

CATEGORY 1

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 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 MECREDY, R.C. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 VISSING, G.S.

SUBJECT: Informs that util will change insp frequency of thimble tube performance based on results of 1997 refueling outage insp. Licensee stated in 900601 ltr to test once every two years, in response to NRC Bulletin 88-009.

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ROBERT C. MECREDY
Vice President
Nuclear Operations

March 11, 1998

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: NRC Bulletin 88-09: Thimble Tube Thinning in Westinghouse Reactors, dated July 26, 1988
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

- Reference(s): (a) Letter from R.C. Mecredy (RG&E) to A.R. Johnson (NRC), dated June 6, 1989, Subject: NRC Bulletin 88-09
- (b) Letter from R.C. Mecredy (RG&E) to A.R. Johnson (NRC), dated June 1, 1990, Subject: NRC Bulletin 88-09
- (c) Letter from R.C. Mecredy (RG&E) to A.R. Johnson (NRC), dated April 8, 1993, Subject: NRC Bulletin 88-09

Dear Mr. Vissing:

By Reference (a), Rochester Gas and Electric Corp. (RG&E) committed to conduct three inspections of thimble tube wear rate and then establish an appropriate inspection frequency to monitor thimble tube performance. By Reference (b), RG&E notified the NRC that a testing frequency of once every other refueling outage (i.e., once every two years) is appropriate for thimble tube inspections. RG&E included the technical justification and acceptance criterion for this frequency. RG&E also stated that the NRC will be notified upon any change in this frequency.

By Reference (c), RG&E notified the NRC of a change in inspection frequency and provided the technical justification and acceptance criterion. RG&E also stated that the NRC will be informed of any such future changes in inspection frequency.

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During the 1997 refueling outage, an inspection of incore monitoring system thimble tubes was conducted. Based on the results of this inspection, RG&E will change the inspection frequency. The 1997 inspection identified:

- o The same wear indications at the Lower Core Plate with no significant growth noted since the previous inspection
- o A slight change in wall thinning for tube G-6 at the same location where wear had been previously recorded. Corrective action for this thimble tube was taken by repositioning in 1990.
- o Growth in the indications of degradation that reside outside the reactor vessel in tubes E-2, B-6, and A-8. These indications are not indicative of support plate wear. Corrective action was taken on Tube E-2, since it exceeded 65%.

Since tube E-2, B-6 and A-8 indications are not indicative of support plate wear, a different methodology, applied in a conservative manner, was used to size the indications for these two tubes. NRC reporting requirements were reviewed, and the results of the 1997 inspections are not considered reportable under 10 CFR 50.72 or 10 CFR 50.73.

Based on these inspection results, RG&E has determined that the inspection frequency and acceptance criterion will be:

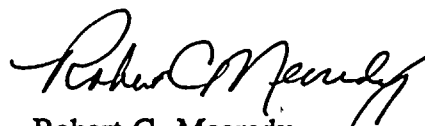
- o previous indication 10% to less than 45% - every third refueling outage (approximately once every 4 1/2 years)
- o previous indication 45% to less than 55% - every other refueling outage (approximately once every three years)
- o previous indication 55% or greater - perform corrective action, if support plate wear is the suspected cause. For other indications, corrective action will be taken at 65% or greater. Future inspection frequency will be every other or every third outage, as stated above.
- o previous inspection never exceed 10% through-wall - no specified frequency. Future inspections will be based on RG&E periodic assessment.

Initial justification for inspection frequencies and acceptance criterion is discussed in detail in Reference (c), and remains unchanged for the inspection frequency. If support plate wear is not the phenomenon, then 65% is an appropriate threshold because wear between inspections need not be accounted for. This still leaves margin to the true minimum criteria of 75% through-wall.

In addition, RG&E is evaluating possible replacement of selected tubes during the 1999 outage. During the 1999 outage, an eddy current inspection of selected tubes will be performed. If tubes are replaced, they will be examined as well, to obtain baseline information.

If additional information is desired, please contact Mr. Peter Bamford at Ginna Station.

Very truly yours,



Robert C. Mecredy

xc: Mr. Guy S. Vissing (Mail Stop 14B2)
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