the safe shutdown systems are considered as part of this analysis.

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- d. Plant operating and system actuation parameters are consistent with the plant safety analysis and technical specifications for the initiating event.
- e. The reactor decay heat is modeled with the mean value of the 1979 ANS decay heat. This is judged to be the most appropriate decay heat assumption for this study and is also consistent with previous analyses for the Emergency Procedure Guidelines (Reference 1).
- f. The initial core power is conservatively set at 104.2% of rated. Initial steam flow, core flow, and vessel pressure are consistent with the heat balance for 104.2% rated core power.
- g. The core remains in nucleate boiling until core uncover. This is again supported by test results described in Reference 1.
- h. Essentially no credit is taken for feedwater flow. With the loss of offsite power, the flow is linearly ramped to zero flow in 1 second. The feedwater enthalpy is constant during the coastdown.

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4. RESULTS

The performance of the minimum safe shutdown systems was evaluated for the event scenario described in Section 3.2 assuming the conditions discussed in Section 3.3. The results of the analysis are shown in Figures 4-1 through 4-5. The depressurization and rate of vessel inventory loss increases when the 6 SRVs are manually opened 600 seconds into the event. The minimum water level inside the shroud occurs about 15 minutes into the event and is located approximately 4.1 feet below the top of the active fuel. As soon as LPCI flow enters the vessel the water level begins to rise. The fuel node having the highest calculated peak cladding temperature (PCT) is uncovered for 272 seconds. The PCT is calculated to be 762°F and occurs approximately 19 minutes after initiation of the event.

No fuel cladding damage is expected to occur at this low temperature. No fuel rod perforations are expected to occur below 925°C (approximately 1700°F) based on General Electric cladding swelling and rupture model (see References 2 and 3). This model has been approved by the US NRC (Reference 4). Cladding expansion begins about 200°F lower than the perforation temperature (Reference 5), so no fuel cladding damage should occur as long as the peak cladding temperature remains below 1500°F.

The sensitivity of the peak cladding temperature to the specific fuel design is not expected to be substantial for this event since fuel uncover does not occur until well after initiation of the event and the period of core uncover is most strongly a function of ECCS performance characteristics. The actual peak cladding temperature is dependent on the thermal characteristics of the fuel (i.e. heat generation and gap conductance).

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While calculating suppression pool temperature no pool cooling is assumed, and the final pool temperature is the maximum value. The pool temperature is 90°F at the beginning of the transient. All the mass and energy flow out of the SRVs is assumed to be absorbed completely in the pool volume at 1575 seconds into the event when the water level inside the shroud is restored to its initial level. During this time approximately 420 million BTUs of thermal energy are added to the pool by 0.35 million pounds of steam. The initial pool mass is approximately 6.9 million pounds of water at 90°F. An energy balance yields a final pool temperature of approximately 145°F.

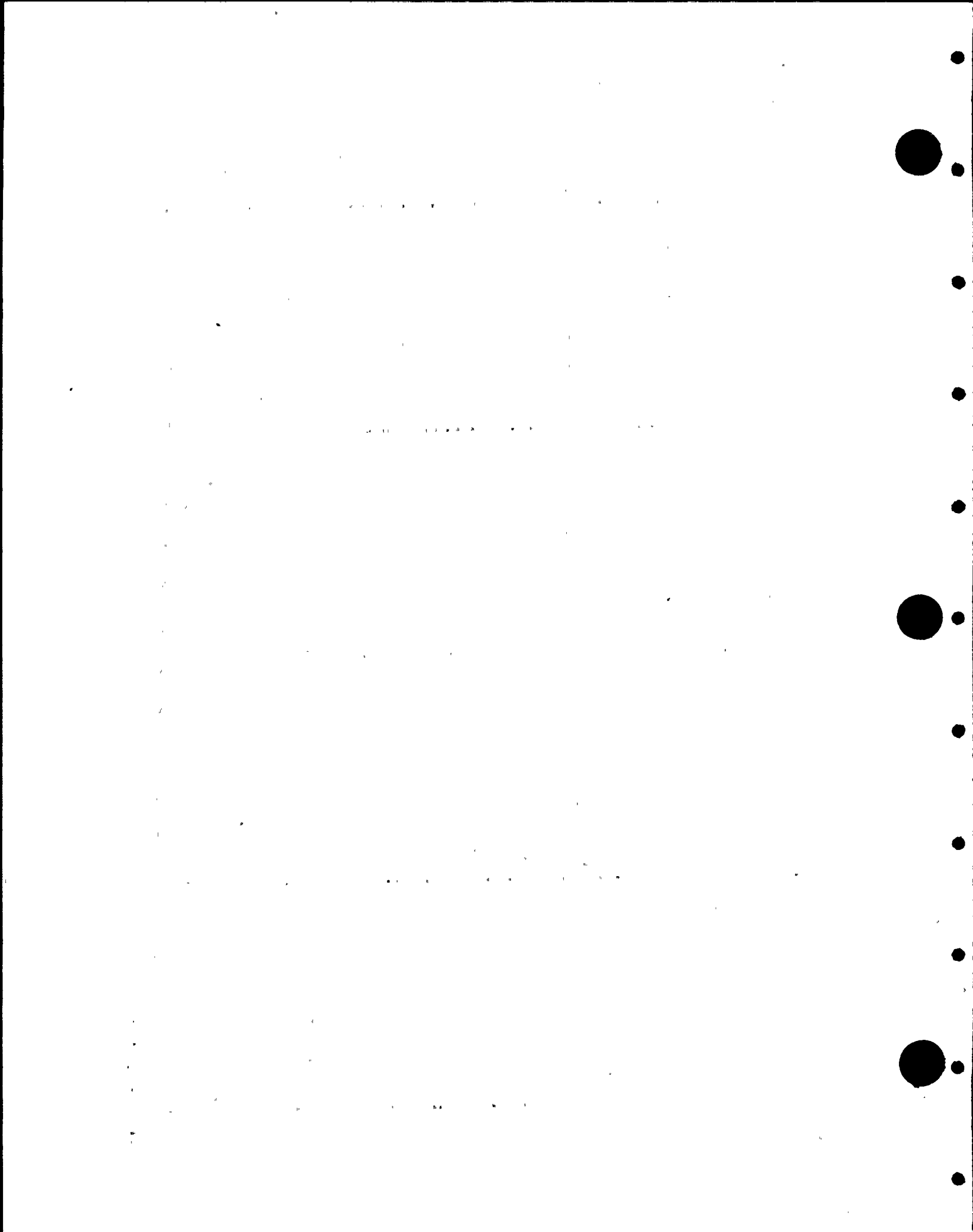


FIGURE 4-1 REACTOR PRESSURE VS TIME

HANFORD 2

0.0 BAK

PCI + GADS

1 SYSTEM PRESSURE

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PRESSURE (PSIA)

$\times 10^3$

1.2

0.8

0.4

0.

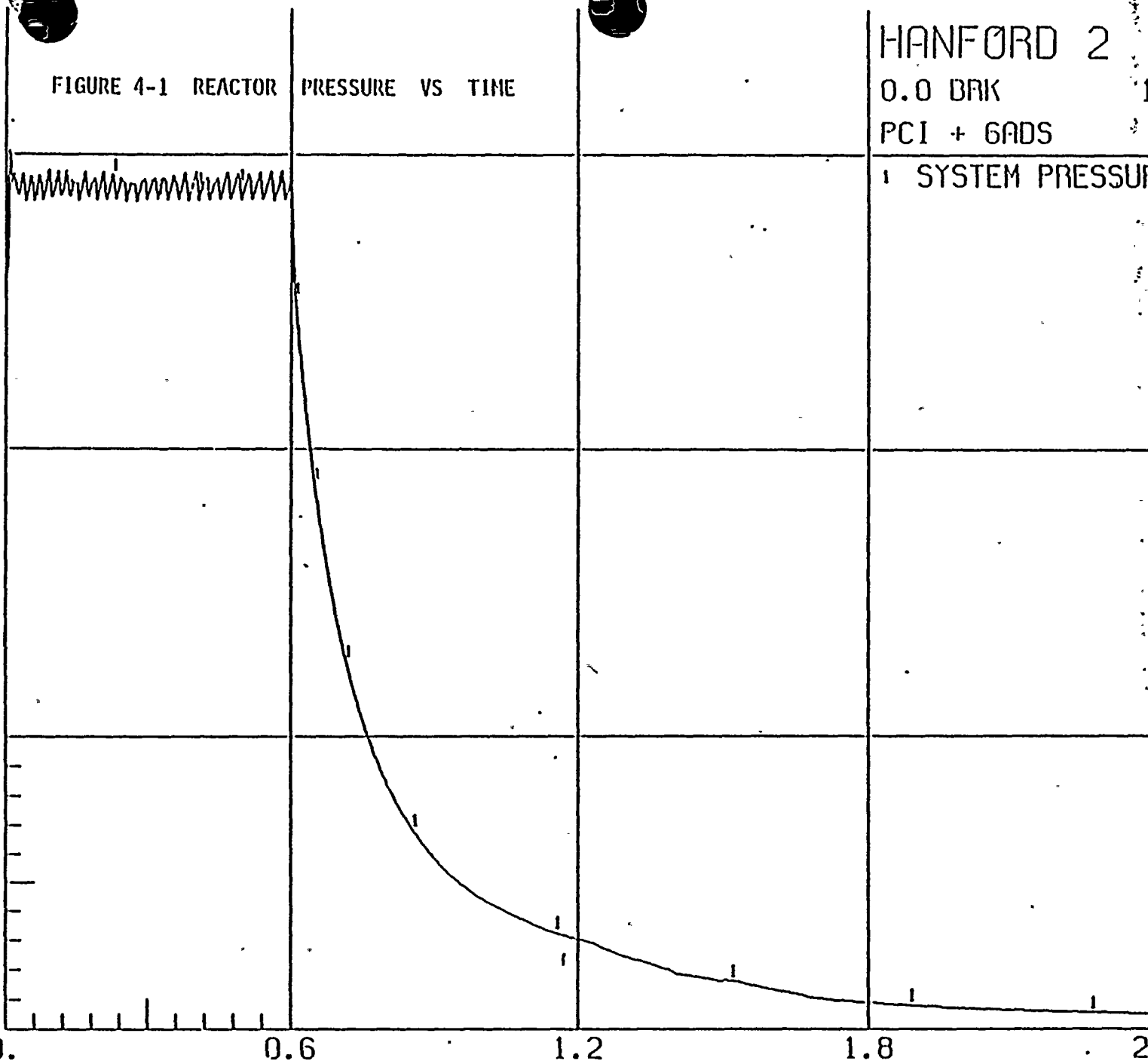
TIME (SEC)

