EOP:	TITLE:	REV: 7
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	PAGE 1 of 6

ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER 23

R E Sidelinger

RESPONSIBLE MANAGER

8-5-97

EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY: _____

A. PURPOSE - This procedure provides guidance necessary to operate the plant with indication of high reactor coolant activity.

B. ENTRY CONDITIONS/SYMPTOMS

1. SYMPTOMS - The symptoms of HIGH REACTOR COOLANT ACTIVITY are;

   a. Unexplained increase in letdown line monitor, R-9, or

   b. Sampling indicates I-131 equivalent GREATER THAN 1.0 uCi/gm, or

   c. Sampling indicates gross degassed activity GREATER THAN 20 uCi/gm, or

   d. Sampling indicates that total specific activity exceeds 100/E.

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED

* *

CAUTION
- IF LETDOWN FLOW EXCEEDS 60 GPM, THEN LOCALLY MONITOR D/P ACROSS THE CVCS DI(S) TO VERIFY THAT FLOW IS CONTINUING AND THAT RELIEF VALVE, V-209, HAS NOT LIFTED.
- LETDOWN FLOW THROUGH THE DI'S SHOULD BE LIMITED TO 90 GPM.

* *

NOTE: Conditions should be evaluated for site contingency reporting (Refer to EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION.

1 Verify RCS Activity:

a. Direct RP Tech to sample RCS for activity		
b. RCS activity - GREATER THAN NORMAL (Check with RP Department for normal activity)		b. IF normal activity verified, THEN direct I&C to check operability of R-9, letdown line monitor, AND return to normal operations.

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
2	Increase Letdown Flow To 60 GPM:	
	a. Verify deborating DI isolated - DIVERT VLV CATION DEBOR DI AOV-244 IN BYPASS POSITION	a. Place AOV-244 in bypass position.
	b. Place letdown controllers in MANUAL at 60% open • TCV-130 • PCV-135	
	c. Increase letdown flow as follows: 1) Close letdown orifice valve (AOV-200A or AOV-200B) 2) Immediately open 60 gpm letdown orifice valve, AOV-202	
	d. Adjust low pressure letdown pressure to approximately 250 psig	
	e. Place TCV-130 in AUTO at 105°F	
	f. Place PCV-135 in AUTO at 250 psig	
	g. Adjust charging pump speed and HCV-142 as necessary	

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
3	Check Letdown Line Monitor, R-9 - LESS THAN 200 MR/HR ABOVE BACKGROUND	Evaluate conditions to determine whether local radiation emergency exists (Refer to EPIP 1-13, LOCAL RADIATION EMERGENCY).

* *

CAUTION PLACING A NEW DI IN SERVICE MAY RESULT IN A POSITIVE OR NEGATIVE REACTIVITY ADDITION DUE TO A BORON CHANGE.

* *

4	Direct RP Tech To Sample Letdown DI Efficiency - DECONTAMINATION FACTOR GREATER THAN 10	IF DI efficiency is NOT acceptable, THEN place a new mixed bed in service (Refer to S-3.2B, PLACING A MIXED BED DEMINERALIZER IN SERVICE - BORON CONCENTRATION DIFFERENT THAN RCS).
5	Evaluate AUX BLDG Radiation Levels:	
	a. Direct RP Tech to survey AUX BLDG	
	b. Check AUX BLDG radiation monitors - NORMAL • R-4 • R-9 • R-10B • R-13 • R-14	b. Perform the following: 1) Direct RP Tech to survey AUX BLDG areas as necessary. 2) Evaluate conditions to determine whether local radiation emergency exists (Refer to EPIP 1-13, LOCAL RADIATION EMERGENCY).

STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
6	Determine If Plant Operation Can Continue (Consult Plant staff if necessary) - OPERATION CAN CONTINUE	IF plant shutdown is required, THEN refer to 0-2.1, NORMAL SHUTDOWN TO HOT SHUTDOWN.

NOTE: Refer to 0-9.3, NRC IMMEDIATE NOTIFICATION, for reporting requirements.

7 Notify Higher Supervision

-END-

50-244
6/7/01
Revised
6/29/01

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ADAMS DISTRIBUTION NOTIFICATION.
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Title: Improved Technical Specifications (ITS), Amendment No. 79; ITS Bases Revision 20; and Technical Requirements Manual Revision 17.
Docket Number: 05000244
Document Date: 06/07/2001
Author Name: Harding T L
Author Affiliation: Rochester Gas & Electric Corp

Addressee Name:
Addressee Affiliation:
Document Type: Manual
Availability: Publicly Available
Document Sensitivity: Non-Sensitive
Comment:

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Rochester Gas and Electric Corporation
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June 7, 2001

Subject: Improved Technical Specifications (ITS), Amendment No. 79;
ITS Bases Revision 20; and
Technical Requirements Manual (TRM) Revision 17

To: Distribution

Attached is Amendment No. 79 to the Ginna Station Improved Technical Specifications (ITS). This Amendment revises the ITS requirements for the storage of new and spent fuel within the Spent Fuel Pool. This Amendment was approved by the NRC in a safety evaluation dated December 7, 2000.

In addition to the Amendment, the associated changes to the ITS Bases and the Technical Requirements Manual are included. Instructions for the necessary changes to your controlled copy of the ITS are attached.

These changes are considered effective June 8, 2001.

