

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Powe 05000397
 AUTH. NAME: AUTHORITY AFFILIATION
 BOUCHEY, G.D. Washington Public Power Supply System
 RECIP. NAME: RECIPIENT AFFILIATION
 SCHWENCER, A. Licensing Branch 2

SUBJECT: Recommends deletion of all refs to RHR steam condensing sys
 from encl Interim Deficiency Rept 216 re RHR relief valve
 vents. Steam condensing mode of RHR sys will be deactivated.

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Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way • Richland, Washington 99352 (509) 372-5000

June 22, 1983
G02-83-553

Docket No. 50-397

Director of Nuclear Reactor Regulation
Attention: Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Schwencer:

Subject: NUCLEAR PROJECT NO. 2
DRAFT TECHNICAL SPECIFICATIONS

Reference: Letter, G02-83-436, C. S. Carlisle (SS) to
J. B. Martin (NRC Region V), "Nuclear Project
No. 2, 10CFR50.55(e) Potentially Reportable
Conditions #216, RHR Relief Valve Vents; #213,
Standby Liquid Control System (SLCS); and #232
LPCS and RHR Support Pin Receptacles, dated
May 18, 1983. (Applicable sections attached.)

The referenced letter provided an interim report on three 10CFR50.55(e) potentially reportable items. Item one, Condition #216 concerned a potential open leakage path (given a failure in the RCIC System) from the primary to the secondary containment through four relief valves in the RHR Steam Condensing System. The action decided upon by the project is to deactivate the steam condensing mode of the RHR System.

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PDR ADDCK 05000397
S PDR

Boo!
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A. Schwencer
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June 22, 1983
DRAFT TECHNICAL SPECIFICATIONS

The purpose of this letter is to provide Mr. D. Hoffman, NRC/SSPB with the recommendation to delete from the subject document all references to the RHR Steam Condensing System. Limiting Condition for Operation (LCO) 3.3.2, Tables 3.3.2-1 through -3, and 4.3.2.1-1, item 5 specifically, should be deleted.

For any additional information, please contact either Mr. M. R. Wuestefeld on (509) 377-2501 extention 2843 or Mr. P. Powell on extention 2298.

Very truly yours,



G. D. Bouchey
Manager, Nuclear Safety and Regulatory Programs

MRW/PLP/tmh
Enclosures

cc: R Auluck - NRC
WS Chin - BPA
A Toth - NRC Site
D Hoffman - NRC

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June 22, 1983
G02-83-553



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4 pp *PLC*

6-23-83

AUTHOR: MR Wuestefeld/PL2/Powell			FOR SIGNATURE OF: GD Bouchey		
SECTION					
FOR APPROVAL OF	RM Nelson	CM Powers	LT Harrold	BA Holmberg	JD Martin
APPROVED	<i>[Signature]</i>	<i>[Signature]</i>	<i>[Signature]</i>	<i>[Signature]</i>	<i>[Signature]</i>
DATE			GC Sorensen	6/17/83	

A. Schwencer
Page Two
June 22, 1983
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Very truly yours,



G. D. Bouchey
Manager, Nuclear Safety and Regulatory Programs

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cc: R Auluck - NRC
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D Hoffman - NRC:

p. Follow steps n through s of 5.4.6.2.5.1.

5.4.6.2.5.3 Steam Condensing (Hot Standby) Operation

The steam condensing mode of RHR for WNP-2 has been deactivated. However, the major pieces of equipment are installed with the exception of the steam supply relief valves, RHR-RV-55A and B and RHR-RV-95A and B, and are shown on the RCIC and RHR P&ID's (Figures 5.4-9a and b and 5.4-13a and b, respectively). That equipment which is dedicated to the steam condensing mode will be identified on these figures with reference to the notes identifying this portion as being deactivated. WNP-2 operating procedures have been revised to reflect this deactivation. Deletion of this mode of operation for RCIC and RHR will not adversely affect either systems' capability to bring the reactor to cold shutdown should it be required.

5.4.6.2.5.4 Limiting Single Failure

The most limiting single failure in the combined function of RCIC and HPCS systems is the failure of HPCS. If the capacity of RCIC System is adequate to maintain reactor water level, the operator follows 5.4.6.2.5.1. If however, the RCIC capacity is inadequate 5.4.6.2.5.1 applies, but additionally the operator may also initiate the ADS system described in 6.3.2.

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or unusual difficulties without a compensating increase in the level of quality and safety. Based on the granting of relief from these preservice examination requirements, the staff concludes that the preservice inspection program for WNP-2 is in compliance with 10 CFR 50.55a(g)(2). The staff's detailed evaluation supporting this conclusion is in Appendix K to this report.

The initial inservice inspection program has not been submitted. This program will be evaluated after the applicable ASME Code edition and addenda can be determined based on 10 CFR 50.55a(b), but before the first refueling outage when inservice inspection begins.

5.4 Component and Subsystem Design

5.4.2 Residual Heat Removal System

In a letter dated June 22, 1983, the applicant stated the intention to deactivate the steam condensing mode of the residual heat removal (RHR) system at WNP-2. The applicant also requested that specific Technical Specifications that refer to the steam condensing mode of operation be deleted. The WNP-2 steam condensing mode of the RHR is not safety related and no credit is taken for this mode of operation in any safety analyses. Furthermore, deactivation will not impede the safety function of other safety-related modes of the RHR system. Deactivation of the steam condensing mode is therefore acceptable to the staff.

The Technical Specifications related to the steam condensing mode of operation include limiting condition for operation 3.3.2. Tables 3.3.2-1, 3.3.2-2, 3.3.2-3, and 4.3.2.1-1 (item 5 of each table). These specifications are related to isolation actuation instrumentation that operate valve group 12, which consists of only one valve, E51-F064. Deactivation of the steam condensing mode involves locking closed and removing power from valve E51-F064. This obviates the need for automatic isolation and, therefore, deleting these specifications is acceptable.