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JOHNSON,A.R. I

SUBJECT: Informs that RG&E endorses encl rept BAW-2257, "Response to
Part (1) of GL 92-01,Rev 1,Suppl 1" submitted by B&WOG on
950801,as representing required response to GL 92-01,Rev 1,
Suppl 1 for RE Ginna Nuclear Power Plant. O

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ROBERT C. MECREDY
Vice President
Nuclear Operations

August 11, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-1
Washington, D.C. 20555

Subject: Response to NRC Generic Letter 92-01, Revision 1,
Supplement 1: Reactor Vessel Structural Integrity, dated
May 19, 1995 (TAC No. M83733)
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Ref.(a): Letter from J. H. Taylor (B&W) to Document Control Desk
(NRC), Subject: B&W Owners Group Reactor Vessel
Integrity Program, OG-95-1527, dated August 1, 1995

Dear Mr. Johnson:

Generic Letter 92-01 Revision 1, Supplement 1 requested, within 90 days a response to Part (1) of the Required Information section of the generic letter. Part (1) required the following information:

"a description of those actions taken or planned to locate all data relevant to the determination of RPV integrity or an explanation of why the existing data base is considered complete as previously submitted."

By reference (a) the B&W owners group submitted BAW-2257 July 1995 on behalf of the B&W Owners Group Reactor Vessel Working Group, of which RG&E is a member. That report entitled "Response to Part (1) of Generic Letter 92-01, Revision 1, Supplement 1" provides the list of documents that are in the possession of NRC and were considered in the development of the previous responses to Generic Letter 92-01 Revision 1 and associated requests for additional information. RG&E endorses reference (a) as representing the required response to Generic Letter 92-01, Revision 1, Supplement 1 for R. E. Ginna Nuclear Power Plant. A copy of BAW-2257 is attached herein.

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RG&E response to Parts (2), (3), and (4) of the generic letter will be submitted by November 19, 1995 as requested.

Very truly yours,


Robert C. Mecredy

REJ\390
Attachment

Subscribed and sworn to before me
on this 11th day of August, 1995



JOANNE S. GORMAN
Notary Public in the State of New York
Orleans County
Commission Expires Nov. 19 96

xc: Mr. Allen R. Johnson (Mail Stop 14B2)
Project Directorate I-1
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Ginna Senior Resident Inspector

JOHANNES S. GORMAN
Notary Public in the State of New York
Queens County, N.Y.
Commission Expires Nov. 19

JOHANNES S. GORMAN

BAW-2257
JULY 1995

RESPONSE TO PART (1) OF
GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1

Prepared by

M. J. DeVan
L. B. Gross

BWNT Document No. 43-2257-00

Prepared for

B&W Owners Group Reactor Vessel Working Group

Commonwealth Edison Company
Duke Power Company
Entergy Operations, Inc.
Florida Power Corporation
Florida Power & Light Company
GPU Nuclear Corporation
Rochester Gas and Electric Corporation
Toledo Edison Company
Virginia Power
Wisconsin Electric Power Company

B&W NUCLEAR TECHNOLOGIES
Engineering and Project Services Division
P. O. Box 10935
3315 Old Forest Road
Lynchburg, VA 24506-0935

1. INTRODUCTION

This document provides the Owners Group response to Part (1) of Nuclear Regulatory Commission (NRC) Generic Letter 92-01, Revision 1, Supplement 1, for the B&W Owners Group Reactor Vessel Working Group member plants as listed below.

Generic Letter 92-01, Revision 1, Supplement 1, was issued by the NRC on May 19, 1995 to holders of nuclear power plant operating licenses. The generic letter was issued to ensure that all licensees have all of the data relevant to the evaluation of the structural integrity of their RVPs and that all such data is appropriately included in their assessments of compliance with regulatory requirements regarding RPV integrity. Response to Part (1) is required within 90 days of the issue date; with a deadline of August 17, 1995. This document provides the required information in response to Part (1) of the request, for the following plants:

<u>Plant</u>	<u>Owner</u>
Arkansas Nuclear One Unit 1	Entergy Operations, Inc.
Crystal River Unit 3	Florida Power Corporation
Davis-Besse	Toledo Edison Company
R. E. Ginna	Rochester Gas and Electric Corp.
Oconee Unit 1	Duke Power Company
Oconee Unit 2	Duke Power Company
Oconee Unit 3	Duke Power Company
Point Beach Unit 1	Wisconsin Electric Power Co.
Point Beach Unit 2	Wisconsin Electric Power Co.
Surry Unit 1	Virginia Power
Surry Unit 2	Virginia Power
Three Mile Island Unit 1	GPU Nuclear Corporation
Turkey Point Unit 3	Florida Power & Light Company
Turkey Point Unit 4	Florida Power & Light Company
Zion Unit 1	Commonwealth Edison Company
Zion Unit 2	Commonwealth Edison Company