0.6

1.2

1.8

2.4×10^3



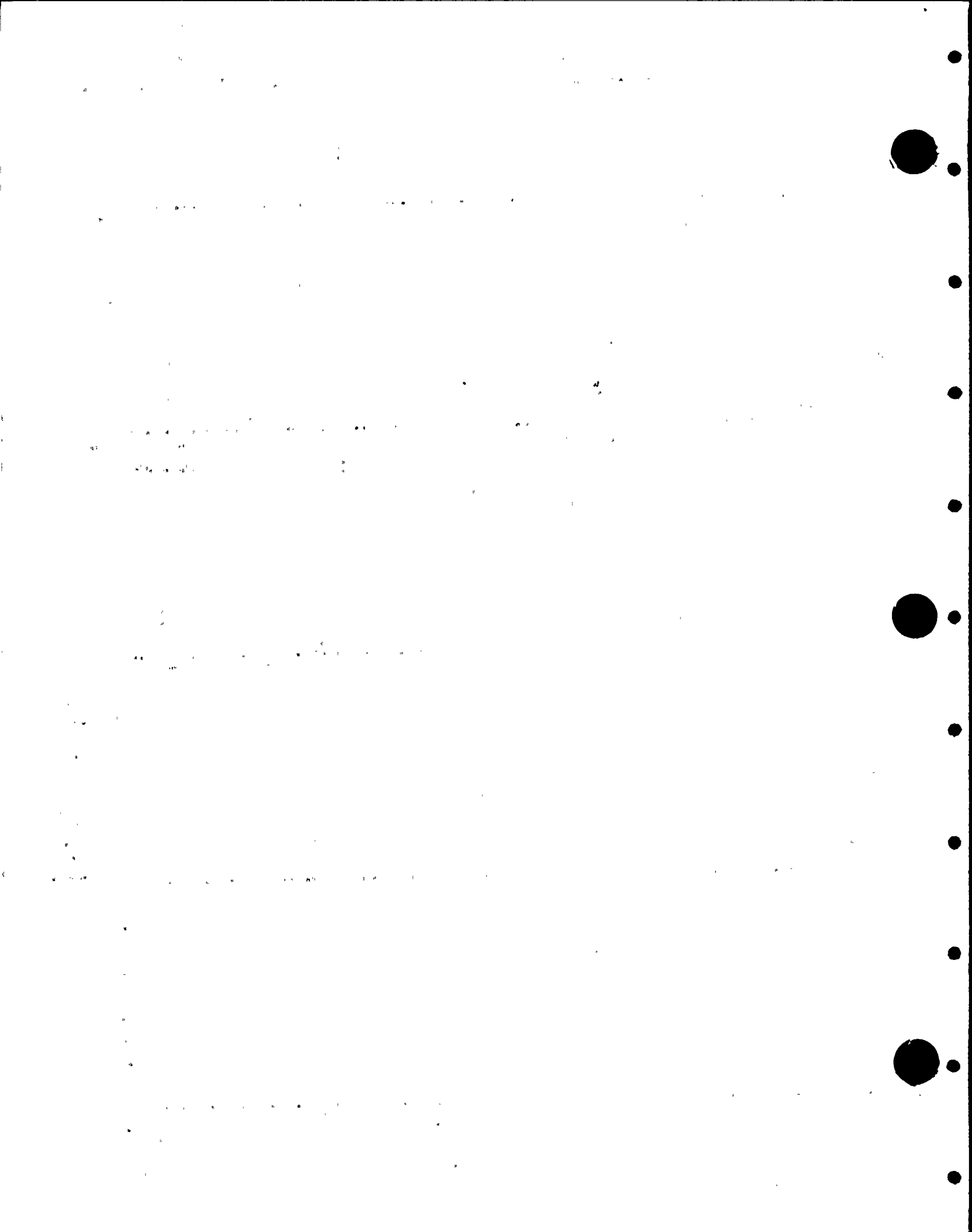


FIGURE 4-2 WATER LEVEL INSIDE SHROUD VS TIME

HANFORD 2

0.0 BRK

PCI + GADS

LEVEL 2 INSIDE SHROUD

WATER LEVEL (FT)

60.

40.

20.

0.

TIME (SEC)

0.6

1.2

1.8

2.4×10^3

TOP OF ACTIVE FUEL

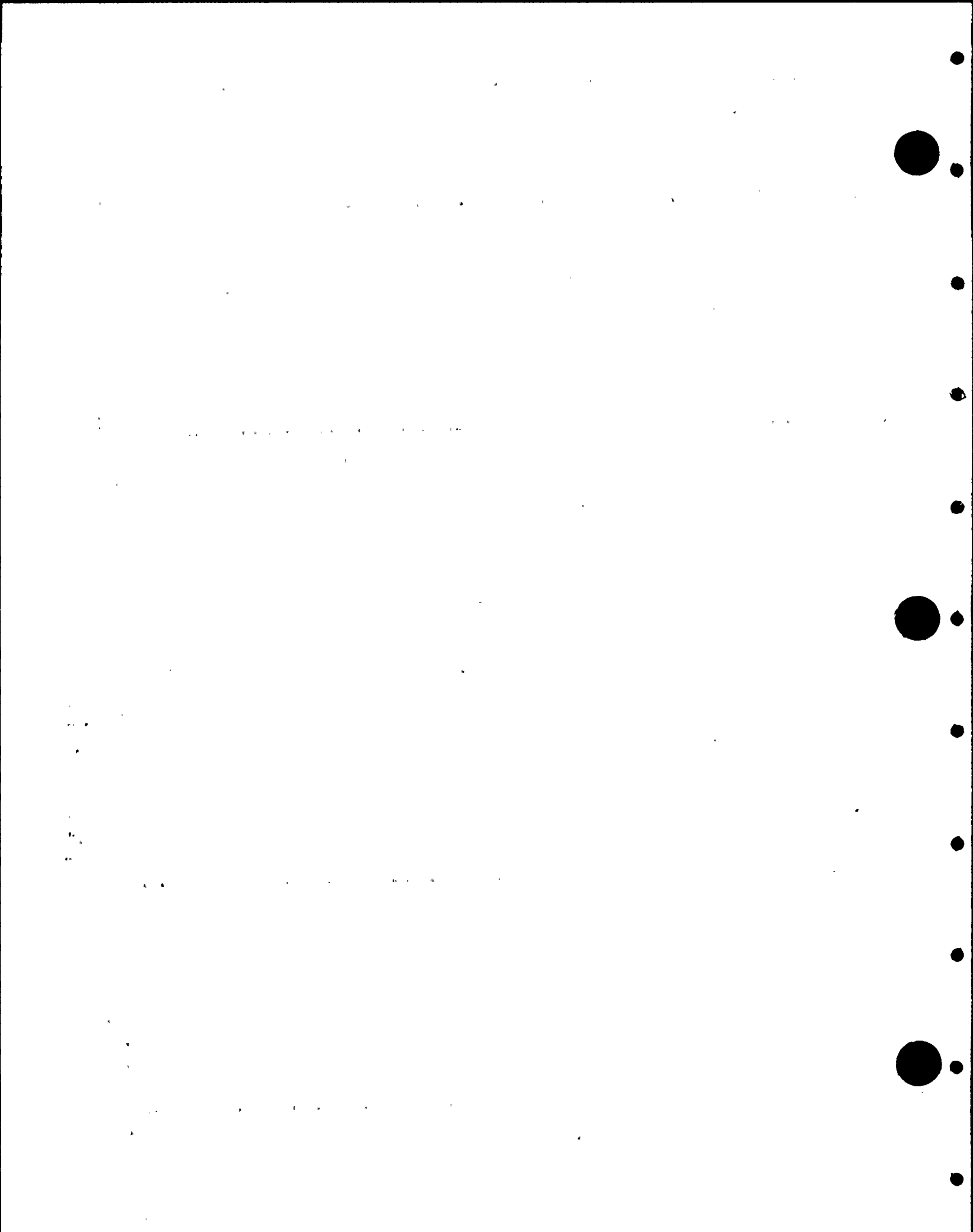
BOTTOM OF ACTIVE FUEL

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HANFORD 2

FIGURE 4-3

WATER LEVEL OUTSIDE

SHROUD VS TIME

0.0 BRK

1 L

PCI + GADS

1 LEVEL 4 OUTSIDE SHROUD

WATER LEVEL (FT)

60.

40.

20.

0.

0.6

1.2

1.8

2.4×10^3

TIME (SEC)

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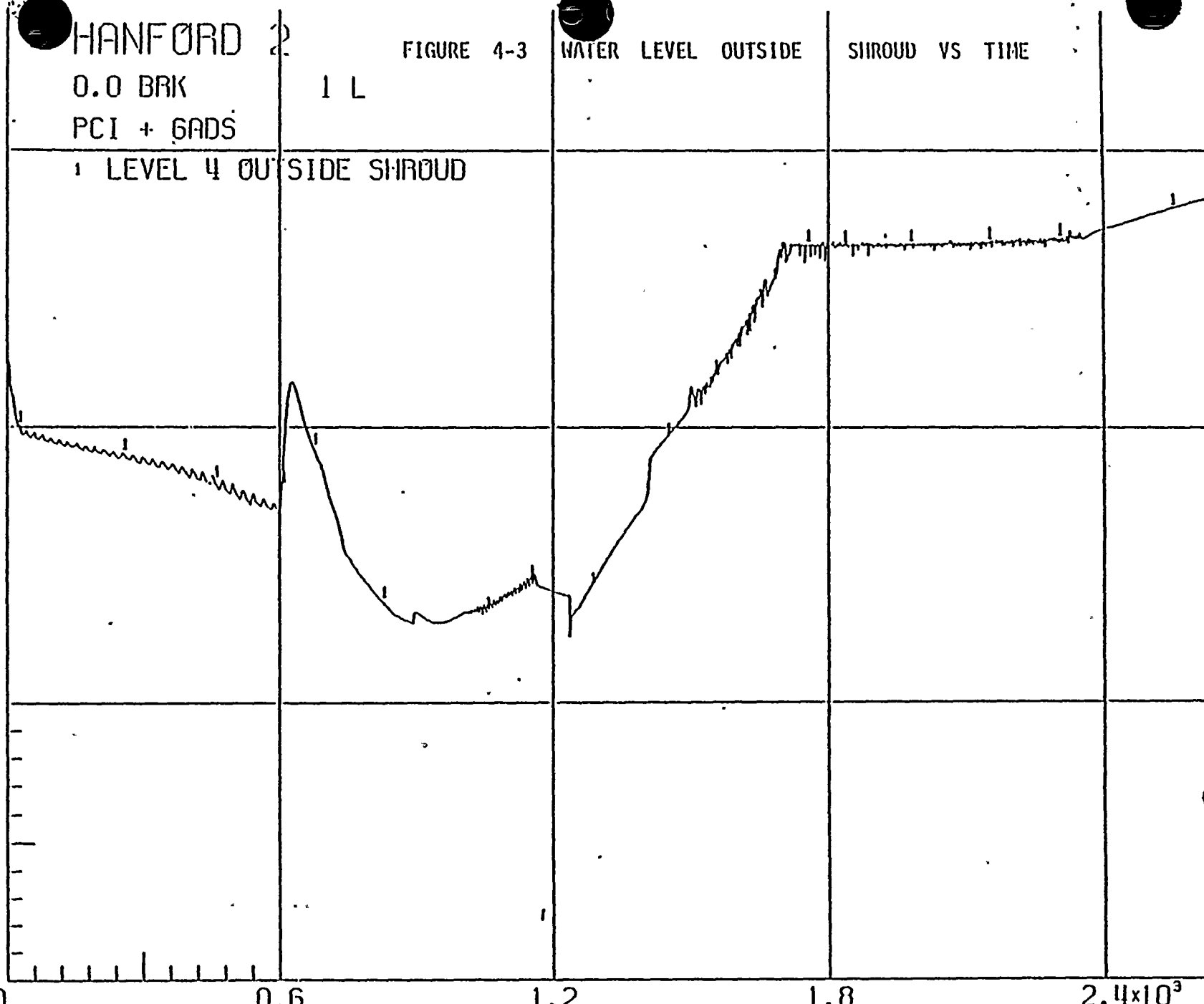