Please contact Tom Harding (extension 3384) if you have any questions.

Thomas L. Harding

Attachment A

Please replace the following pages of your controlled copy of the ITS as follows:

Volume	Section	Remove	Insert
I	Attachment A to License	First Page	First Page
I	Attachment A to License	LEP-i through LEP-vii	LEP-i through LEP-vii
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II	Chapter 3.7 (LCO)	3.7-31	3.7-31
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III	TRM Chapter 3.7 (Bases)	----	TRB 3.7.7-6

R.E. Ginna Nuclear Power Plant

Technical Requirements Manual
TRM

Revision 16

Responsible Manager: George Wrobel

Effective Date: 6/1/2001
6/1/2001

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3.7 PLANT SYSTEMS

TR 3.7.7 Spent Fuel Pool (SFP) Cooling System

TR 3.7.7 The SFP cooling system shall be maintained as follows:

a. The SFP water temperature shall be ≤ 150°F; and

b. Two SFP cooling systems shall be OPERABLE, each commensurate with the SFP heat load.

- NOTE -

The SFP heat load and SFP cooling capabilities are determined by Nuclear Engineering Services.

APPLICABILITY: Whenever any irradiated fuel assembly is stored in the SFP.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	SFP temperature not within limit.	A.1	Suspend movement of irradiated fuel assemblies from the reactor to the SFP.	Immediately
		AND		
		A.2	Initiate action to restore SFP temperature to within limit.	Immediately
B.	One required SFP cooling system inoperable.	B.1	Suspend movement of irradiated fuel assemblies from the reactor to the SFP.	Immediately
		AND		
		B.2	Restore a second SFP cooling system to OPERABLE status.	14 days

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Both required SFP cooling systems inoperable.	C.1 Perform TSR 3.7.7.1. AND	Once within 1 hour and every 4 hours thereafter
	C.2 Restore one SFP cooling system to OPERABLE status.	Prior to the SFP water temperature exceeding 120°F

SURVEILLANCE REQUIREMENTS.

SURVEILLANCE		FREQUENCY
TSR 3.7.7.1	Verify SFP temperature is within limit.	24 Hours
TSR 3.7.7.2	Verify power available to the two OPERABLE SFP cooling systems.	7 days

B 3.7 PLANT SYSTEMS

TRB 3.7.7 Spent Fuel Pool (SFP) Cooling System

BASES

BACKGROUND

The Spent Fuel Pool (SFP) Cooling System is designed to remove from the SFP the heat generated by stored spent fuel assemblies.

The SFP Cooling System consists of three cooling loops (see Figure TRB 3.7.7-1). The primary loop (loop B) is made up of the SFP Pump B, SFP Heat Exchanger B, and piping. The backup loops include installed loop A with the SFP Pump A, SFP Heat Exchanger A, and piping, and a standby loop with the Skid-Mounted SFP Pump, SFP Standby Heat Exchanger, and hoses. Service water (SW) circulates through the shell while SFP water circulates through the tubes of all three heat exchangers. The SFP Cooling System loop B is sized for 100% of the design SFP heat load. The SFP Cooling System loop A and the standby loop are each capable of removing the normal basis heat load and when operated in parallel can remove more than 100% of the design heat load (Ref. 1). Motor-operated valves provide automatic and remote manual isolation of the SW supply to the heat exchangers associated with the SFP Cooling System loops A and B and the Component Cooling Water Heat Exchangers. These valves close automatically upon coincidence of safety injection and loss of offsite power. Handwheels are provided for manual operation.

The SFP Cooling System is designed to maintain the pool ≤ 120°F during normal refueling conditions and ≤ 150°F during full core discharge operations (Ref. 1). The cooling systems can take a suction from either near the surface of the SFP and/or at a point above the irradiated fuel assemblies, such that a failure of any pipe in the system will not drain the pool to a point where the fuel would be exposed. The cooling system return line to the pool also contains a 0.25 inch vent hole located near the SFP surface level to prevent siphoning. Control board alarms exist with respect to the SFP level and temperature. These features all help to prevent inadvertent draining of the SFP.

APPLICABLE SAFETY ANALYSES

For structural integrity reasons, the pool water temperature is not to exceed 180°F. In order to provide sufficient time to take corrective action in the event of a SFP Cooling System failure, the pool temperature limit is not to exceed 150°F for all modes of operation including a full core discharge. The requirement for two 100% SFP Cooling Systems is based on being able to provide cooling following either a loss of offsite power or an active single failure within the SFP Cooling System.

TR

The SFP water temperature is required to be ≤ 150°F. The specified water temperature provides sufficient margin to the assumptions of the SFP structural integrity. As such, it is the maximum allowed while storing irradiated fuel assemblies within the SFP.

To ensure that the SFP temperature can be maintained within the required limit, two SFP Cooling Systems must be OPERABLE. This requirement provides for 100% backup capability assuming either a loss of offsite power or an active single failure within the SFP Cooling System.

SFP Cooling System loop A and loop B are considered OPERABLE when the respective pump has power available and the respective heat exchanger is capable of providing cooling water with the ability to remove the decay heat load of the irradiated fuel assemblies stored within the SFP. The standby loop is considered OPERABLE when the Skid-Mounted SFP Pump has power connected to a temporary power source, the SFP Standby Heat Exchanger has been staged, and the temporary hoses have been connected and leak checked. During a full core offload the SFP Cooling System loop A and standby loop may be operated in parallel to comprise one of the required OPERABLE systems. Loop A and the standby loop have the ability to utilize fire water for cooling in lieu of SW to provide for increased redundancy. The temporary electrical power source for the standby loop pump may also be varied to provide for increased redundancy.