2. STATEMENT OF RESPONSE

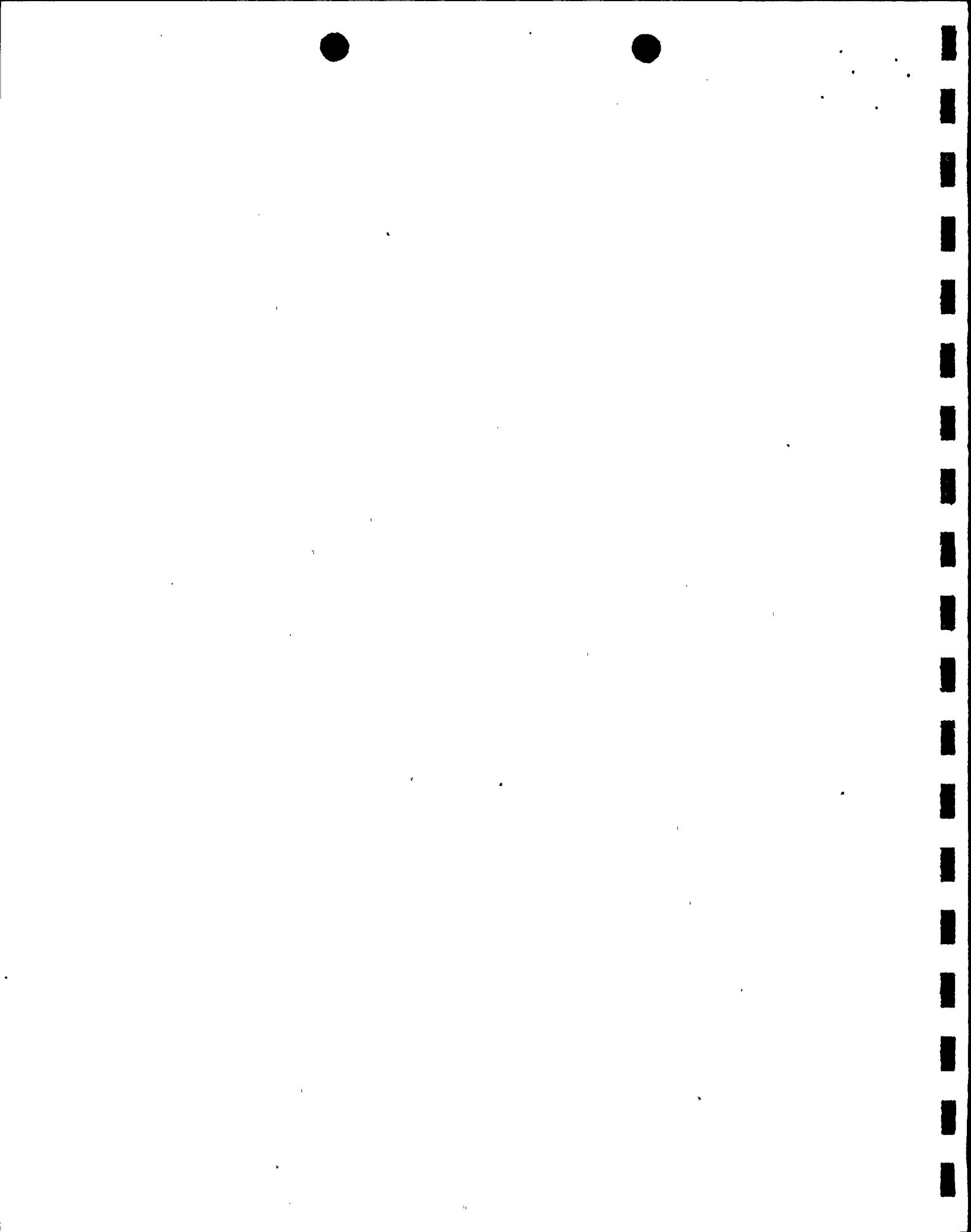
Part (1) of the Generic Letter requires the Addressees to provide the following information:

"a description of those actions taken or planned to locate all data relevant to the determination of RPV integrity, or an explanation of why the existing data base is considered complete as previously submitted"

The B&W Owners Group's Materials Committee initiated efforts in 1977 to resolve the reactor vessel integrity issue for high copper, Linde 80, automatic submerged arc welds. Its successor organization, the B&W Owners Group's Reactor Vessel Working Group (RVWG), has continued these efforts to acquire and document relevant data for determining reactor vessel integrity of its member plants. Appendix A provides a list of documents that are in the possession of the NRC and were considered in the development of the previous RVWG responses to Generic Letter 92-01, Revision 1 and associated requests for additional information.

