

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8405140390 DOC. DATE: 84/05/04 NOTARIZED: NO DOCKET #
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Power 05000397
 AUTH. NAME: SORENSEN, G.C. AUTHOR AFFILIATION: Washington Public Power Supply System
 RECIP. NAME: SCHWENCER, A. RECIPIENT AFFILIATION: Licensing Branch 2

SUBJECT: Requests changes to Tech Specs clarifying design alarm identifying changes to battery terminal voltages for 125 & 250-volt batteries & correcting identification for proper breaker. Marked-up Tech Specs encl.

DISTRIBUTION CODE: B001S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 6
 TITLE: Licensing Submittal: PSAR/FSAR Amdts & Related Correspondence

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL		RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	NRR/DL/ADL	1 0		NRR LB2 BC	1 0
	NRR LB2 LA	1 0		AULUCK, R. 01	1 1
INTERNAL:	ELD/HDS2	1 0		IE FILE	1 1
	IE/DEPER/EPB 36	3 3		IE/DEPER/IRB 35	1 1
	IE/DQASIP/QAB21	1 1		NRR/DE/AEAB	1 0
	NRR/DE/CEB 11	1 1		NRR/DE/EHEB	1 1
	NRR/DE/EQB 13	2 2		NRR/DE/GB 28	2 2
	NRR/DE/MEB 18	1 1		NRR/DE/MTEB 17	1 1
	NRR/DE/SAB 24	1 1		NRR/DE/SGEB 25	1 1
	NRR/DHFS/HFEB40	1 1		NRR/DHFS/LQB 32	1 1
	NRR/DHFS/PSRB	1 1		NRR/DL/SSPB	1 0
	NRR/DSI/AEB 26	1 1		NRR/DSI/ASB	1 1
	NRR/DSI/CPB 10	1 1		NRR/DSI/CSB 09	1 1
	NRR/DSI/ICSB 16	1 1		NRR/DSI/METB 12	1 1
	NRR/DSI/PSB 19	1 1		NRR/DSI/RAB 22	1 1
	NRR/DSI/RSB 23	1 1		REG FILE 04	1 1
	RGN5	3 3		RM/DDAMI/MIB	1 0
EXTERNAL:	ACRS 41	6 6		BNL (AMDTS ONLY)	1 1
	DMB/DSS (AMDTS)	1 1		FEMA-REP DIV 39	1 1
	LPDR 03	1 1		NRC PDR 02	1 1
	NSIC 05	1 1		NTIS	1 1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 53 ENCL 46

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

May 4, 1984
G02-84-285

Docket No. 50-397

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Schwencer:

Subject: NUCLEAR PLANT NO. 2
OPERATING LICENSE NPF-21, REQUEST
FOR LICENSE AMENDMENT

- References:
- 1) Letter, G02-84-032, G. C. Sorensen (SS) to A. Schwencer (NRC), Same Subject, dated January 20, 1984
 - 2) Letter, G02-84-126, G. C. Sorensen (SS) to A. Schwencer (NRC), Same Subject, dated March 9, 1984
 - 3) Letter, G02-84-129, G. C. Sorensen (SS) to A. Schwencer (NRC), Same Subject, dated March 13, 1984
 - 4) Federal Register (FR), April 6, 1983, pages 14864-14880, "Standards for Determining Whether License Amendments Involve No Significant Hazards Considerations"

The references requested changes to specific portions of the WNP-2 technical specifications. Recent discussions between NRC staff and Supply System reviewers have identified additional changes required to three technical specification changes requested in the references. Accordingly, per the Code of Federal Regulations, Title 10, Parts 50.90 and 2.101, Washington Public Power Supply System requests revisions, as attached, to the Technical Specification, Appendix A to the subject license.

The Supply System has reviewed the technical specification changes enclosed per 10 CFR 50.59 and 50.92 and considers that no significant hazards or unreviewed safety questions will result from the changes. A 50.92 analysis is provided for each technical specification change. The appropriate example amendment that is considered not likely to involve significant hazards (as listed in FR 14870, Reference 4) is cited where applicable.

8405140390 840504
PDR ADDOCK 05000397
P PDR

Boo!
1/1

A. Schwencer
Page Two
May 4, 1984

The proposed changes will not:

- 1) involve a significant increase in the probability or consequences of an accident previously evaluated; or
- 2) create the possibility of a new or different kind of accident from any accident previously evaluated; or
- 3) involve a significant reduction in a margin of safety.

Enclosed are revised WNP-2 technical specification pages 3/4 9-3 - originally submitted in reference 1), 3/4 8-20 - originally submitted in reference 2), and 3/4 8-13 - originally submitted in reference 3).

o Page 3/4 9-3

As currently stated, an "audible indication" is required and has been interpreted by an NRC Region V inspector to require the type of audible indication employed by PWR vendors. An audible indication of SRM operating levels is not a standard BWR design. The present design alarms only to alert operators of increasing levels. The attached revised technical specification change clarifies the current technical specification. Example (i) of reference 4) is cited in justification for this change as it is administrative to correct error and achieve consistency with plant design.

o Page 3/4 8-20

Reference 2) identified breaker 2BR as in error and requested 8BR as correction. 8BR is in error and 8AR is the correct nomenclature. Additionally, reference 2) identified motor control center MC-3DR as being de-energized in operational conditions 1, 2, and 3. The correct identification is MC-3DA. Example (i) of reference 4) is cited in justification for these changes as they are administrative and correct errors.

o Page 3/4 8-13

Reference 3) identified changes to the battery terminal voltages for the 125 and 250 volt batteries for battery capacity operability demonstrations as being 102 and 203 volts, respectively. These values are in error and 105 and 210 volts are the correct values. Example (i) of reference 4) is cited in justification as this change is administrative and corrects an error. Additionally, example (ii) is cited as this change represents a more stringent requirement.