Also included in the determination of OPERABILITY are all necessary support systems not addressed by this TR (e.g., service water, fire water, electrical). Single active failures are not required to be considered within the support systems for the purpose of this TR; however, the support systems must be capable of performing their support function per the definition of OPERABLE-OPERABILITY in ITS Section 1.1.

The TR is modified by a note. The SFP heat load from the irradiated fuel assemblies stored within the SFP is a variable based on the total number of assemblies stored, the power history of the individual assemblies, and the time since the assemblies were last irradiated. The heat removal capabilities of the individual SFP cooling loops is also a variable based on the temperature and flow rate of the cooling source, SW or fire water. The SFP heat load and the SFP Cooling System heat removal capabilities are determined by Nuclear Engineering Services and provided, as necessary, based on plant conditions.

APPLICABILITY

This TR applies whenever any irradiated fuel assembly is stored in the SFP, to provide sufficient margin to the assumptions of the SFP structural integrity. Specific requirements applicable during a full core discharge are covered by TR 3.9.4.

ACTIONS

A.1 and A.2

If the SFP temperature is not within limit, steps should be taken to preclude the assumptions of the SFP structural integrity from being exceeded. Suspending any operation that would increase SFP decay heat load, such as discharging a fuel assembly from the reactor to the SFP, is a prudent action under this condition. Therefore, actions shall be taken immediately to suspend movement of irradiated fuel assemblies from the core to the SFP.

With the potential for exceeding the assumptions of the SFP structural integrity, corrective actions to restore the SFP temperature to within limit shall be initiated immediately.

B.1 and B.2

With one of the required SFP Cooling Systems inoperable, suspending any operation that would increase SFP decay heat load, such as discharging an irradiated fuel assembly from the reactor to the SFP, is a prudent action under this condition. Therefore, actions shall be taken immediately to suspend movement of irradiated fuel assemblies from the core to the SFP.

With the suspension of fuel discharge into the SFP then a second SFP Cooling System must be restored to OPERABLE status within 14 days. In this condition the remaining OPERABLE SFP Cooling System is adequate to remove the decay heat load. The 14 day Completion Time is adequate to perform typical maintenance activities associated with a SFP Cooling System and takes into account the large heat sink capabilities of the SFP.

C.1 and C.2

If no SFP Cooling System is OPERABLE, there will be no forced cooling of the SFP and as such the SFP temperature must be monitored more frequently until the cooling is restored. This monitoring is accomplished by performing surveillance TSR 3.7.7.1 to verify SFP temperature is within limit. The Completion Time of 1 hour and every 4 hours thereafter is sufficient due to the large heat sink of the SFP and slow heatup rate.

Actions must also be initiated to restore one SFP Cooling System to OPERABLE status prior to the SFP temperature exceeding 120°F. The 120°F is not a safety requirement but is a limit set for normal operation and provides adequate margin to the 150°F limit.

SURVEILLANCE REQUIREMENTS

TSR 3.7.7.1

This TSR verifies that the SFP temperature is within the required limit. The temperature in the SFP must be checked periodically to ensure the SFP structural integrity is met. The 24 hour Frequency is appropriate due to the large volume of water in the pool and the relatively slow heatup rate.

