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 MECREY, R. C. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 VISSING, G. S.

SUBJECT: Submits response to NRC Administrative Ltr 95-03, rev 2,
 "Availability of Reactor Vessel Integrity Database, Version,"
 dtd 990726.

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ROBERT C. MECREDY
Vice President
Nuclear Operations

August 31, 1999



U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Response to NRC Administrative Letter 95-03, Revision 2, "Availability of the
Reactor Vessel Integrity Database, Version 2", July 26, 1999

- References:
1. Letter from R, C, Mecredy (RG&E) to Guy S. Vissing (NRC), "RAI Related to
Reactor Pressure Vessel Integrity", August 25, 1998.
 2. Letter from G. S. Vissing (NRC) to R. C. Mecredy (RG&E), July 6, 1999.
 3. NRC Administrative Letter 95-03, Revision 2, July 26, 1999.

Dear Mr. Vissing:

References 2 & 3 mentioned the availability of the subject database on the NRC website
and gave access instructions. Rochester Gas & Electric (RG&E) retrieved and reviewed the
database, particularly as it pertains to the R. E. Ginna Nuclear Power Plant.

Reviewing the database, we found that for the Ginna reactor vessel (RV) limiting material, which
is Heat ID 61782, Circ. Weld, SA-847, the calculated values of several parameters derived from
information submitted in Reference 1 had not been incorporated in the new version 2 of the
database. These parameters are listed below:

Parameter	RVID Ver.2	Current Value (per Ref. 1)
1. Chemistry Factor	158.7 F	160.7 F
2. Delta RTndt @ EOL	210.4 F	213.08 F
3. RTpts @ EOL	253.9 F	256.58 F

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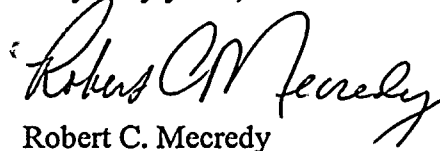
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These parameters are entered in two parts of the RVID, the Weld Detail Data Page and the PTS Summary Report Page.

Data for other welds, forgings, and plate materials are correct as we found no other discrepancies. We request that in your next revision of the RVID, the above parameters be updated to reflect the new data that we submitted in Reference 1, as shown above. This is particularly important since the data pertains to the Ginna RV limiting weld which has been utilized to generate Ginna heat up & cooldown curves up to 40 EFPY (Effective Full Power Years).

We sincerely appreciate and offer our thanks for your consideration of this request.

Very truly yours,


Robert C. Mecredy

xc: Mr. Guy S. Vissing (Mail Stop 8C2)
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U.S. Nuclear Regulatory Commission
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U.S. NRC Ginna Senior Resident Inspector

Andrea Lee