FIGURE 4-4 PEAK CLAD

TEMPERATURE VS TIME

KK1 - HANFORD 2
PEAK CLAD TEMP
VERSUS TIME

PEAK CLAD TEMP-DEG F

$\times 10^3$

3.

2.

1.

600

500

0.

1,000 F

1

10

100

1000

TIME (SECONDS)

LOG SCALE

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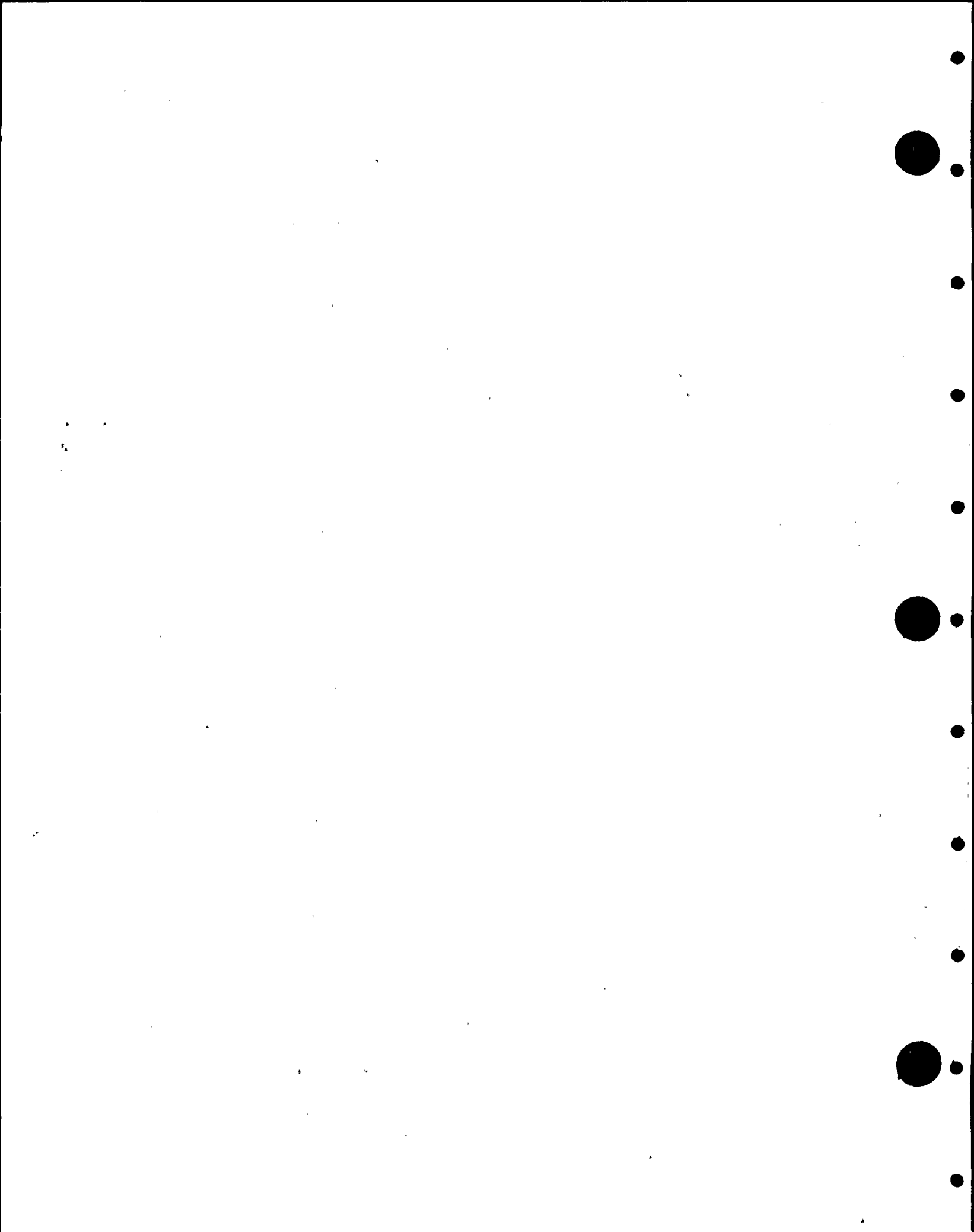
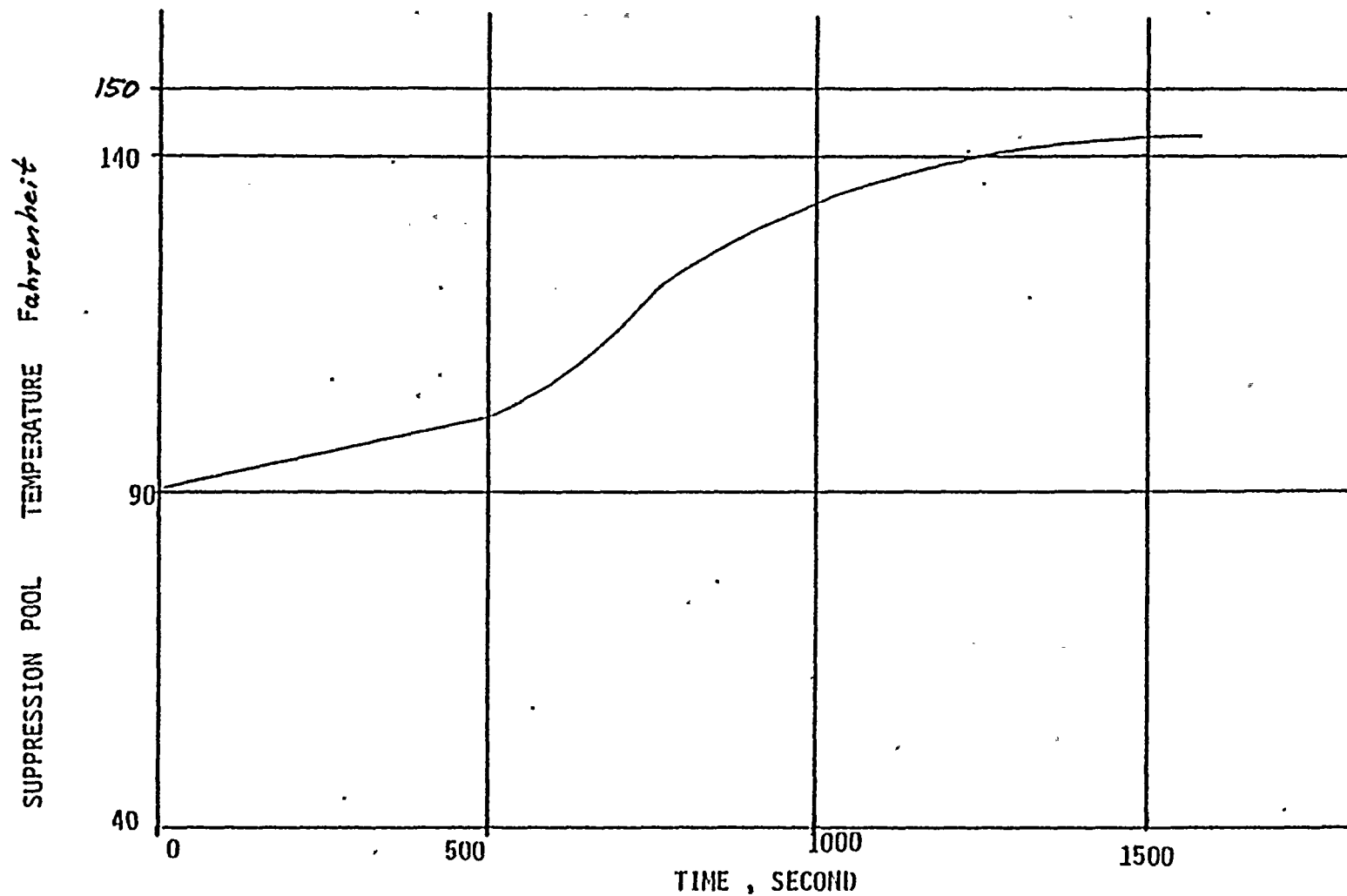


FIGURE 4-5

POOL TEMPERATURE VS. TIME
(SUPPRESSION POOL)