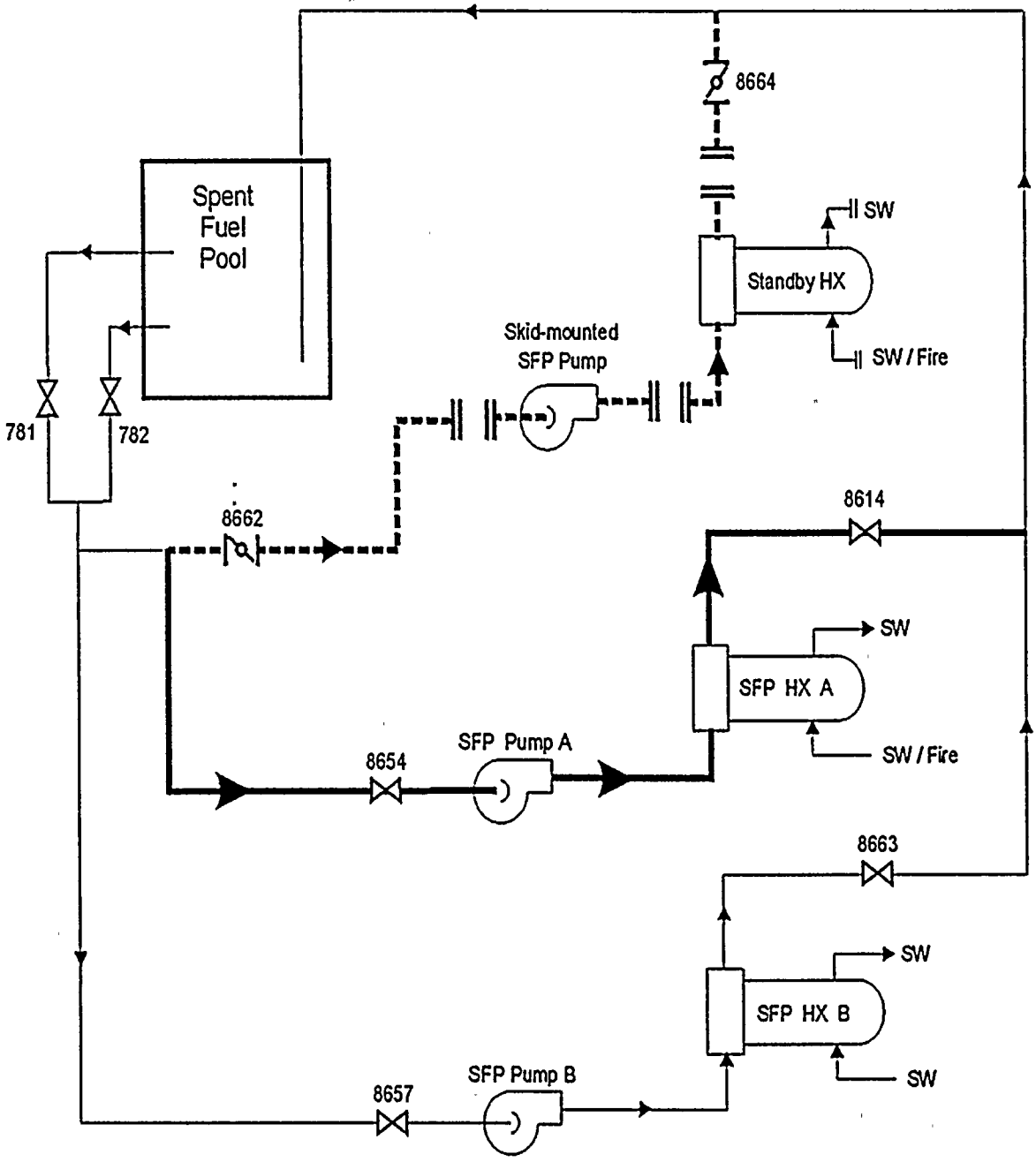
Verification of SFP water temperature is normally accomplished by the use of TIA-635, which also provides a SFP high temperature alarm to alert the operators of an increasing SFP temperature.

TSR 3.7.7.2

Verification that the required pumps are OPERABLE ensures that an additional SFP pump can be placed in operation, if needed, to maintain decay heat removal. Verification is performed by verifying proper breaker alignment and power available to the required pumps. The Frequency of 7 days is considered reasonable in view of other administrative controls available and has been shown to be acceptable by operating experience.

REFERENCES

1. UFSAR, Section 9.1.3.



For illustration only

Legend:

- Loop B Flowpath
- Loop A Flowpath
- Standby Loop Flowpath

Figure TRB 3.7.7-1
SFP Cooling System

50-244
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6/14/01

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Document Date: 06/04/2001

Author Name: Poifleit P S
Author Affiliation: Rochester Gas & Electric Corp
Addressee Name: Vissing G S
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June 4, 2001

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Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are revisions to Ginna Station Emergency Plan Implementing Procedures (EPIPs).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,

Peter S. Polfleit
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

PSP/jtw

A045

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PROCEDURE	REVISION NUMBER
EPIP 1-5	43
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EPIP 2-5	11
EPIP 2-17	6
EPIP 3-1	15
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Title: R. E. Ginna Nuclear Power Plant Technical Requirements Manual TRM, Revision 16.
Docket Number: 05000244
Document Date: 05/31/2001

6/12

Author Name:
Author Affiliation: Rochester Gas & Electric Corp
Addressee Name: Vissing G
Addressee Affiliation: NRC/Document Control Desk
Addressee Affiliation: NRC/NRR/DLPM/LPD1
Document Type: Manual
Document Type: Technical Specification
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Document Sensitivity: Non-Sensitive
Comment:

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Doc Id	Doc Title	Rev	Rev Date	Qty
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A001

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Rochester Gas and Electric Corporation
Inter-Office Correspondence

May 31, 2001

Subject: Technical Requirements Manual (TRM) Revision 16

To: Distribution

Attached are the revised pages for the Ginna Station Technical Requirements Manual (TRM). The change included within Revision 16 is summarized as follows:

> Table TR 5.1-1 was revised to add a new more conservative, letdown flow rate based, requirement for reactor coolant DOSE EQUIVALENT I-131 specific activity as the result of ACTION Report 2000-0450.

Revision 16 of the TRM is considered effective June 1, 2001. Instructions for the necessary changes to your controlled copy of the ITS are as follows:

Volume	Section	Remove	Insert
III	TRM	Cover Page	Cover Page
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III	TRM	LEP-ii	LEP-ii
III	TRM Chapter 5.0	TRM 5.1-1	TRM 5.1-1
III	TRM Chapter 5.0	TRM 5.1-2	TRM 5.1-2

Please contact Tom Harding (extension 3384) if you have any questions.

Thomas L. Harding

R.E. Ginna Nuclear Power Plant

Technical Requirements Manual

TRM

Revision 15

Responsible Manager: George Wrobel

Effective Date: 10/10/00
10/11/2000

Controlled Copy No. 1217

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5.0 ADMINISTRATIVE CONTROLS

TR 5.1 Temporary LCO Requirements

This section defines additional administrative restrictions placed on systems, structures, and components (SSCs) which are addressed within the ITS in order to ensure that they remain OPERABLE with respect to their ITS requirements. These are temporary requirements only until the ITS can be modified via a license amendment request, plant modification with a subsequent bases change, etc. Table TR 5.1-1 contains the listing of temporary LCO requirements and all associated background information.

