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 MECREDY, R.C. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 VISSING, G.

SUBJECT: Submits 30-day annual ECCS rept re error in LOCA analysis
 per 10CFR50.46, paragraph (a)(3)(ii).

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ROBERT C. MECREDY
Vice President
Nuclear Operations

July 8, 1997

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Guy Vissing
Project Directorate I-1
Washington, D.C. 20555

Subject: 10CFR50.46 30 Day and Annual ECCS Report
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref. (a): Westinghouse letter SAE-ESI-97-340, Subject: 10CFR50.46,
Reporting Information, dated June 4, 1997.

(b) Westinghouse letter RGE-97-200, Subject 10CFR50.46 Annual
Notification and Reporting, dated April 17, 1997.

Dear Mr. Vissing:

In accordance with the requirement in 10CFR50.46, paragraph
(a)(3)(ii), this 30 day and annual ECCS report is hereby submitted.

Westinghouse, the provider of LOCA analysis for the R.E. Ginna
Nuclear Power Plant, has provided RG&E with information regarding
an error in the LOCA analysis that requires 30 day NRC notification
(Reference a). In addition, References (a) and (b) provide an
update to the peak cladding temperature (PCT) margin for 1996 and
1997 changes.

The large break LOCA PCT has increased by 58°F due to the
misinterpretation/re-calculation of nominal initial accumulator
water volume and transposition errors in transfer of code restart
data. The new large break LOCA PCT is 2158°F and is summarized in
Attachment 1 to this letter. The 58°F PCT penalty is the result of
Ginna specific computer runs. The computer runs utilized the
latest code version, a bounding initial accumulator water volume,
implicitly corrects the restart data transfer errors, and corrects
the errors reported in the 1995 Westinghouse Annual Report.
Therefore, the current PCT is the result of direct plant specific
sensitivity reanalysis. No schedule for reanalysis is proposed
because Ginna specific analysis has been completed in determining
the PCT penalty.

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The small break LOCA PCT has increased by 10°F due to an error in the fuel rod initialization process. The new small break LOCA PCT is 1323°F and is summarized in Attachment 1 to this letter.

Very truly yours,

Thomas A. Marlow For

Robert C. Mecredy

Attachment
RWE/465

xc: Mr. Guy Vissing (Mail Stop 14B2)
Project Directorate I-1
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Ginna Senior Resident Inspector

ATTACHMENT 1

LOCA PCT SUMMARY

JULY 1997 UPDATE

ATTACHMENT 1

LOCA PCT SUMMARY

Small Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: NOTRUMP Fuel: OFA
 $F_Q = 2.50$ $F_{AH} = 1.75$ SGTP = 15%

- A. Analysis of Record (6/95) (effective 6/96) PCT = 1308°F
- B. 1995 10CFR50.46 Model Assessments
 - 1. NOTRUMP Specific Enthalpy Error $\Delta PCT = 20^\circ F$
 - 2. SALIBRARY Double Precision Errors $\Delta PCT = -15^\circ F$
- C. 1996 10CFR50.46 Model Assessments
 - 1. SBLOCA Fuel Rod Initialization Error $\Delta PCT = +10^\circ F$
- D. 1997 10CFR50.46 Model Assessment
 - 1. none $\Delta PCT = 0^\circ F$
- E. Other margin allocations
 - 1. none $\Delta PCT = 0^\circ F$

Licensing Basis

PCT = 1323°F

Revision Date: 7/97

ATTACHMENT 1

LOCA PCT SUMMARY

Large Break LOCA
R.E. Ginna Nuclear Power Plant
Rochester Gas and Electric Corporation

Evaluation Model: UPI SECY Fuel: OFA
 $F_Q = 2.45$ $F_{AH} = 1.75$ SGTP = 15%

- A. Analysis of Record (5/95) (effective 6/96) PCT = 2051°F
- B. 1995 10CFR50.46 Model Assessments
 1. Fixed heat transfer node assignment
 error/Accumulator water injection
 error ΔPCT = 48°F
- C. 1996 10CFR50.46 Model Assessments
 1. none ΔPCT = 0°F
- D. 1997 10CFR50.46 Model Assessments
 1. Accumulator Initial Water Volume ΔPCT = 58°F
 Restart Data Transposition Error
 Plant Specific Analytical Reassessment
 of 1995 Model Assessments
- E. 10CFR50.59 Evaluations
 1. Service Water Temp. $\geq 30^\circ\text{F}$ ΔPCT = 1°F
 (1997 evaluation; SEV-1090)

Licensing Basis PCT = 2158°F

Revision Date: 7/97