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Fig. 4-6

RV Flow } vs Time
Injection Flow }

HANFORD 2

0.0 BRK

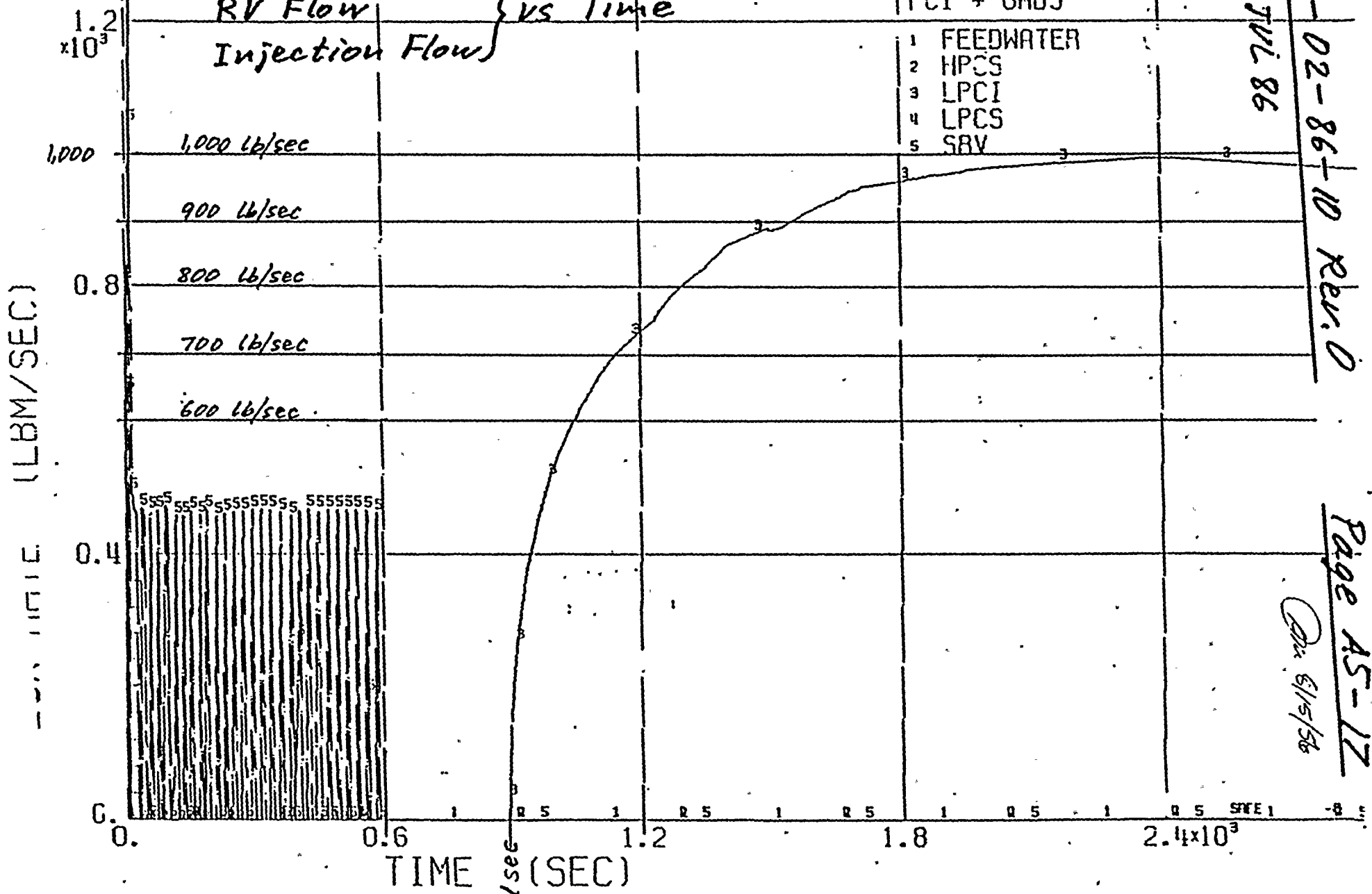
PCI + GADS

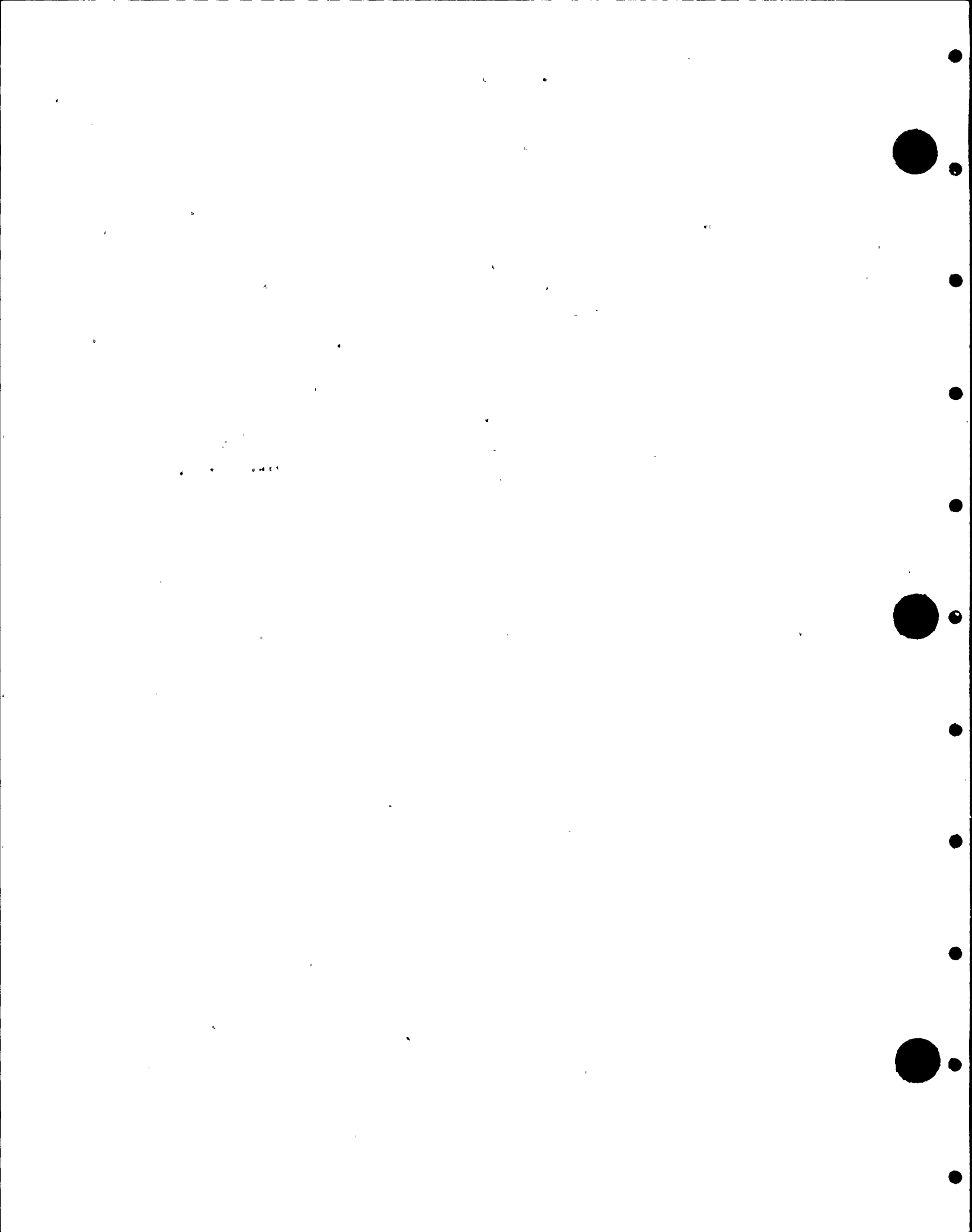
- 1 FEEDWATER
- 2 HPCS
- 3 LPCI
- 4 LPCS
- 5 SBV

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5. REFERENCES

1. "Additional Information Required for NRC Staff Generic Report on Boiling Water Reactors", NEDO-24708A, Revision 1, December, 1980.
2. Letter, R. H. Buchholz (GE) to D.D. Eisenhut (NRC), "GE Cladding Hoop Stress at Perforation, "MEN 278-79, November 16, 1979.
3. "Cladding Swelling and Rupture Models for LOCA Analysis, "NUREG-0630, April 1980.
4. Letter, Harold Bernard (NRC) to G.G. Sherwood (GE), "Supplementary Acceptance of Licensing Topical Report NEDE 20566A(P), " May 11, 1982.
5. "General Electric Company Analytical Model for Loss-of-Coolant Analysis in Accordance with 10CFR50 Appendix K, Vol. I, "NEDO-20566, January 1976.

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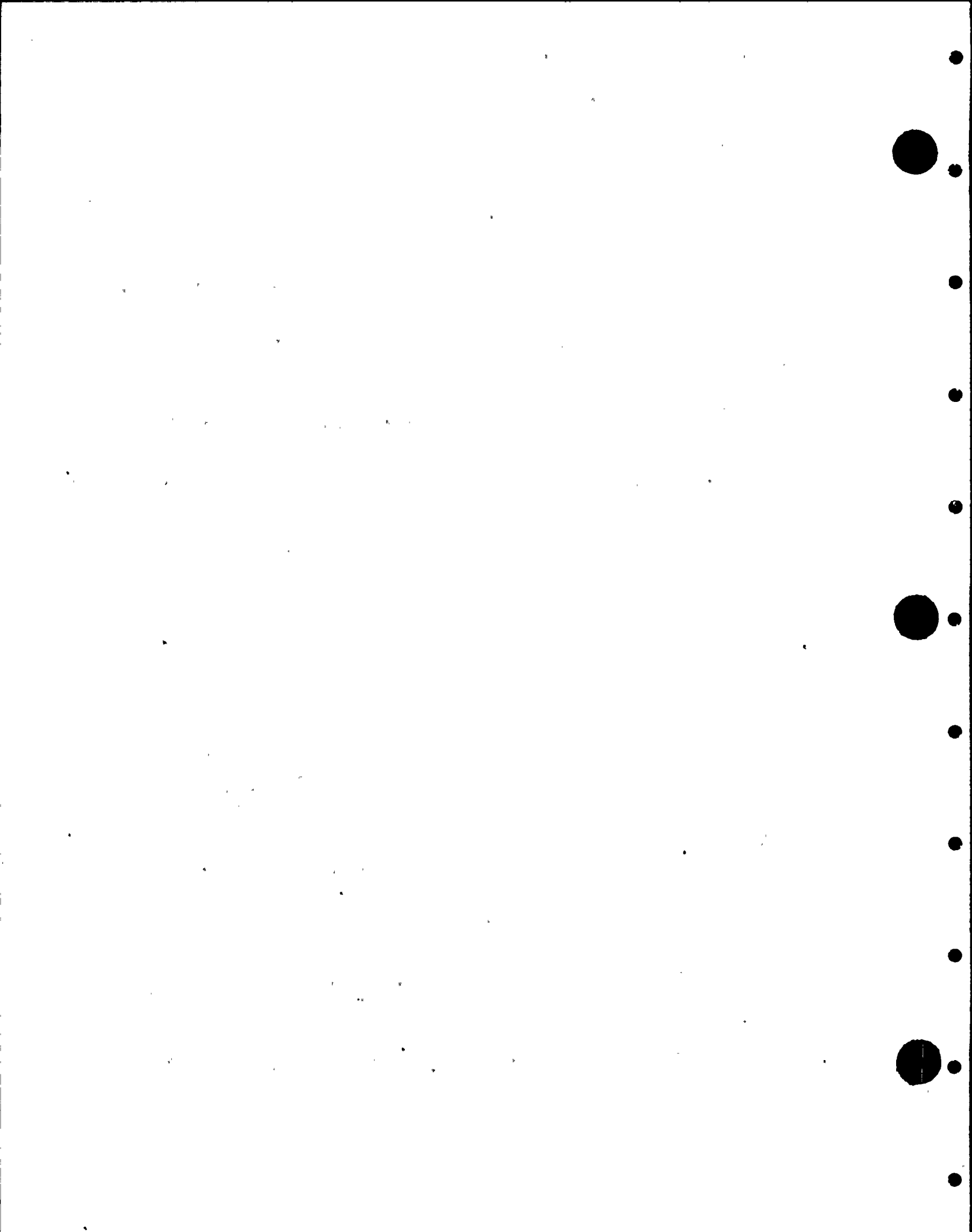
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APPENDIX 6