Table TR 5.1-1
Temporary LCO Requirements

Item	Action Report #	Temporary LCO Requirement	Required Actions If Not Met	Effective Dates
5.1.1	97-0783	a. Verify key switches for MOVs 852A and 852B are in "off position" every 31 days.	a. Declare associated ECCS train inoperable per LCO 3.5.2.	6/19/97 until LCO 3.5.2 is changed
5.1.2	97-0894	a. Verify NaOH System tank NaOH solution concentration is ≤ 35% by weight every 184 days.	a. Declare NaOH system inoperable per LCO 3.6.6.	6/19/97 until LCO 3.6.6 is changed.

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Revision 20 to Nuclear Emergency Response Plan R.E. Ginna Nuclear Power Plant.

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A045 - OR Submittal: Emergency Preparedness Plans, Implementing Procedures, Correspondence

Docket: 05000244

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

TELEPHONE
AREA CODE 716 546-2700

May 18, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555
Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Nuclear Emergency Response Plan
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are corrections to Revision 20 of the Ginna Station Nuclear Emergency Response Plan (NERP).

We have determined, per the requirements of 10 CFR 50.54(q), that the changes do not decrease the effectiveness of the Nuclear Emergency Response Plan.

Very truly yours,

Peter S. Polfleit
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of the plan)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of the plan)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

PSP/jtw

A045

011440392

ROCHESTER GAS & ELECTRIC CORPORATION
INTEROFFICE CORRESPONDENCE

Date: May 17, 2001

Subject: Changes to Rev. 20 of the Nuclear Emergency Response Plan (NERP)

From: Peter Polfleit (Ext. 6772)

To: Controlled Copyholders

Attached please find the corrected pages for Revision 20 of the Nuclear Emergency Response Plan. Please remove/add the pages as directed below:

Remove	Replace With
iii	iii
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29	29
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99	99
100	100

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The company's Emergency Support Organization is charged with the responsibility of bringing together a cohesive company management and technical team. This organization will be calling upon the maximum resources available within the company and the entire nuclear industry for the purpose of (1) assuring the safe shutdown and recovery of Ginna Station following an accident condition and, (2) minimizing the impact of the situation on the health and safety of the public.

The Ginna Station short-term responding organization is made up of site personnel, those on-shift and those immediately available from the plant staff complement. The Emergency Support Organization will be available to the Ginna Station Plant Manager for implementation of long-term recovery operations.

The company's Emergency Support Organization is established under the leadership of a single individual called the EOF/Recovery Manager. The EOF/Recovery Manager is supported by various technical and advisory disciplines including Engineering, Facilities and Personnel, Nuclear Operations, Public Communications and Energy Delivery.

4.2.4 Implementation of the Offsite Emergency Organization:

Ginna Station procedures provide that the EOF/Recovery Manager (or qualified alternate) is notified and provided with details concerning the emergency, the emergency classification, station status, and immediate corporate assistance, if any, which may be required. The EOF/Recovery Manager has the authority to activate the Emergency Operations Facility.

The Emergency Support Organization (Figure 4.2C) will be activated under the Alert, Site Area Emergency and General Emergency categories although it may be activated under other categories. Upon activation of the Emergency Support Organization, the EOF is established in the basement of the leased office building at 49 East Avenue, adjacent of the main corporate office building. A Joint Emergency News Center (JENC) will also be activated in the main corporate office building. The JENC will be used to coordinate all news releases and press conferences with the appropriate Federal, State and local authorities.

Personnel who have responsibilities in the EOF will be notified of an incident and the need for them to report to the EOF according to an approved procedure. Each individual assigned to the EOF will have a designated alternate who will be notified as necessary.

3. Design and construction support with the responsibility of coordinating the activities of the company, A/E, NSSS supplier and construction forces on proposed station modifications or other design and construction support required for response and recovery.

4. Advisory support functions with advisory support consisting of senior representatives of the NSSS supplier and special consultants as necessary.

5. Public Communications' staff with the responsibility of providing administration, logistics, communications, and personnel support for the response and recovery operations.

6. Administration and logistics with the responsibility of providing administration, logistics, communications, and personnel support for the response and recovery operations.

The EOF will be organized as shown on the attached Emergency Support Organization Chart, Figure 4.2C. The positions will be filled by trained individuals as listed in Section 4.2.6.

4.2.6 Responsibilities:

The following positions are the principal managers and coordinators which make up the team in the EOF and Joint Emergency News Center in response to a nuclear emergency at Ginna Station. The responsibilities for each position are given in EPIP 4-7 and EPIP 5-7.

4.2.6.1 EOF/Recovery Manager assumes overall command and control of the recovery operation of the Ginna facility.

4.2.6.2 Energy Distribution Liaison provides advisory technical support, supplementary and complementary to onsite personnel using consultants and in-house technical experts, as necessary. Directs electric system operations to meet the emergency.

4.2.6.3 Nuclear Operations Manager assists EOF/Recovery Manager in coordinating activities of the offsite organization to support site activities.

