

CATEGORY 1

REGULATOR INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9603210226 DOC. DATE: 96/03/15 NOTARIZED: NO DOCKET #
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 MECREDY, R.C. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 JOHNSON, A.R.

SUBJECT: Forwards application for amend to license DPR-18, to
 incorporate methodology for determining LTOP limits into
 Administrative Controls Section 5.6.6, RCS Pressure Temp
 Limits Rept.

DISTRIBUTION CODE: A001D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3+11
 TITLE: OR Submittal: General Distribution

NOTES: License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
PD1-1 LA	1 1	PD1-1 PD	1 1
JOHNSON, A	1 1		
INTERNAL: <u>FILE CENTER</u> 01	1 1	NRR/DE/EMCB	1 1
NRR/DRCH/HICB	1 1	NRR/DSSA/SPLB	1 1
NRR/DSSA/SRXB	1 1	NUDOCS-ABSTRACT	1 1
OGC/HDS3	1 0		
EXTERNAL: NOAC	1 1	NRC PDR	1 1

NOTE TO ALL "RIDS" RECIPIENTS:
 PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK,
 ROOM OWFN 5D-5 (EXT. 415-2083) TO ELIMINATE YOUR NAME FROM
 DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 12 ENCL 11

AA2

C
A
T
E
G
O
R
Y

1

D
O
C
U
M
E
N
T



ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001



AREA CODE 716 546-2700

ROBERT C. MECREDY
Vice President
Nuclear Operations

March 15, 1996

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Application for Amendment to Facility Operating License
Implementation of Generic Letter 96-03
Rochester Gas & Electric Corporation
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

- References:
- (a) Letter from R.C. Mecredy, RG&E, to A.R. Johnson, NRC, Subject: *Application for Amendment to Facility Operating License, Methodology for Low Temperature Overpressure Protection (LTOP) Limits*, dated February 9, 1996.
 - (b) Generic Letter 96-03, *Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits*
 - (c) Letter from L.T. Marsh, NRC, to R.C. Mecredy, RG&E, Subject: *R.E. Ginna - Acceptance for Referencing of Pressure Temperature Limits Report (TAC # M92320)*, dated December 26, 1995.

Dear Mr. Johnson,

By Reference (a), RG&E proposed to revise the Ginna Station Technical Specifications to incorporate the methodology for determining the low temperature overpressure protection (LTOP) limits into Administrative Controls Section 5.6.6, RCS Pressure and Temperature Limits Report (PTLR). This change would allow RG&E to perform future LTOP evaluations using NRC approved methodology without requiring a potential technical specification change. However, any changes to the RCS pressure temperature (P/T) limits, which are also included in the PTLR, would still require prior NRC review and approval. Incorporation of the methodology for the P/T limits was not requested at the time since the NRC's review of the latest reactor vessel capsule data (which would provide input into the P/T limit determination) was incomplete.

210015
9603210226 960315
PDR ADCK 05000244
PDR

A001
11



Number of hauls	<i>P. setiferus</i> (%)	<i>P. setiferus</i> + <i>P. setiferus</i> + <i>P. setiferus</i> (%)
1	~10	~10
2	~20	~10
3	~30	~10
4	~40	~10
5	~50	~10
6	~60	~10
7	~70	~10
8	~80	~10
9	~90	~10
10	~95	~10

www.ck12.org

However, the NRC's review of the latest reactor vessel capsule data is nearing completion which would allow use of previously NRC approved methodology. In addition, Generic Letter 96-03 (Ref. (b)) has been released by the NRC. This generic letter provides guidance for licensees to relocate the P/T limits and LTOP limits from technical specifications and to provide a mechanism for licensees to make changes to the limits using NRC approved methodology. RG&E has previously relocated these limits; however, a license amendment is still required for all changes to the actual limits since the methodology for determining these limits has not been approved for future applications (Ref. (c)). As such, RG&E is submitting this license amendment request (LAR) to incorporate the methodology for determining both LTOP limits and RCS P/T limits into Administrative Controls Section 5.6.6 consistent with Generic Letter 96-03. Therefore, this LAR supersedes that provided in Reference (a).


Generic Letter 96-03 requests that licensees provide the following in their LAR:

1. Provide a reference to the NRC approved P/T limit and LTOP limit methodology in the technical specifications;
2. Develop a PTLR or similar document that contains the figures, values, parameters, and any explanations derived from the methodology; and
3. Provide other required changes to various technical specifications (e.g., definitions section) to support the relocation of the P/T and LTOP limits.

The purpose of this LAR is to provide the reference to the P/T limit and LTOP limit methodology in the technical specifications (item 1). A PTLR has been previously developed as part of the conversion to improved standard technical specifications (Ref. (c)). However, the revised PTLR required to support use of the proposed methodology is dependant upon the input data which is used, including that derived from the latest reactor vessel capsule. Once the NRC's review of the reactor vessel capsule is complete, RG&E will provide the revised PTLR using the proposed methodology. Therefore, item 2 is not being provided at this time. No other technical specification changes are required to implement Generic Letter 96-03 (item 3) since this was addressed during the conversion to the improved technical specifications.

Following the NRC's review of the latest reactor vessel capsule data, it is anticipated that the P/T limits may not be valid for startup from the 1996 refueling outage. As such, RG&E requests NRC consideration of this LAR in order to support a planned startup date of May 30, 1996. To provide a necessary implementation period, RG&E should receive NRC acceptance of this LAR by May 10, 1996. The revised PTLR using the proposed methodology will be submitted shortly after receipt of the NRC's review of the latest reactor vessel capsule in order to support this short schedule.

Very truly yours,


Robert C. Mecredy

MDF\817
Attachments

xc: U.S. Nuclear Regulatory Commission
Mr. Allen R. Johnson (Mail Stop 14B2)
PWR Project Directorate I-1
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Ginna Senior Resident Inspector

Mr. F. William Valentine, President
New York State Energy, Research, and Development Authority
2 Rockefeller Plaza
Albany, NY 12223-1253