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SUBJECT: Submits response to 980519 telcon discussion between util,
 licensee & NRC on util spent fuel pool rerack safety
 evaluation.

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ROBERT C. MECREDDY
Vice President
Nuclear Operations

May 22, 1998

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate 1-1
Washington, D.C. 2055

Subject: Response to Request for Additional Information on the Structural Aspects of
the Spent Fuel Pool Rerack
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

This letter is in response to a May 19, 1998 telephone discussion between RG&E, Framatome, and the NRC Staff on Rochester Gas and Electric's Spent Fuel Pool Rerack structural safety evaluation.

In the original spent fuel pool design, prior to the 1985 rerack, the spent fuel racks were supported laterally on the pool wall. Those supports were removed during the 1985 rerack. The licensing basis of the proposed spent fuel pool design is such that the six remaining racks, as well as the seven new ATEA storage racks, are freestanding, and are therefore vertically supported on the pool floor only. The large gaps between the racks and the pool wall are designed such that the racks do not impose any impact loading on the pool wall. These conditions were verified throughout the current analysis. Because of the removal of the supports, the racks only impose hydrodynamic loads on the pool wall during a seismic event. These hydrodynamic loads are much lower than the lateral support loads produced in the original design. The new spent fuel storage design is consistent with the licensing basis established during the 1985 rerack.

The new racks are high-density storage racks and are capable of storing additional fuel. The number of support legs is designed such that the new racks do not impose any higher loading on the reinforced concrete pool floor than the current design. The spent fuel pool floor is 3.0 feet thick reinforced concrete and rests on solid bedrock. Section 3.5.3.1.9.1.2 of the Licensing Report (Reference 3.44) provides the analysis and results for the loading on the floor. The results, Table 3.5-130 of the Licensing Report, shows a large margin against the allowable concrete bearing stresses of ACI 349-85 (Reference 3.20).

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References (Numbers are kept the same as in the Licensing Report dated February 1997):

- 3.20 ACI 349-85, "Code Requirements for Nuclear Safety Related Concrete Structures," American Concrete Institute, 1985.
- 3.24 NRC Letter to RG&E - Mr. Kober, dated November 14, 1984. Safety Evaluation Report to Amendment No. 65, "Increase of the Spent Fuel Storage Capacity," License No. DPR-18, Docket No. 50-244.
- 3.44 Letter from RG&E to U. S. NRC, dated March 31, 1997, Application for Amendment to Facility Operating License, Revised Spent Fuel Pool Storage Requirements, RG&E, R.E.Ginna Nuclear Power Plant, Docket No. 50-244.

Very truly yours,


Robert C. Mecredy

RCM/mab

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