REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:8401310376 DOC.DATE: 84/01/24 NOTARIZED: NO may DOCKET #
FACIL:50-397 WPPSS Nuclear Project, Unit 2, Washington Public Powe 05000397
AUTH.NAME AUTHOR AFFILIATION
SORENSEN,G.C. Washington Public Power Supply System
RECIP.NAME RECIPIENT AFFILIATION
SCHWENCER,A. Licensing Branch 2

SUBJECT: Advises that plant mods re final as-built stress reconcilation will be completed by 840131,per util 830616 & 0328 ltrs re hydrodynamic loads outside containment.Response resolves NUREG-0892,SER,Confirmatory Item 4.

DISTRIBUTION CODE: B001S COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 3
TITLE: Licensing Submittal: PSAR/FSAR Amdts & Related Correspondence

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL
	NRR/DL/ADL	1	0	NRR LB2 BC	1	0
	NRR LB2 LA	1	0	AULUCK,R. 01	1	1
INTERNAL:	ELD/HDS2	1	0	IE FILE	1	1
	IE/DEPER/EPB 36	3	3	IE/DEPER/IRB 35	1	1
	IE/DQASIP/QAB21	1	1	NRR/DE/AEAB	1	0
	NRR/DE/CEB 11	1	1	NRR/DE/EHEB	1	1
	NRR/DE/EQB 13	2	2	NRR/DE/GB 28	2	2
	NRR/DE/MEB 18	1	1	NRR/DE/MTEB 17	1	1
	NRR/DE/SAB 24	1	1	NRR/DE/SGEB 25	1	1
	NRR/DHFS/HFEB40	1	1	NRR/DHFS/LQB 32	1	1
	NRR/DHFS/PSRB	1	1	NRR/DL/SSPB	1	0
	NRR/DSI/AEB 26	1	1	NRR/DSI/ASB	1	1
	NRR/DSI/CPB 10	1	1	NRR/DSI/CSB 09	1	1
	NRR/DSI/ICSB 16	1	1	NRR/DSI/METB 12	1	1
	NRR/DSI/PSB 19	1	1	NRR/DSI/RAB 22	1	1
	NRR/DSI/RSB 23	1	1	REG FILE 04	1	1
	RGN5	3	3	~~RM/DDAMI/MIB~~	1	0
EXTERNAL:	ACRS 41	6	6	BNL(AMDTS ONLY)	1	1
	DMB/DSS (AMDTS)	1	1	FEMA-REP DIV 39	1	1
	LPDR 03	1	1	NRC PDR 02	1	1
	NSIC 05	1	1	NTIS	1	1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 53 ENCL 46

[illegible]

[illegible]

[illegible]

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

January 24, 1984
G02-84-038

Docket No. 50-397

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Schwencer:

Subject: NUCLEAR PROJECT NO. 2
DESIGN ASSESSMENT FOR HYDRODYNAMIC LOADS

References:
1) Letter, G02-83-530, G. D. Bouchey (SS) to A. Schwencer (NRC), Hydrodynamic Loads Outside Containment, dated June 16, 1983
2) Letter, G02-83-260, G. D. Bouchey (SS) to A. Schwencer (NRC), Hydrodynamic Loads Outside Containment, dated March 28, 1983

The attached clarification was requested by Mr. R. Auluck and Ms. R. Li (NRC) in a phone conversation on August 4, 1983 with Mr. P. Powell (SS) concerning the basis for design assessment of safety-related equipment and piping for hydrodynamic loads, inside and outside containment. The initial question response (provided in Reference 1) has been modified to include that changes in the design have been implemented as a result of the consideration of SRV discharge and LOCA hydrodynamic loads. The assessment concludes that all the safety-related piping and equipment and their supports, inside containment and outside containment to the first anchor, are capable of withstanding the effects of hydrodynamic loads resulting from SRV actuation and postulated LOCA events in conjunction with other applicable loads. All plant modifications relating to the final as-built stress reconciliation are scheduled for completion by

8001
1/1

January 31, 1984. These modifications will be tracked to completion per the Master Completion List, item 3, attachment 1 to the WNP-2 Operating License No. NPF-21. This response resolves NUREG-0892, Safety Evaluation Report, Confirmatory Item 4 (Licensing Condition 23, of the WNP-2 Operating License No. NPF-21).

Should you have any questions, please contact Mr. P. L. Powell, Manager WNP-2 Licensing.

Very truly yours,

G. C. Sorensen, Manager
Regulatory Programs

PLP/tmh
Attachment

cc: R Auluck - NRC
WS Chin - BPA
AD Toth - NRC Site

NRC QUESTION:

Describe the basis for design assessment of safety-related equipment and piping, for hydrodynamic loads, inside and outside containment.

RESPONSE:

The detailed assessment is complete for the equipment and piping inside containment and outside primary containment up to the first anchor. Assessment of such material inside the wetwell is described in the Plant Design Assessment Report, Revision 3. The assessment of the equipment and piping inside the drywell is reported in the Final Safety Analysis Report, Section 3.9. Changes in the design have been implemented as a result of the consideration of SRV discharge and LOCA hydrodynamic loads. The assessment concludes that all the safety-related piping and equipment and their supports, inside containment and outside containment to the first anchor, are capable of withstanding the effects of hydrodynamic loads resulting from SRV actuation and postulated LOCA events in conjunction with other applicable loads.

For equipment and piping outside containment beyond the first anchor, the report transmitted by Reference 2 demonstrated that the acceleration response spectra for the original seismic loads effectively equal or exceed the acceleration response spectra due to comparable combinations of new seismic and hydrodynamic loads. Thus, the loadings for the design of the equipment and piping was at all frequencies of interest equal to or greater than the comparable loadings due to combinations of new seismic and hydrodynamic loads. Consequently, the use of the original seismic loads as the basis for the design assessment of safety-related equipment and piping outside containment is justified.

B 43