

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8202180246 DOC. DATE: 82/02/02 NOTARIZED: NO DOCKET #
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Power 05000397
 AUTH. NAME AUTHOR AFFILIATION
 BOUCHEY, G.D. Washington Public Power Supply System
 RECIP. NAME RECIPIENT AFFILIATION
 SCHWENCER, A. Licensing Branch 2

SUBJECT: Submits additional information re suppression pool temp transient analysis. Reactor decay heat would be transferred to suppression pool in event of single failure.

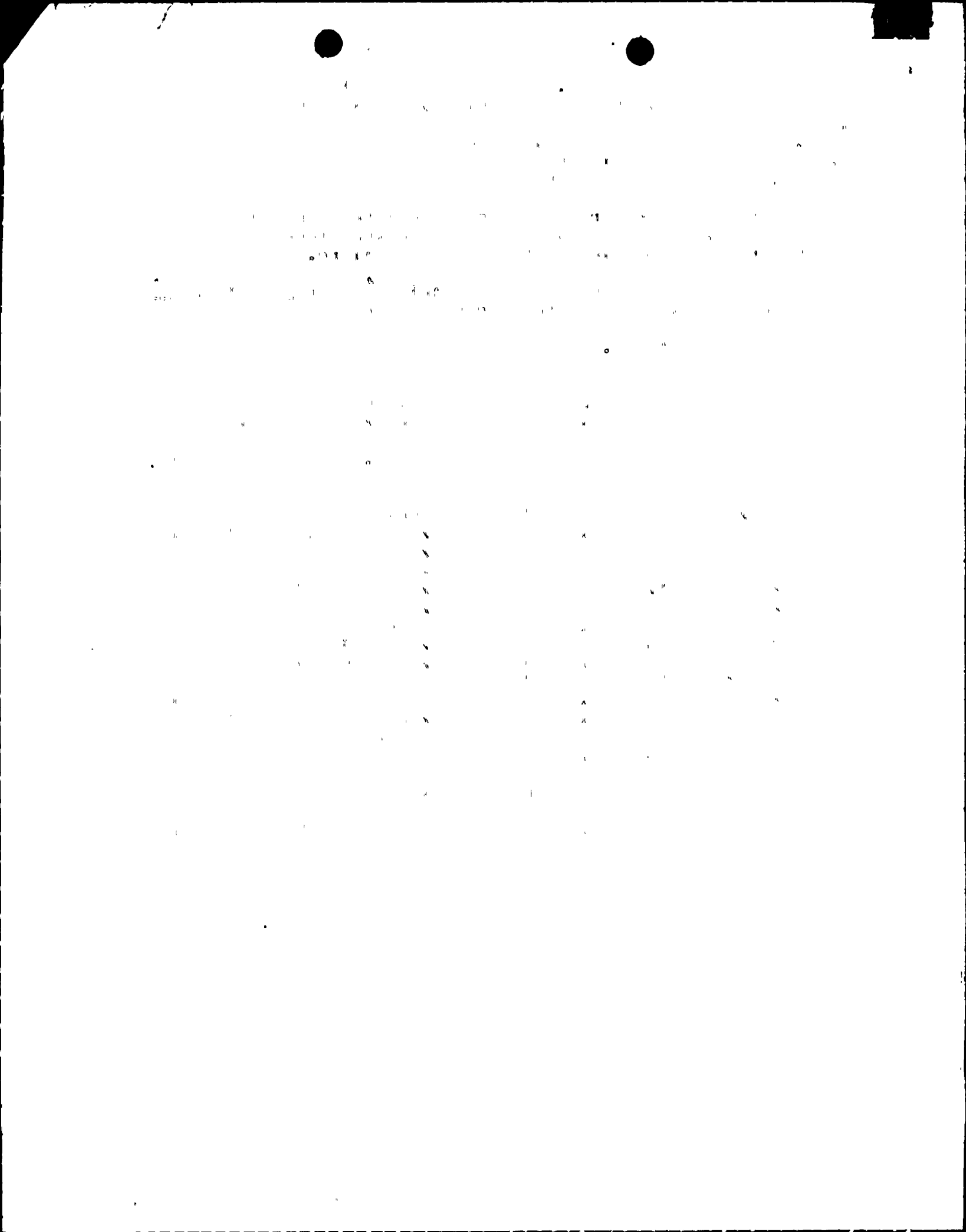
DISTRIBUTION CODE: B001S COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2
 TITLE: PSAR/FSAR AMDTS and Related Correspondence

NOTES: 2 copies all matl: PM.