4.2.6.4 Engineering Manager coordinates the design and construction activities of the utility, A/E, NSSS Supplier, construction forces, and outside vendors.

4.2.6.5 Facilities and Personnel Manager provides administrative, logistic, communications, and personnel support for the recovery operation.

4.2.6.6 News Center Manager directs implementation of procedures governing JENC facility activation, operations and public information functions.

4.2.6.7 Corporate Spokesperson advises the media and the community through the Public Communications function of current and potential plant conditions.

4.2.6.8 Offsite Agency Liaison advises County, State, and Federal Agency Representatives in the EOF or outside of conditions related to the incident.

4.2.6.9 EOF Dose Assessment Manager advises EOF/Recovery Manager of projected doses and Protective Action Recommendations (PARS) to limit exposure to the public in the affected Emergency Response Planning areas.

6.4.5 Contamination Control Provisions:

The plant site is divided into two categories, the Clean Area and the Radiologically Controlled Area. Entry to and exit from the Radiologically Controlled Area is normally through the designated Access Control Point. Any area in which radioactive materials and radiation are present shall be surveyed, classified, roped and conspicuously posted with the appropriate radiation caution sign. These then become Radiologically Controlled Areas and proper access is provided and controlled. Plant procedures provide the radiation or contamination levels at which an area is declared a Radiologically Controlled Area or removed from radiation control status.

The general arrangement of the service facilities is designed to provide adequate personnel decontamination and change areas. The clean locker room is used to store items of personal clothing not required or allowed in the Controlled Area.

The Access Control Point is employed as a protective clothing change area. A supply of clean protective clothing for personnel is maintained in this area, and there is provision for collection of used protective clothing.

All personnel will survey themselves on leaving the Controlled Area using equipment provided at the Access Control Point. A decontamination shower and washroom are located adjacent to the Access Control Point.

Personnel decontamination kits with instructions posted for their use are available in the dispensary described in Section 6.5, First Aid and Medical Facilities.

In the event of a site evacuation, provisions for decontamination are available at the Survey Center.

A 1000-gallon holding tank is available to contain decontamination water from a sink and shower located in the Survey Center. Decontamination water will be sampled prior to transfer, treatment or disposal.

6.4.6 Protective Equipment and Supplies:

Personnel entering the Controlled Area may be required to wear protective clothing. The nature of the work to be done governs the selection of protective clothing to be worn by individuals. The protective apparel available are shoe covers, head covers, gloves and coveralls. Additional items of specialized apparel such as plastic suits, face shields, and respirators are available for operations involving high levels of contamination. In all cases, Radiation Protection personnel shall evaluate the radiological conditions and specify the required items of protective clothing to be worn.

Respiratory protective devices are required wherever an airborne radiation area exists or is expected. In such cases, Radiation Protection personnel monitor the airborne concentrations and specify the necessary protective devices according to concentration and type of airborne contaminants present.

Available respiratory devices include full face air purifying respirators (filter type both negative and pressured powered air purifying units). Air line supplied respirators of pressure demand type are used as well as constant flow hoods. Self-Contained Breathing Apparatus, using full face masks and pressure demand regulators are also available.

For use in an emergency, equipment and supplies are located in the Control Room, Technical Support Center, Respiratory Protection Facility and the Survey Center. Equipment categories are given in Appendix D.

6.4.7 Emergency Vehicles:

In the event it becomes necessary to make use of automotive equipment, a number of vehicles will be available. These include company-owned vehicles and personal vehicles. A small delivery truck and a small work truck are assigned to the Station. The ground maintenance garage, nearby, is assigned a 4-wheel drive truck. Use of personal vehicles is allowed by the company policy regarding paid mileage for company use. Lastly, a large and diverse fleet of vehicles is available from the Rochester Gas and Electric Physical Services Department.

6.5 First Aid and Medical Facilities:

First aid and medical provisions include both onsite and offsite facilities. The latter are described in Section 4.3, Augmentation of the Emergency Organization. A dispensary onsite contains sinks, a toilet, a bed, a stretcher, and miscellaneous first aid equipment and supplies. Decontamination kits can be obtained from the main Radiation Protection group. Personnel decontamination kits and bioassay collection kits are available at Rochester General Hospital and Newark Wayne Community Hospital.

Auxiliary Operators and Security Officers are trained in first aid procedures using Red Cross Multi-Media or an equivalent program. An administrative procedure establishes a First Aid Team and the actions to be followed in the event of illness or injury at Ginna Station.

7.0 Maintaining Nuclear Emergency Preparedness:

Formalized training program(s) have been established to ensure that all personnel who actively participate in the Nuclear Emergency Response Plan (NERP) maintain their familiarity with the plan and their required response. A radiation emergency exercise shall be conducted at least annually, with emphasis placed upon orderly implementation of the emergency plan.

Personnel trained for onsite response to a radiation emergency are part of the regular plant staff and are trained to specific responsibilities within the emergency organization. Training is documented by the Department Manager, Nuclear Training, and Corporate Nuclear Emergency Planner. Any emergency plan work by consultants will be under the control of, and reviewed by, the Corporate Nuclear Emergency Planner.

Exercises shall be evaluated by the Corporate Nuclear Emergency Planner and reviewed by the Plant Operations Review Committee (PORC), thereby assuring the effectiveness of the plan throughout the lifetime of the R.E. Ginna facility.