Comparison of Results and
Discussion of Differences between
G-KK-6-140 and NE-02-86-10



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The independent analysis done by GE (App. 5) shows somewhat less favorable results than NE-02-86-10:

1. The fuel uncover time is longer
2. The minimum coolant level is lower
3. The injection flow is lower

Consequently, all significant input data and modeling features were discussed with GE. It was found that the original WPPSS calculation needed the stored heat adjusted (see Pages A2-09 thru A2-12), and it needed to account

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for the RHR pump miniflow (about 500 gal/min), which causes a decrease in available shutoff head by about 7.5 psid. After these two adjustments were made, the following differences remain:

1. The WPPSS analysis uses the actual RHR pump curve (see Page 5.047), while the GE analysis is based on a generic pump curve, adjusted for FSAR Chap. 15 type analysis (FSAR Figure 6.3-9, see next page). The FSAR curve has a shutoff head of 227.9 psid, while the actual curve has a

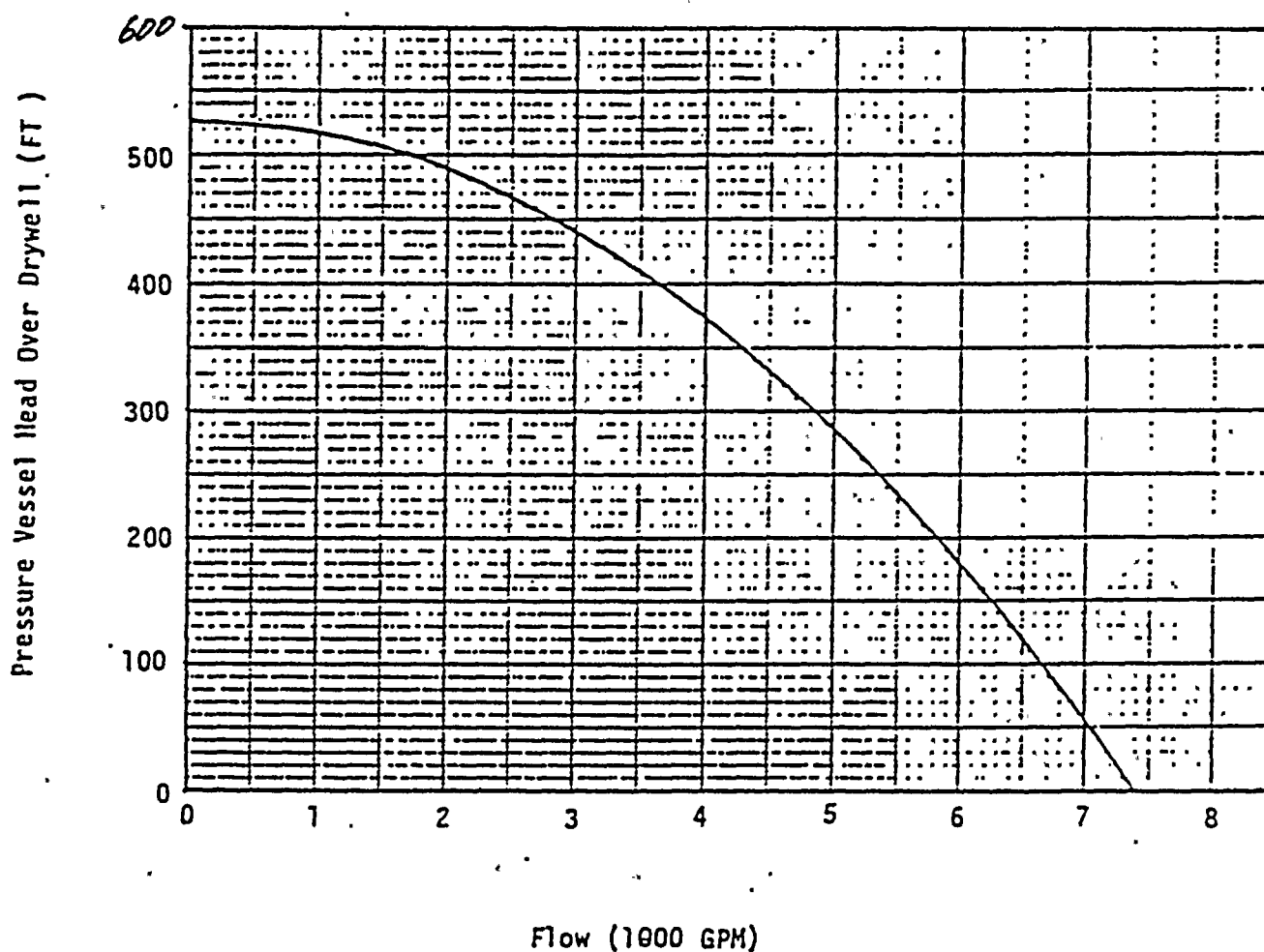
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(FOR 1 PUMP ONLY)



→ Curve is tabulated
on next page

PUMP CURVE

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PUMP	FLOW	HEAD	HEAD
1	0.00	111.70	111.70
2	0.00	111.70	111.70
3	0.00	111.70	111.70
4	0.00	111.70	111.70
5	0.00	111.70	111.70
6	0.00	111.70	111.70
7	0.00	111.70	111.70
8	0.00	111.70	111.70
9	0.00	111.70	111.70
10	0.00	111.70	111.70
11	0.00	111.70	111.70
12	0.00	111.70	111.70
13	0.00	111.70	111.70
14	0.00	111.70	111.70
15	0.00	111.70	111.70
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95	0.00	111.70	111.70
96	0.00	111.70	111.70
97	0.00	111.70	111.70
98	0.00	111.70	111.70
99	0.00	111.70	111.70
100	0.00	111.70	111.70

Pump Curve

***** INITIAL CONDITIONS ***** TUE, 29 MAY 86, 08:00 HR *****

TOTAL SYSTEM VOLUME = 21008 FT**3
 LIQUID VOLUME WITHOUT VOIDS = 12166.1 FT**3
 LIQUID VOLUME WITH VOIDS = 18071.3 FT**3
 VAPOR VOLUME INCLUDING VOIDS = 8899.7 FT**3 (TOTAL VAPOR)
 VAPOR VOLUME ABOVE ACTUAL LIQUID LEVEL = 7911.1 FT**3

MASS OF LIQUID ONLY (VOIDS EXCLUDED) = 355.42 SLB
 MASS OF VAPOR ABOVE LIQUID = 13.146 SLB
 TOTAL MASS OF VAPOR (VOIDS EXCLUDED) = 13.146 SLB
 TOTAL SLICED MASS (LIQUID+TOTAL VAPOR) = 372.514 SLB
 MASS OF THERMALLY ACTIVE STEEL STRUCTURE = 11650 SLB

COLLAPSED REACTOR LEVEL = 42.0486 FT (NO VOIDS)
 ACTUAL REACTOR LEVEL = 44.9743 FT (WITH VOIDS)
 REACTOR PRESSURE = 1097.17 LB/IN**2
 REACTOR DIAPHRAGM TEMPERATURE = 581.5 FAHRENHEIT
 AVERAGE COOLANT QUALITY = 0.022 PERCENT
 AVERAGE COOLANT VOID FRACTION = 1.800 PERCENT
 CORRESPONDING AVERAGE VOLUMETRIC VOID AT
 FULL POWER, WITH LEVEL AT SETPOINT LN = 11.067 PERCENT

SPECIFIC VOLUME OF LIQUID = 2.18711E-02 FT**3/LB
 SPECIFIC VOLUME OF VAPOR = .41662E FT**3/LB
 SPECIFIC ENTHALPY OF LIQUID = 552.047 BTU/LB
 SPECIFIC ENTHALPY OF VAPOR = 1182.41 BTU/LB

ALL DATA FOR STEADY STATE WRITTEN TO DISK, CHANNEL 2 ***

TUE, 29 MAY 86, 08:00 HR



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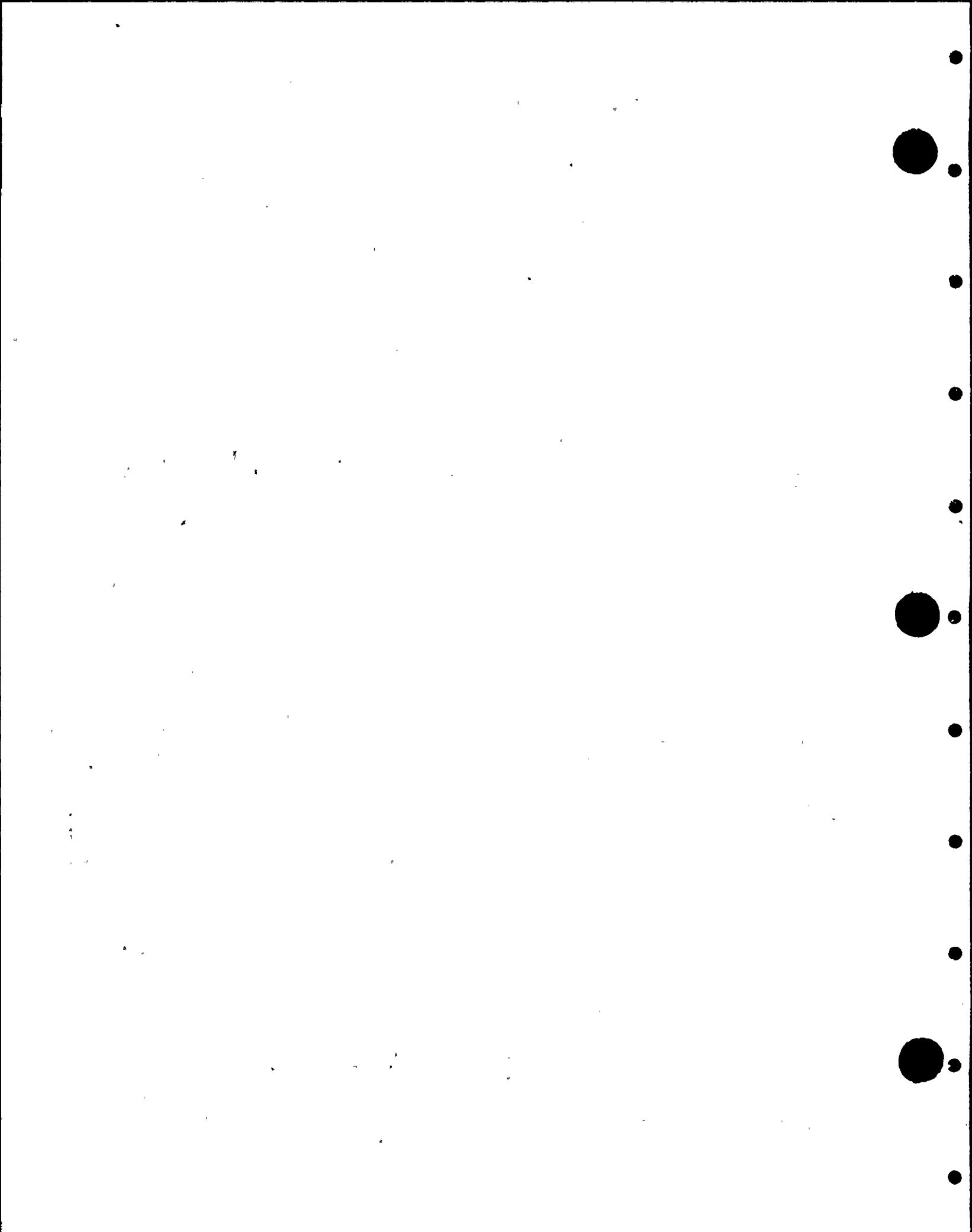
Date 8/15/86

