

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9904290036 DOC.DATE: 99/04/19 NOTARIZED: NO DOCKET #
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 AUTH.NAME AUTHOR AFFILIATION
 MECREDY,R.C. Rochester Gas & Electric Corp. *See Report*
 RECIP.NAME RECIPIENT AFFILIATION
 VISSING,G.S.

SUBJECT: Forwards rev 0 & rev 1 to COLR,Cycle 28 for RE Ginna NPP,
 per TS 5.6.5.

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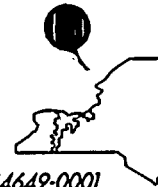
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ROBERT C. MECREDY
Vice President
Nuclear Operations

April 19, 1999

U.S. Nuclear Regulatory Commission
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Attn: Guy S. Vissing
Project Directorate I-1
Washington, DC 20555

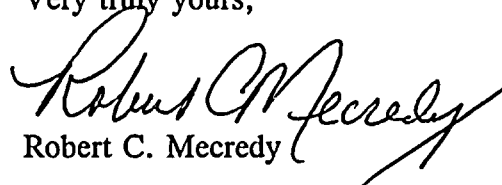
Subject: Transmittal of Core Operating Limits Report (COLR)
Rochester Gas & Electric Corporation
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref.(a): Technical Specification 5.6.5

Dear Mr. Vissing:

Ginna Station Technical Specification 5.6.5 requires that RG&E provide the NRC with any revisions of the COLR. As such, attached please find a copy of Revision 0 and Revision 1 of this document for Cycle 28. Revision 0 of the COLR was approved to support the loading of fuel for Cycle 28 and Revision 1 incorporated the effects of a revised steam line break analysis for power operation. Future revisions to this document will be forwarded to the NRC in accordance with the applicable technical specification.

Very truly yours,


Robert C. Mecredy

Attachment

xc: Mr. Guy Vissing (Mail Stop 14B2)
Project Directorate I-1
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Ginna Senior Resident Inspector

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CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 MECREDY,R.C. Rochester Gas & Electric Corp.
 RECIP.NAME RECIPIENT AFFILIATION
 VISSING,G.S.

*See
Reports*

SUBJECT: Forwards response to all NRC questions & comments related to
 earlier flooding analysis,primarily based on rev 1 to "GNPP
 Internal Flooding Probabilistic Safety Assessment Final
 Rept," encl.

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