7.1 Training and Drills:

Training classes on the emergency plan shall be conducted annually (+/- 3 months) for all RG&E personnel who may actively participate in the radiation emergency plan. Details of the training programs are established in TR C.22 (Nuclear Emergency Response Plan Training Program). Training will include a demonstration of their ability to perform the functions to which they may be assigned by participating in a Drill or Exercise at least once every two years. During drills, on-the-spot corrections of erroneous performance may be made, followed by a critique or corrective action.

Efforts will also be made to vary the timing of the exercises such that back-shifts will be involved once every six years (i.e. between 6 p.m. and 4 a.m.). Attempts will be made to have some drills unannounced.

Specialized training will be provided for:

1. Technical Support Center assignees
2. Operation Support Center assignees
3. First Aid Teams
4. Survey Teams
5. Emergency Operations Facility personnel

7.2 Annual Review and Revision of the Plan and Procedures:

Annual review and revision of the Nuclear Emergency Response Plan (EPIP 5-6) will occur following the annual QA audit. Revisions to the Plan are subject to approval by PORC and review by the Nuclear Safety and Audit Review Board (NSARB).

Revisions to the Plan and Emergency Plan Implementing Procedures (EPIP) may be the result of drills, exercises, training or routine surveillance. The Plan and Emergency Plan Implementing Procedures are reviewed by the EPIP Committee. EPIP Procedure changes are controlled using the guidance in the administrative procedures A-205.2 and the IP-PRO series.

Emergency procedure changes are controlled so that only current copies are available for use. Revised procedures are distributed to a list of controlled copy holders with receipt verification. Shift Operators and licensed staff are made aware of revisions during regularly scheduled training coordinated by the Nuclear Training Department. Emergency telephone numbers are kept up-to-date through quarterly review and distribution of revisions.

7.3 Emergency Equipment and Supplies:

The operational readiness of all items of emergency equipment and supplies will be assured through monthly inspections of emergency equipment. The implementing procedure includes inspecting and testing of equipment stored in the Control Room, Survey Center, Technical Support Center, Radiation Protection office, JENC and EOF. Also included is the procedure for testing the operability of the equipment.

Necessary transportation for offsite surveys will be a personal car supplied by one member of the team. Since most employees commute by private car because of the remoteness of the facility, no lack of vehicles is anticipated. Company policy provides for mileage reimbursement.

7.3.1 Siren Tests and operability:

The Ginna Emergency Sirens shall be activated at intervals not to exceed one year (+ or - Three calendar Months). This test will be considered successful if no more than 10% (10 of 96) of the sirens fail to operate properly. Any time it is found that more than 25% (24 of 96) of the sirens are inoperable for more than One Hour, Then a One-Hour notification will be made to the NRC. Using procedure 0-9.3 Attachment 3 (reference: 10CFR50.72(b)(1)(v)).

7.4 Auditing:

The Nuclear Emergency Response Plan, its implementing procedures, equipment, training and interface with State and county authorities is audited at a 12 Month frequency ± 3 months and audit results are reviewed by the NSARB. Follow-up implementation tasks are assigned through the use of the ACTION Reports. Audit documents are retained for 5 years.

8.0. Recovery:

After the initial emergency response actions are concluded (i.e., the plant is in cold shutdown and under control), a decision to begin the recovery phase will be initiated. A number of considerations will enter into the decision to begin the recovery phase and dismantle the Emergency Response Organization. The decision to enter the recovery phase will be made by the EOF/Recovery Manager in consultation with his support managers, the Plant Operation Review Committee, and the NSARB and onsite personnel. EPIP 3-4, "Emergency Termination and Recovery" will be used by the organization to transition from a response organization to a recovery organization.

The decision to enter the recovery phase should be based upon a comprehensive review of station parameters and conditions. These should include, but are not limited to the following:

1. The initial emergency response actions are concluded (i.e., the station is in cold shutdown and under control).

2. Station parameters of operation no longer indicate a potential or actual emergency exists.

3. The reactor shutdown conditions are stable.

4. The reactor containment building integrity is intact.

5. The release of radioactivity from the station is controllable and no longer exceeds permissible levels, and no danger to the general public from the above source(s) is credible.

6. Radioactivity waste systems and decontamination facilities are operable to the extent needed.

7. A reactor heat sink is available and operating.

8. The integrity of power supplies and electrical equipment needed for the station to be capable of sustaining itself in a long term shutdown condition is intact.

50-244.
4/30/01
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5/24/01

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Docket: 05000244

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April 30, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555
Attn: Mr. Guy S. Vissing (Mail Stop 14D11)
Project Directorate I-1

Subject: Revision to Emergency Plan Implementing Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Gentlemen:

In accordance with 10 CFR 50.4(b)(5), enclosed are revisions to Ginna Station Emergency Plan Implementing Procedures (EPIPs).

We have determined, per the requirements of 10 CFR 50.54(q), that these procedure changes do not decrease the effectiveness of our Nuclear Emergency Response Plan.