- shutoff head of 303.2 psid
(minus 7.5 psid for miniflow).
The effect on results is significant.
2. The WPPSS decay power curve is about 12 pct too high after $T = 100$ sec, compared to the best estimate curve used by GE. The effect on results is minimal.
 3. For the total primary coolant system volume, the WPPSS figure of 21,005 ft³ is 4.5 pct low. Effect on results is negligible.

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4. For initial liquid inventory, the WPPSS figure of 555.35 Klb is 0.9 pct low. Effect on results: none.
5. For the relief valves, the GE analysis uses the spring setpoints combined with a 30 psid hysteresis (between opening and closing). The WPPSS analysis uses the power actuation set points combined with a hysteresis of 50 psid. The WPPSS data are more realistic, but no effect on results is expected.
6. For calculation of massflow thru the relief valves GE uses the Moody



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Table (homog. equilibrium model) with an effective valve crosssectional area calibrated at 1,200 psia. The WPPSS analysis uses the actual valve crosssection times a discharge coefficient, together with a sonic velocity table taken from the ASME Steam Tables (see Pages 5.023, 5.024 and A4-05). The GE model tends to produce massflow values slightly on the low side for lower pressures (below ~400 psia), which is equivalent to having a slightly smaller valve crosssectional area. The ef-

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- fect on results should be negligible.
7. The GE analysis does not assume an increase in containment pressure, while the WPPSS analysis assumes an increase to 30 psia = 15 psig, based on the fact that containment cooling is postulated to be unavailable during the transient. The effect on results is noticeable (higher injection flow because of higher pump suction pressure).
 8. The GE analysis uses an initial liquid level of 44.6 ft, while the WPPSS model uses 45.0 ft. According to GE, both of these values

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are conservative. A true best estimate would be 46.6 ft, based on realistic feedpump coastdown and control system action. However, the higher level of 46.6 ft was not used. Effect on results would be small.

9. The WPPSS model has only 2 fluid nodes in the reactor vessel: one for liquid, one for vapor. Consequently, the LPCI flow is mixed with the entire liquid inventory. This results in the liquid node staying in saturation (and no void collapse taking place) during depressuri-

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zation: stored heat, decay heat and excess enthalpy from the depressurizing liquid supply enough heat to sustain flashing even with the coolant coming in.

The GE model⁽¹⁾ is much more articulate, it has 5 fluid nodes in the vessel. However, it lumps all core coolant inside the shroud into one node, ignoring the fuel assembly cans. The incoming liquid is then mixed with this node, causing it to go subcooled (all voids collapse). The level trace on

⁽¹⁾ SAFE code

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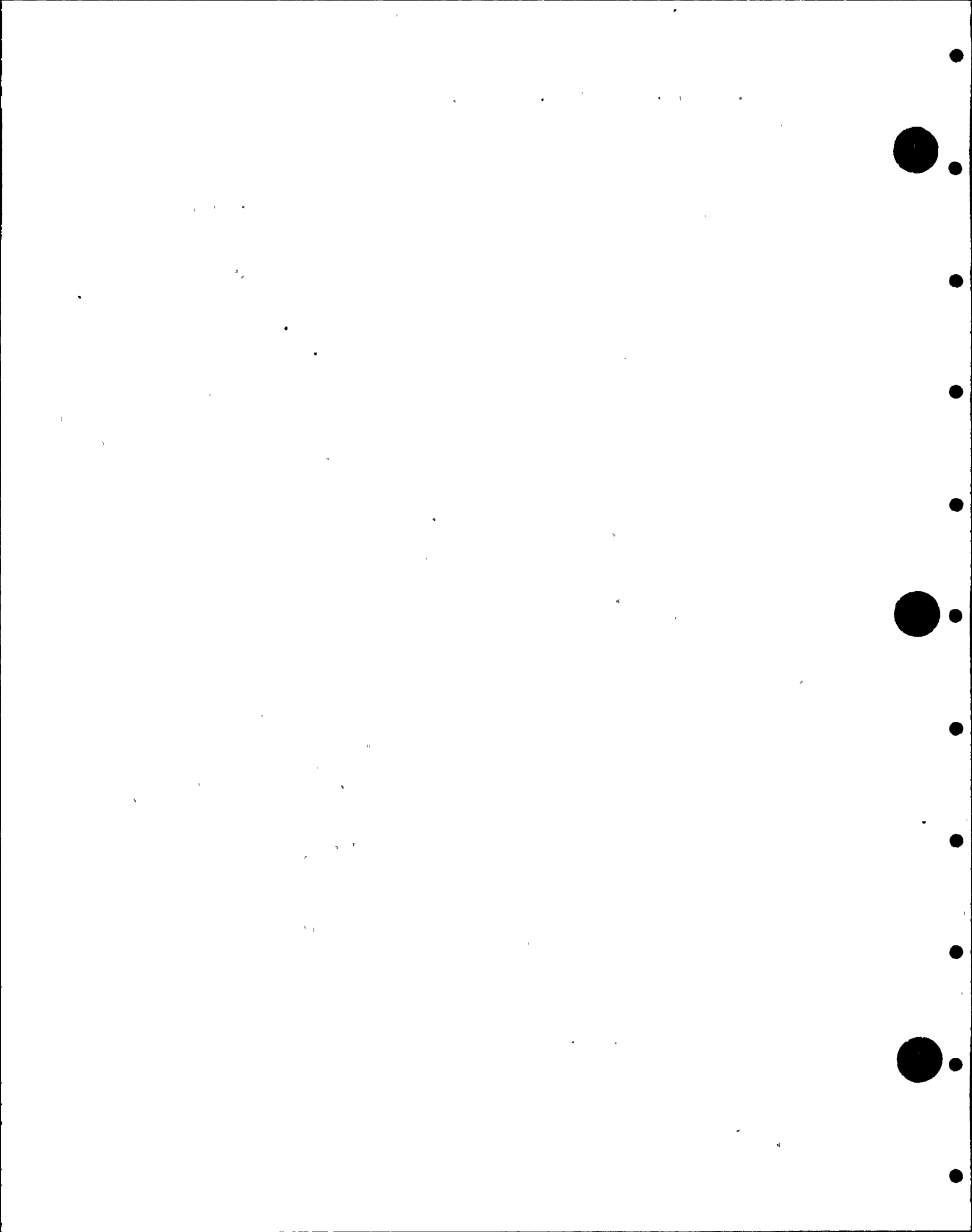
Page A5-13 is for this node.

At first glance the GE model appears more realistic than the WPPSS model. However, the opposite is the case. The WPPSS level trace is somewhat more representative of the liquid level inside the fuel cans, while the GE trace is more representative of the liquid level outside the cans (also called bypass region). Therefore, the GE trace is pessimistic as far as actual fuel uncover is concerned.

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In physical reality, LPCI goes into the bypass region, causing void collapse there and cooling the outside surface of the cans, while the liquid level inside the cans is being kept up by sustained flashing and by vapor bubbles blowing in thru the side entry orifices from the flashing lower plenum. The counter current flow limiting (CCFL) effect precludes draining of liquid from the cans. (Ref. 15, Ref. 16)



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The final conclusion is that the actual liquid level inside the cans is likely to be higher than the level trace from the WPPSS model indicates because flashing inside the cans is sustained throughout the critical "fuel uncover" period, without mixing of cold coolant inside the cans taking place. The GE trace is considered strongly conservative, more representative of the uncover of control blades than uncover of fuel assemblies.

(Rev. 8/15/86)

