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 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G    05000244  
 AUTH. NAME    AUTHOR AFFILIATION  
 MECREDY, R.C.    Rochester Gas & Electric Corp.  
 RECIP. NAME    RECIPIENT AFFILIATION  
 VISSING, G.S.

*See  
 Revised  
 Change To  
 Tech Specs*

SUBJECT: Forwards application for amend to License DPR-18, revising  
 RCS pressure & Temp limits rept, per GL 96-03. Non-proprietary  
 WCAP-14684 rept encl.

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Vice President  
Nuclear Operations

September 13, 1996

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Attn: Guy S. Vissing  
Project Directorate I-1  
Washington, D.C. 20555

Subject: Application for Amendment to Facility Operating License, Revision to Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)  
Rochester Gas & Electric Corporation  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

Dear Mr. Vissing,

By Reference (a), the NRC approved changes to Administrative Controls Section 5.6.6 of the Ginna Station Technical Specifications which allowed referencing of Revision 1 of the Ginna Station PTLR for the RCS pressure and temperature (P/T) limits and low temperature overpressure protection (LTOP) limits. However, this approval was only provided until December 31, 1996 since the fluence factors used to generate the values provided in PTLR Revision 1 were estimated and not based on the methodology required by Reference (b). As such, the NRC required that RG&E submit a license amendment request (LAR) with a new PTLR by October 2, 1996 using the methodology provided in Reference (b) to allow the NRC time for appropriate review.

Therefore, the purpose of this letter is to provide the required LAR, Revision 2 to the PTLR, and the necessary supporting calculations (i.e., WCAP-14684). Since the P/T limits provided in Revision 2 of the PTLR were developed using the methodology of Reference (b), the LAR will incorporate reference to this methodology into the Administrative Controls section. The LTOP limits are based on the methodology of Reference (c) which is already listed in the Administrative Controls section. As such, these changes will allow RG&E to perform future P/T and LTOP evaluations using NRC approved methodology without requiring a potential technical specification change. The change will also complete incorporation of Generic Letter 96-03 into the Ginna Station Technical Specifications since the existing Ginna Station PTLR cannot be revised without a license amendment request (LAR).

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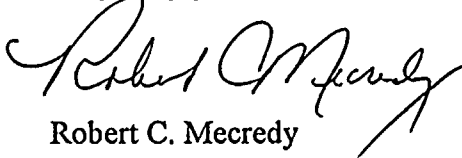
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Consistent with Generic Letter 96-03, the proposed revised PTLR contains: (1) reactor vessel material information, (2) LTOP setpoints, and (3) P/T curves. Although the NRC recently approved a new pressurized thermal shock (PTS) evaluation for Ginna Station (Ref. (d)), the use of the new fluence values (and additional surveillance capsule chemistry samples) impacts that evaluation as documented in WCAP-14684. In summary, this letter provides the following:

- a. An LAR which revises Administrative Controls Section 5.6.6 to replace the reference to the May 23, 1996 NRC letter (which is the basis for the existing LTOP and P/T limits) with a new reference containing the NRC's approval of the attached PTLR. In addition, the list of documents containing the approved NRC methodology will be updated to reference WCAP-14040-NP-A for the P/T curves and this letter. The methodology for the LTOP setpoints and enable temperature (Ref. c) remains unchanged. Finally, a typographical error is corrected.
- b. Revision 2 of the PTLR which contains the following:
  - i. Update of all reactor vessel material information as documented in WCAP-14684.
  - ii. Revision to the LTOP enable temperature using the methodology submitted as Attachment II to Reference (c) due to the revised P/T curves. The LTOP setpoints remain unchanged as based on the calculation included as Attachment IV to Reference (e).
  - iii. Update of the P/T curves based on the methodology of Reference (b) as documented in WCAP-14684.
- c. WCAP-14684 which provides supporting calculations.

Since the existing PTLR expires on December 31, 1996, RG&E requests NRC consideration of this LAR by December 15, 1996 in order to support necessary implementation.

Very truly yours,

  
Robert C. Mecredy

MDF\870  
Attachments

References:

- (a) Letter from G.S. Vissing NRC, to R.C. Mecredy, RG&E, Subject: *Issuance of Amendment No. 64 to Facility Operating License No. DPR-18, R.E. Ginna Nuclear Power Plant (TAC No. M94770)*, dated May 23, 1996.
- (b) WCAP-14040-NP-A, *Methodology Used to Develop Cold Overpressure Mitigating System Setpoints and RCS Heatup and Cooldown Limit Curves*, January 1996.
- (c) Letter from R.C. Mecredy, RG&E, to A.R. Johnson, NRC, Subject: *Technical Specification Improvement Program, Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)*, dated December 8, 1995.
- (d) Letter from A.R. Johnson, NRC, to R.C. Mecredy, RG&E, Subject: *R.E. Ginna Nuclear Power Plant - Pressurized Thermal Shock Evaluation (TAC No. M93827)*, dated March 22, 1996.
- (e) Letter from R.C. Mecredy, RG&E, to A.R. Johnson, NRC, Subject: *Application for Amendment to Facility Operating License, Methodology for Low Temperature Overpressure Protection (LTOP) Limits*, dated February 9, 1996.

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