Very truly yours,

Peter S. Pollieit
Corporate Nuclear Emergency Planner

Enclosures

xc: USNRC Region 1 (2 copies of letter and 2 copies of each procedure)
Resident Inspector, Ginna Station (1 copy of letter and 1 copy of each procedure)
RG&E Nuclear Safety and Licensing (1 copy of letter)
Dr. Robert C. Mecredy (2 copies of letter only)

PSP/jtw

ML011280194

A045

PROCEDURE	REVISION NUMBER
EPIP 2-2	10
EPIP 2-11	15
EPIP 2-12	18
EPIP 4-7	16

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4/17/01

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A002 - OR Submittal:Inadequate Core Cooling (Item II.F.2) GL 82-28

Docket: 05000244

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JOSEPH A. WIDAY
VICE PRESIDENT & PLANT MANAGER
GINNA STATION

March 26, 2001

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy S. Vissing
Project Directorate I
Washington, D.C. 20555

Subject: Emergency Operating Procedures
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Vissing:

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

Joseph A. Widay

JAW/jdw

xc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

AP Index
AP-PRZR.1, Rev 12

A002

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REPORT NO. 01
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PROCEDURES INDEX
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03/15/01 PAGE: 1

PARAMETERS: DOC TYPES - PRAP STATUS: EF QU 5 YEARS ONLY:

PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-CCW.1	LEAKAGE INTO THE COMPONENT COOLING LOOP	014	01/09/01	05/01/98	05/01/03	EF
AP-CCW.2	LOSS OF CCW DURING POWER OPERATION	014	05/18/00	08/17/99	08/17/04	EF
AP-CCW.3	LOSS OF CCW - PLANT SHUTDOWN	012	05/18/00	08/17/99	08/17/04	EF
AP-CR.1	CONTROL ROOM INACCESSIBILITY	016	01/11/00	01/11/00	01/11/05	EF
AP-CVCS.1	CVCS LEAK	012	05/01/98	05/01/98	05/01/03	EF
AP-CVCS.3	LOSS OF ALL CHARGING FLOW	002	02/11/00	02/26/99	02/26/04	EF
AP-CW.1	LOSS OF A CIRC WATER PUMP	010	07/16/98	05/01/98	05/01/03	EF
AP-ELEC.1	LOSS OF 12A AND/OR 12B BUSSES	020	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.2	SAFEGUARD BUSSES LOW VOLTAGE OR SYSTEM LOW FREQUENCY	009	03/22/99	03/22/99	03/22/04	EF
AP-ELEC.3	LOSS OF 12A AND/OR 12B TRANSFORMER (BELOW 350 F)	008	09/08/00	05/01/98	05/01/03	EF
AP-ELEC.14/16	LOSS OF SAFEGUARDS BUS 14/16	003	03/15/01	06/09/97	06/09/02	EF
AP-ELEC.17/18	LOSS OF SAFEGUARDS BUS 17/18	002	10/18/99	06/09/97	06/09/02	EF
AP-FW.1	PARTIAL OR COMPLETE LOSS OF MAIN FEEDWATER	012	02/11/00	02/27/98	02/27/03	EF
AP-IA.1	LOSS OF INSTRUMENT AIR	017	12/02/99	05/01/98	05/01/03	EF
AP-PRZR.1	ABNORMAL PRESSURIZER PRESSURE	011	12/02/99	12/02/99	12/02/04	EF
AP-RCC.1	CONTINUOUS CONTROL ROD WITHDRAWAL/INSERTION	006	02/24/96	05/14/98	05/14/03	EF
AP-RCC.2	RCC/RPI MALFUNCTION	008	11/16/98	02/06/97	02/06/02	EF
AP-RCC.3	DROPPED ROD RECOVERY	004	11/16/98	02/27/98	02/27/03	EF
AP-RCP.1	RCP SEAL MALFUNCTION	013	06/09/00	05/01/98	05/01/03	EF
AP-RCS.1	REACTOR COOLANT LEAK	015	09/08/00	05/01/98	05/01/03	EF
AP-RCS.2	LOSS OF REACTOR COOLANT FLOW	010	12/14/98	05/01/98	05/01/03	EF
AP-RCS.3	HIGH REACTOR COOLANT ACTIVITY	007	08/05/97	08/05/97	08/05/02	EF
AP-RCS.4	SHUTDOWN LOCA	011	12/02/99	05/01/98	05/01/03	EF
AP-RHR.1	LOSS OF RHR	015	02/08/01	05/01/98	05/01/03	EF

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PROCEDURE NUMBER	PROCEDURE TITLE	REV	EFFECT DATE	LAST REVIEW	NEXT REVIEW	ST
AP-RHR.2	LOSS OF RHR WHILE OPERATING AT RCS REDUCED INVENTORY CONDITIONS	009	10/13/00	03/31/00	03/31/05	EF
AP-SG.1	STEAM GENERATOR TUBE LEAK	000	09/08/00	09/08/00	09/08/05	EF
AP-SW.1	SERVICE WATER LEAK	015	10/18/99	06/03/98	06/03/03	EF
AP-TURB.1	TURBINE TRIP WITHOUT RX TRIP REQUIRED	010	02/12/99	10/10/97	10/10/02	EF
AP-TURB.2	TURBINE LOAD REJECTION	017	02/11/00	05/13/98	05/13/03	EF
AP-TURB.3	TURBINE VIBRATION	010	02/11/00	02/10/98	02/10/03	EF
AP-TURB.4	LOSS OF CONDENSER VACUUM	014	05/01/98	05/01/98	05/01/03	EF
AP-TURB.5	RAPID LOAD REDUCTION	005	06/09/00	06/09/00	06/09/05	EF

TOTAL FOR PRAP 32