A. Schwencer
Page Three
May 4, 1984

The fees for consideration of these changes were provided separately in references 1), 2), and 3) and since the intent of the original requests has not changed, no additional fee is appropriate.

In accordance with 10 CFR 50.91, the State of Washington has been provided a copy of this letter. Should you have any questions, please contact Mr. P. L. Powell, Manager, WNP-2 Licensing.

Very truly yours,



G. C. Sorensen, Manager
Regulatory Programs

PLP/tmh
Attachments

cc: R Auluck - NRC
WS Chin - BPA
ND Lewis - EFSEC
AD Toth - NRC Site

ELECTRICAL POWER SYSTEMS

Note: All other changes with the exception of clouded area submitted under 602-84-129, dated March 13, 1984

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 18 months, during shutdown, by verifying that either:
1. The battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for 8 hours for Divisions 1, and 2 and 4 hours for Division 3 when the battery is subjected to a battery service test, or
 2. The battery capacity is adequate to supply a dummy load of the following profile while maintaining the battery terminal voltage greater than or equal to 21 volts for the 24-volt battery, 105 volts for the 125-volt battery, and 205 volts for the 250-volt battery, and 105 volts for the 125 v. DIV 3 battery.

BATT SYS	Time	0-3sec	3-13sec	13-30sec	30-60sec	1-60 min	1-2 hr
24V		17	17	17	17	17	17
125V DIV 1		671	252	237	153	86	86
125V DIV 2		426	224	209	125	99	99
125V DIV 3		73.4 66	73.4 66	73.4 66	73.4 66	73.4 66	73.4 66
250V		1462	567	567	567	432	396

during the remainder of the 8-hour test for Divisions 1, 2 and 3 add 4 hours for Division 3.

- e. At least once per 60 months during shutdown by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. At this once per 60-month interval, this performance discharge test may be performed in lieu of the battery service test.
- f. At least once per 18 months during shutdown performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity from its average on previous performance tests, or is below 90% of the manufacturer's rating.

Note: All other changes with the exception of clouded areas submitted under 602-84-126, dated March 9, 1984

ELECTRICAL POWER SYSTEMS

3/4.8.4 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

A.C. CIRCUITS INSIDE PRIMARY CONTAINMENT

LIMITING CONDITION FOR OPERATION

3.8.4.1 At least the following A.C. circuits inside primary containment shall be deenergized*:

- a. Circuits supplied by breakers 2AR and ~~2CR~~ MCC E-MC-8C.
- b. Circuits supplied by panel E-LP-6BAG.
- c. Circuits supplied by panel E-LP-3DAG.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With any of the above required circuits energized, trip the associated circuit breaker(s) in the specified panel(s) within 1 hour.

SURVEILLANCE REQUIREMENTS

4.8.4.1 Each of the above required A.C. circuits shall be determined to be deenergized at least once per 24 hours** by verifying that the associated circuit breakers are in the tripped condition.

- d. Circuits supplied by breakers ~~2BL~~, 1D, ~~MC-3DA~~ ^{in cubicles} and ^{of} ~~2CR~~ ^{3DA}.

Note sentence should read: Circuits supplied by breakers in cubicles 2BL, 1D and 2CR of MC-3DA.

*Except during entry into the drywell.

**Except at least once per 31 days if locked, sealed, or otherwise secured in the tripped condition.



卷之四

●

2. 4. 1991

REFUELING OPERATIONS

3/4.9.2 INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

Notes: Clouded area denotes change.
Previous changes submitted under G02-84-032, dated January 20, 1984.

3.9.2 At least 2 source range monitor* (SRM) channels shall be OPERABLE and inserted to the normal operating level with:

- a. Continuous visual indication in the control room,
- b. At least one with ^{Alarm} ~~audible indication~~ in the control room,
- c. One of the required SRM detectors located in the quadrant where CORE ALTERATIONS are being performed and the other required SRM detector located in an adjacent quadrant, and
- d. The "shorting links" removed from the RPS circuitry prior to and during the time any control rod is withdrawn[#] and shutdown margin demonstrations are in progress.

APPLICABILITY: OPERATIONAL CONDITION 5.

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS** and insert all insertable control rods.

SURVEILLANCE REQUIREMENTS

4.9.2 Each of the above required SRM channels shall be demonstrated OPERABLE by:

- a. At least once per 12 hours:
 1. Performance of a CHANNEL CHECK,
 2. Verifying the detectors are inserted to the normal operating level, and
 3. During CORE ALTERATIONS, verifying that the detector of an OPERABLE SRM channel is located in the core quadrant where CORE ALTERATIONS are being performed and another is located in an adjacent quadrant.

*The use of special movable detectors during CORE ALTERATIONS in place of the normal SRM nuclear detectors is permissible as long as these special detectors are connected to the normal SRM circuits.

**Except movement of IRM, SRM or special movable detectors.

[#]Not required for control rods removed per Specification 3.9.10.1 and 3.9.10.2.