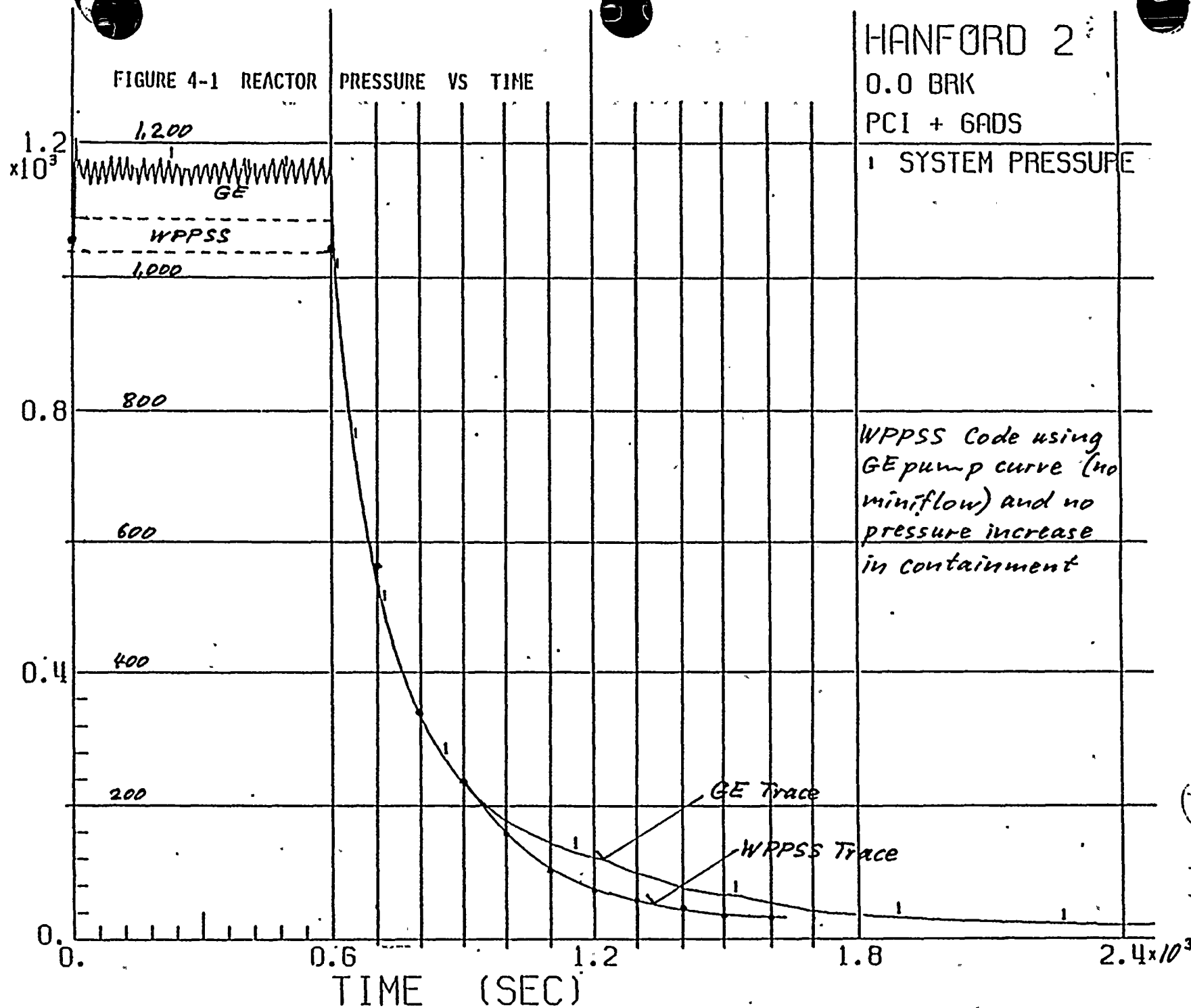
10. The maximum peak cladding temperature in the GE analysis is calculated with the CHASTE code, while the WPPSS analysis uses a highly conservative upper bound estimate. This results in the GE value (762 F) being 26 F below the WPPSS value (788 F) although the GE analysis shows a total fuel uncover time of 5.4 min⁽¹⁾ versus 3.0 min for the WPPSS analysis. The hot fuel node in CHASTE is uncovered for 4.5 min. (see Page A5-10).

(1) Page A5-13

(Dr 8/5/56)

For the purpose of verification, a comparison run has been made with the WPPSS model, using the decreased pump curve (Page A6-04) and no containment pressure increase. The traces on the following pages show the results. The graphs demonstrate good agreement between the 2 models.

PRESSURE (PSIA)



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FIGURE 4-2 WATER LEVEL INSIDE SHROUD VS TIME

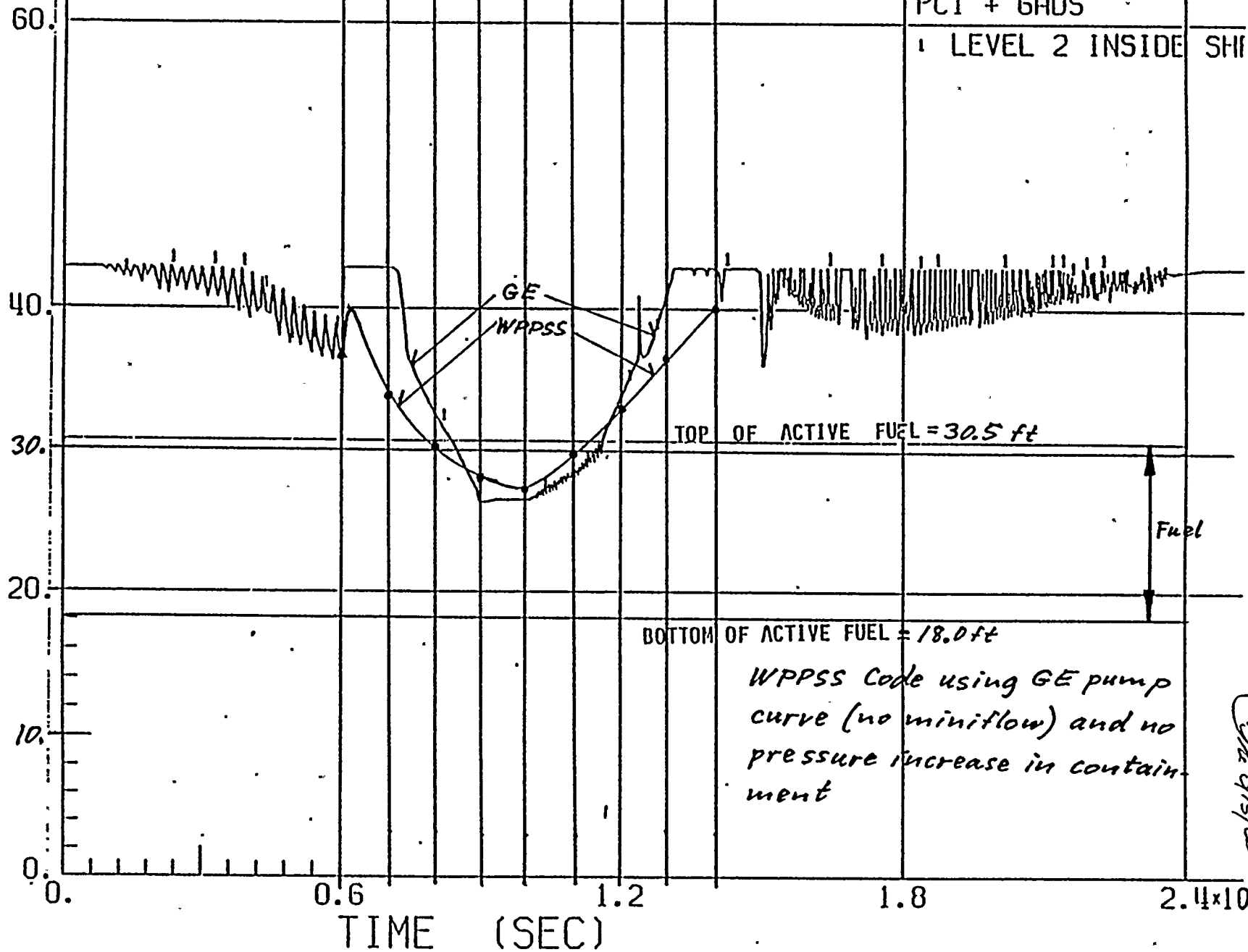
HANFORD 2

0.0 BRK

PCI + GADS

1 LEVEL 2 INSIDE SHI

WATER LEVEL (FT)



WPPSS Code using GE pump curve (no miniflow) and no pressure increase in containment

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Fig. 4-6

RV Flow } vs Time
Injection Flow }

HANFORD 2

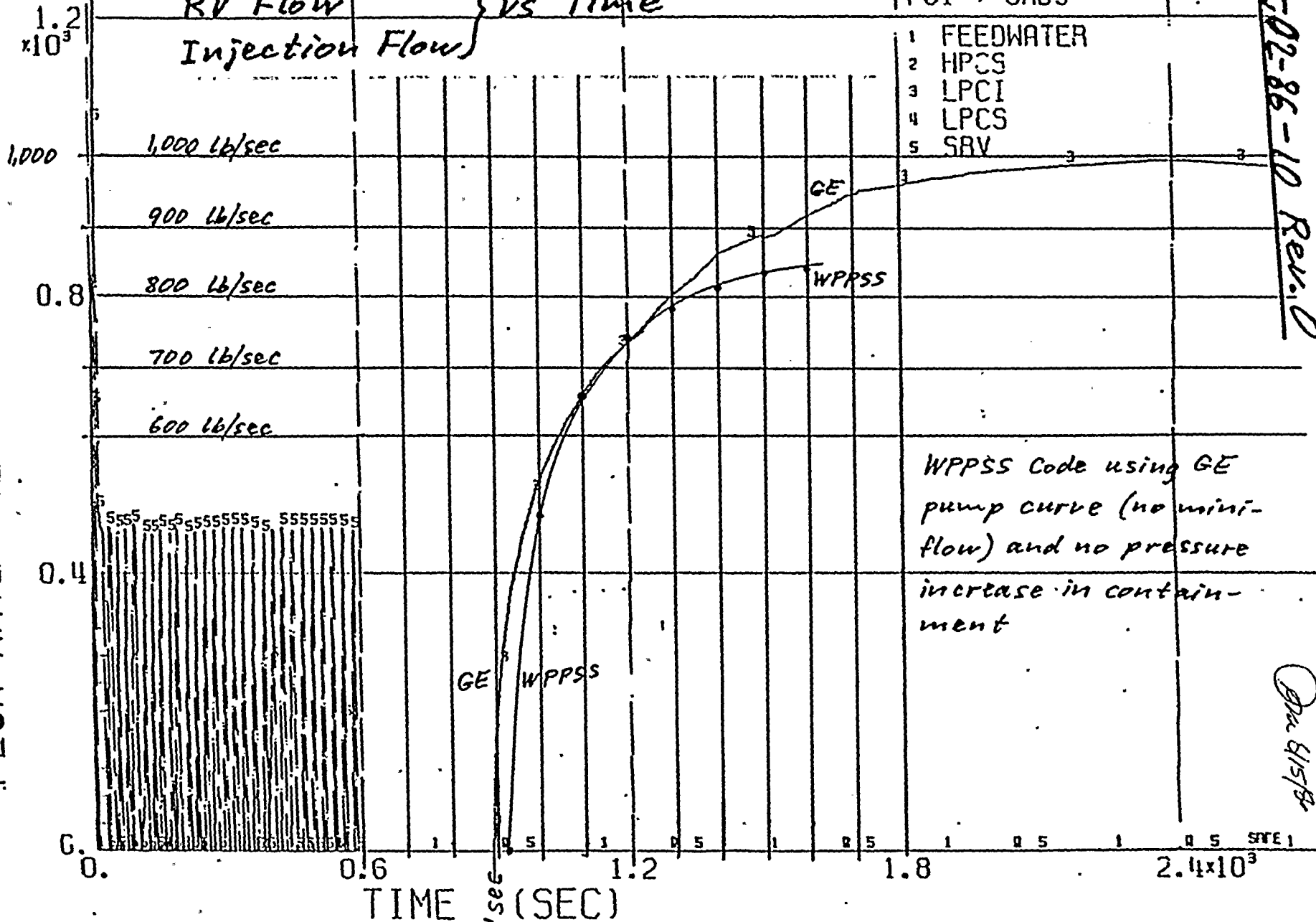
0.0 BRK

PCI + GADS

- 1 FEEDWATER
- 2 HPCS
- 3 LPCI
- 4 LPCS
- 5 SRV

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FLOW RATE (LBM/SEC)



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APPENDIX 7

Completeness of the Postulated
Hypothetical Scenario

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The scenario evaluated in this report is intended to bound the worst accumulation of fire damage and/or malfunctions postulated for a 10 CFR 50, Appendix R fire event. It is noted that the probabilities of simultaneous occurrence of the design basis fire with any combination of the other postulated failures are extremely small. Thus, the evaluated scenario does not have a single, well defined initiator, nor is the number and/or combination of postulated malfunctions based on a specific physical rationale.

