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ACCESSION NBR: 8106020353 DOC. DATE: 81/05/26 NOTARIZED: NO DOCKET #  
 FACIL: 50-397 WPPSS Nuclear Project, Unit 2, Washington Public Power 05000397  
 AUTH. NAME: AUTHOR AFFILIATION:  
 MATLOCK, R.G. Washington Public Power Supply System  
 RECIP. NAME: RECIPIENT AFFILIATION:  
 ENGELKEN, R.H. Region 5, San Francisco, Office of the Director (81/03/01)

SUBJECT: Interim deficiency rept. re annulus pressurization on piping  
 sys attached to or supported from reactor vessel or  
 sacrificial shield wall, initially reported 810122. Deficiency  
 found reportable. Final rept. by June 1982.

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 TITLE: Construction Deficiency Report (10CFR50.55E)

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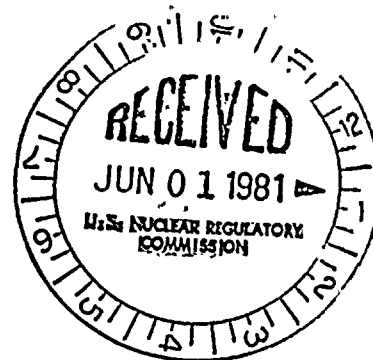
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## Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

May 26, 1981  
G02-81-119



Docket No. 50-397

Mr. R. H. Engelken  
Office of Inspection of Enforcement  
Nuclear Regulatory Commission  
Region V  
Suite 202, Walnut Creek Plaza  
2990 North California Blvd.  
Walnut Creek, California 94596

Dear Mr. Engelken:

Subject: WPPSS NUCLEAR PROJECT NO. 2  
POTENTIALLY REPORTABLE DEFICIENCY - 10CFR50.55(e)  
ANNULUS PRESSURIZATION EFFECT ON PIPING SYSTEMS

Reference: Letter No. G02-81-13, dated January 22, 1981,  
RG Matlock, The Supply System, to RH Engelken,  
NRC Region V

The Reference letter informed your staff of a potentially reportable deficiency regarding the effect of annulus pressurization on piping systems attached to or supported from the reactor vessel or sacrificial shield wall.

Although the analysis of resulting loads on piping systems and supports is not completed at this time, the Supply System has concluded that this condition is a reportable deficiency under the terms of 10CFR50.55(e), since it is a safety concern requiring extensive analysis which should have been, but was not, considered in the design of safety-related piping and supports prior to release for construction.

Final results of the ongoing analysis of piping systems and supports, by the architect-engineer, will be included in the "Plant Design Assessment Report for SRV and LOCA Loads", for the WNP-2 facility. It is anticipated that the final revision to the report will be submitted in June 1982. Since this matter was initially raised by the NRC staff in conjunction with their ongoing reviews on the WNP-2 docket (ref. WNP-2 FSAR, question 110.019) the

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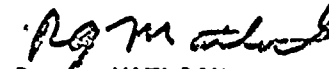


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Supply System will address this concern in that forum, and does not anticipate providing a duplicate or supplementary report to NRC Region V.

Very truly yours,



R. G. MATLOCK  
Program Director  
WNP-2

EAF:dm

cc: V. Stello - NRC  
AD Toth - NRC Resident Inspector  
RC Root - B&R Site  
TA Mangelsdorf - Bechtel Site  
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WNP-2 Files