05000397

ACTION:	RECIPIENT ID CODE/NAME	COPIES		RECIPIENT ID CODE/NAME	COPIES	
		LTTR	ENCL		LTTR	ENCL
ACTION:	A/D LICENSING	1	0	LIC BR #2 BC	1	0
	LIC BR #2 LA	1	0	AULUCK, R. 01	1	0
INTERNAL:	ELD	1	0	IE	06	3
	IE/DEP/EPDB 35	1	1	IE/DEP/EPLB 36	3	3
	MPA	1	0	NRR/DE/CEB 11	1	1
	NRR/DE/EQB 13	3	3	NRR/DE/GB 28	2	2
	NRR/DE/HGEB 30	2	2	NRR/DE/MEB 18	1	1
	NRR/DE/MTEB 17	1	1	NRR/DE/QAB 21	1	1
	NRR/DE/SAB 24	1	1	NRR/DE/SEB 25	1	1
	NRR/DHFS/HFEB40	1	1	NRR/DHFS/LGB 32	1	1
	NRR/DHFS/OLB 34	1	1	NRR/DHFS/PTRB20	1	1
	NRR/DSI/AEB 26	1	1	NRR/DSI/ASB 27	1	1
	NRR/DSI/CPB 10	1	1	NRR/DSI/CSB 09	1	1
	NRR/DSI/ETSB 12	1	1	NRR/DSI/ICSB 16	1	1
	NRR/DSI/PSB 19	1	1	NRR/DSI/RAB 22	1	1
	NRR/DSI/RSB 23	1	1	NRR/DST/LGB 33	1	1
	REG FILE 04	1	1			
	EXTERNAL:	ACRS 41	16	16	BNL (AMDTS ONLY)	1
FEMA-REP DIV 39		1	1	LPDR 03	1	1
NRC PDR 02		1	1	NSIC 05	1	1
NTIS		1	1			

TOTAL NUMBER OF COPIES REQUIRED: LTR 66 ENCL 0



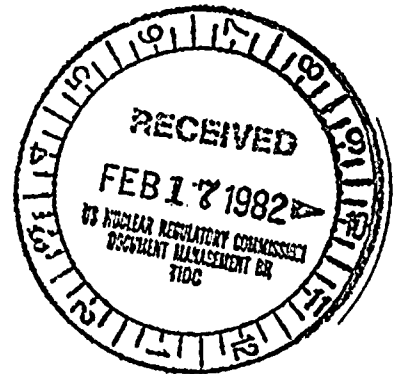
Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

February 2, 1982
G02-82-138

Docket No. 50-397

Mr. A. Schwencer, Chief
Licensing Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Dear Mr. Schwencer:

Subject: NUCLEAR PROJECT NO. 2
SUPPRESSION POOL TEMPERATURE TRANSIENT
ANALYSIS AND IN-PLANT SRV TEST

Reference: Letter, G02-81-524, GD Bouchey to A Schwencer,
dated December 15, 1981

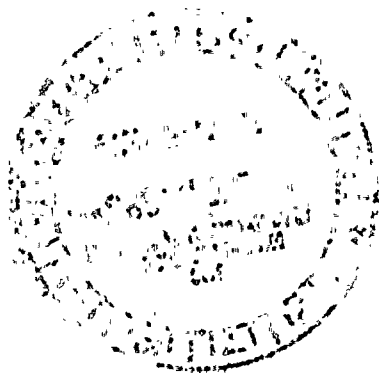
The reference letter transmitted the results of the suppression pool temperature transient analysis for WNP-2 and presented the basis for our conclusion that the scope of the in-plant SRV test for WNP-2 should be limited to measurement of pool temperature distribution. Subsequently, we were asked to provide additional information on these two (2) issues, as indicated below:

- (a) "Explain why a single failure will not disable both the RHR Shutdown cooling function and one (1) RHR loop in the suppression pool cooling mode"

Response:

As discussed in the report transmitted with the reference, loss of the shutdown cooling mode and one RHR heat exchanger is assumed for cases 1a, 2a and 3c. This reflects an electrical division failure which disables either the inboard or outboard containment isolation valve on the RHR shutdown cooling line, and one of the two RHR Heat exchangers. In this situation, WNP-2 would use the alternate shutdown cooling path whereby reactor decay heat would be transferred to the suppression pool through the main steam relief valves, and then to the ultimate heat sink (spray ponds) through the RHR heat exchangers using the available loop of the suppression pool cooling mode of the RHR system. This is discussed in more detail in FSAR Section 15.2.9 and in Appendix I, LRG Issue RSB-8.

Boal
5/10



[Faint, illegible text or stamp at the bottom right corner.]

Mr. A. Schwencer
Page Two
February 2, 1982
G02-82-138

(b) "Compare the distance between quenchers and pool bottom at WNP-2 versus Caorso"

Response:

In Caorso, the distance between the bottom of the quencher arms and the surface of the flat bottom of the containment is approximately 4'-4 3/4".

In WNP-2, the outer row of quenchers is positioned above the sloping pool bottom. The shortest distance between the pool bottom and the outward-pointing quencher arms, measured perpendicular to the surface of the sloping pool bottom, ranges from 4'-5 1/2" to 4'-6 1/2".

The average distance from quencher arm to pool bottom for the outer row of quenchers, calculated by integrating over the length of the inward-pointing and outward-pointing quencher arms, would be much greater than this due to the sloping pool bottom. In conclusion, at no point in the WNP-2 plant is the quencher arm closer to the pool bottom than at Caorso and the average quencher arm to pool bottom distance is much larger than at Caorso. Also, as pointed out in the reference, the outer row of quenchers is further from the containment wall than at Caorso. Therefore, since bubble pressure attenuates with distance, using Caorso test pressures applied directly to the WNP-2 containment walls and pool bottom is conservative.

Very truly yours,

G. D. Bouchey

G. D. Bouchey
Deputy Director, Safety and Security

EAF/jca

cc: R Auluck - NRC
WS Chin - BPA
F Eltawila-NRC
R Feil - NRC Site