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For the purpose of this evaluation the question of completeness has been raised: is there a modification in boundary and/or initial conditions which would alter the final results in a thermally detrimental way?

Out of several possibilities which were evaluated, only two were identified which would potentially affect the conclusions in an adverse way:

Case 1: Offsite power is not lost, ~~so~~ ^{and} ~~that~~ containment cooling is available. Consequently, no containment pressure increase would be postulated,

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which would result in slightly lower injection flow.

Case 2: A relief valve either sticks open or is kept open by a hot short, so that liquid coolant depletion during the first 10 min is increased.

As the tabulation of key relevant results on Page A7-05 shows, the differences between Case 1 and the Base Case are minimal: the minimum level is lower by 0.2 ft, the total uncover time is increased by only 20 sec, and the upper bound estimate for maximum peak cladding temperature is in-

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Case	Liquid Level at $t=10$ min	Reactor Pressure at $t=10$ min	Minimum Liquid Level	Total Fuel Uncovery Time	Estimated Max. Peak Cladding Temp.
—	ft	psia	ft	sec	F
Base Case	36.2	1039	28.5	177	788
Case 1	36.2	1039	28.3	197	828
Case 2	33.8	883	28.6	172	777

For Base Case see Page 5.060 and Appendix 4

Case 1 = No loss of offsite power

Case 2 = 1 relief stuck open for $t > 1.0$ sec

Two modifications of the base case

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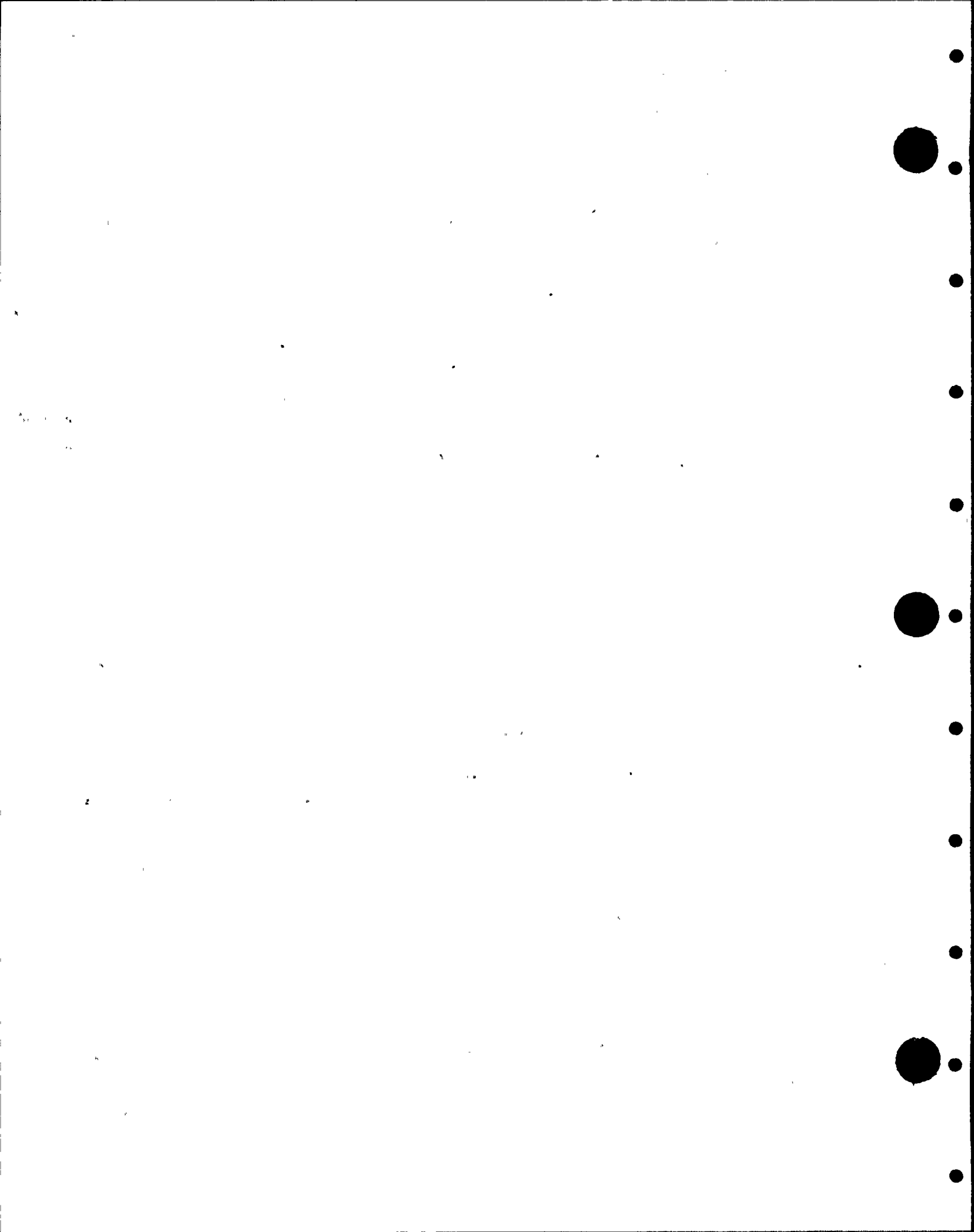
creased by 40 Fahrenheit. The PCT was calculated from

$$\underline{T = 434 + (2)(197) = 828 F}$$

For Case 2 the results look actually better than the Base Case. Its upper bound estimate for maximum peak cladding temperature is

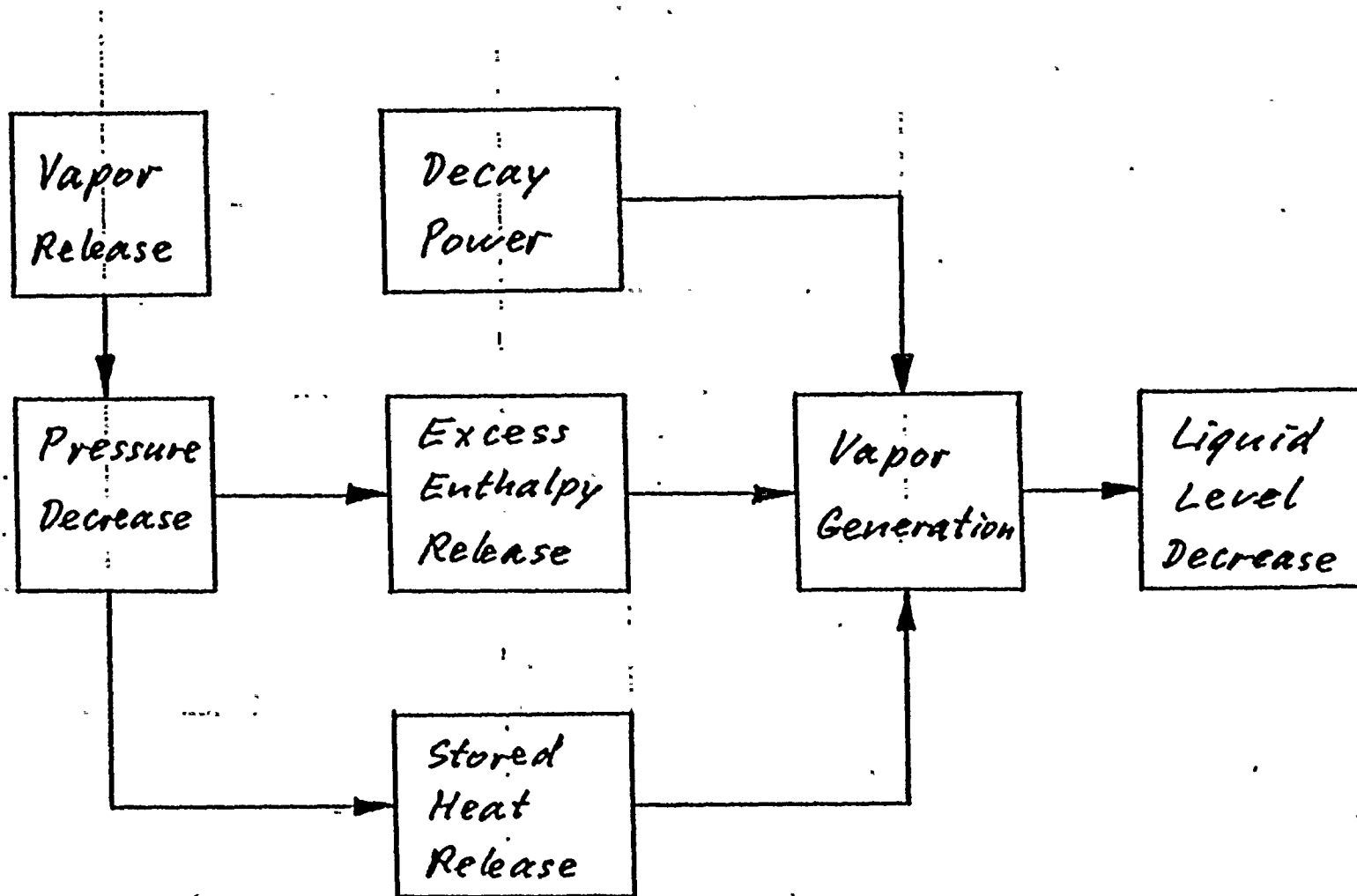
$$\underline{T = 433 + (2)(172) = 777 F}$$

Looking at this case, it can basically be stated that each psi of pressure decrease is "paid for" by a certain amount of liquid lost to evaporation. The sketch on Page A7-07 illustrates this. The amount of ex-



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The three heat sources during depressurization

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cess enthalpy per psi of pressure decrease, and also the heat of evaporation, vary somewhat with pressure. It appears from the results of Case 2 that the thermal "price" to be paid (in terms of lb of liquid per psi of pressure decrease) is slightly lower at elevated pressures.

Although it is possible to construct various modifications for this scenario which would somewhat affect the numerical results, no modifications have been identified which would invalidate

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~~validate~~ the conclusion that the
plant is capable of tolerating
this postulated event with a
wide margin of Safety.