Information for these documents was obtained from the following sources:

- o Babcock & Wilcox fabrication documentation (e.g., weld qualification reports)
- o Investigations sponsored by:
 - B&W Owners Group
 - Electric Power Research Institute (EPRI)
 - NRC (at the Oak Ridge National Laboratory)
- o Reactor vessel material surveillance program reports



The RVWG's Master Integrated Reactor Vessel Material Surveillance Program (MIRVP) links all the operating PWRs that contain high copper, Linde 80 welds and makes careful note of where the same weld, or its surrogate, is used in more than one reactor vessel. These data are shared for regulatory analyses regarding reactor vessel integrity.

A review of the list of reactor vessels fabricated by Babcock & Wilcox indicates that additional weld chemistry and initial Charpy V-notch and Drop Weight impact toughness data from a few domestic BWR and foreign PWR vessels may be available. The B&W Owners RVWG will make efforts to obtain and evaluate the relevance of these data. Nevertheless, the RVWG believes that the available data, relevant to the assessment of its member plant vessels, have been appropriately considered in the submitted reactor vessel integrity evaluations.

Written responses to Parts (2), (3), and (4) of Generic Letter 92-01, Revision 1, Supplement 1, shall be provided within 6 months of the issue date of the Generic Letter (May 19, 1995).

3. CERTIFICATION

This report accurately responds to the information requested in Part (1) to Generic Letter 92-01, Revision 1, Supplement 1.



M. J. DeVan, Engineer III
Materials & Structural Analysis Unit

7/31/95

Date

This report has been reviewed for technical content and accuracy.



S. Fyfe, Supervisory Engineer
Materials & Structural Analysis Unit

7/31/95

Date

Verification of independent review.



K. E. Moore, Manager
Materials & Structural Analysis Unit

7/31/95

Date

This report is approved for release.



D. L. Howell
Program Manager

7/31/95

Date

APPENDIX A
DOCUMENT SUMMARY

Perrin, J.S., et al., "Final Report on Point Beach Nuclear Plant Unit No. 1 Pressure Vessel Surveillance Program: Evaluation of Capsule V," BMI-0673, Battelle Columbus Laboratories, Columbus, Ohio, June 15, 1973.

Perrin, J.S., et al., "Final Report on Point Beach Nuclear Plant Unit No. 2 Pressure Vessel Surveillance Program: Evaluation of Capsule V," BMI-0675, Battelle Columbus Laboratories, Columbus, Ohio, June 10, 1975.

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Lowe, A.L., Jr., et al., "Analysis of Capsule OC1-F from Duke Power Company Oconee Unit 1 Reactor Vessel Materials Surveillance Program," BAW-1421, Rev. 1, Babcock & Wilcox, Lynchburg, Virginia, September 1975.*

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Harbison, L. S., "Master Integrated Reactor Vessel Surveillance Program," BAW-1543, Revision 4, B&W Nuclear Technologies, Inc., Lynchburg, Virginia, February 1993.

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Lowe, A.L., Jr., et al., "Analysis of Capsule CR3-B Florida Power Corporation Crystal River Unit 3 Reactor Vessel Materials Surveillance Program," BAW-1679, Rev. 1, Babcock & Wilcox, Lynchburg, Virginia, June 1982.*

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Lowe, A.L., Jr., et al., "Analysis of Capsule ANI-B from Arkansas Power & Light Company's Arkansas Nuclear One, Unit 1 Reactor Vessel Materials Surveillance Program," BAW-1698, Babcock & Wilcox, Lynchburg, Virginia, November 1981.*

Lowe, A.L., Jr., et al., "Analysis of Capsule OCII-A from Duke Power Company's Oconee Nuclear Station, Unit 2 Reactor Vessel Material Surveillance Program," BAW-1699, Babcock & Wilcox, Lynchburg, Virginia, December 1981.*

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Lowe, A.L., Jr., et al., "Analysis of Capsule RS1-B Sacramento Municipal Utility District Rancho Seco Unit 1 Reactor Vessel Material Surveillance Program," BAW-1702, Babcock & Wilcox, Lynchburg, Virginia, February 1982.*

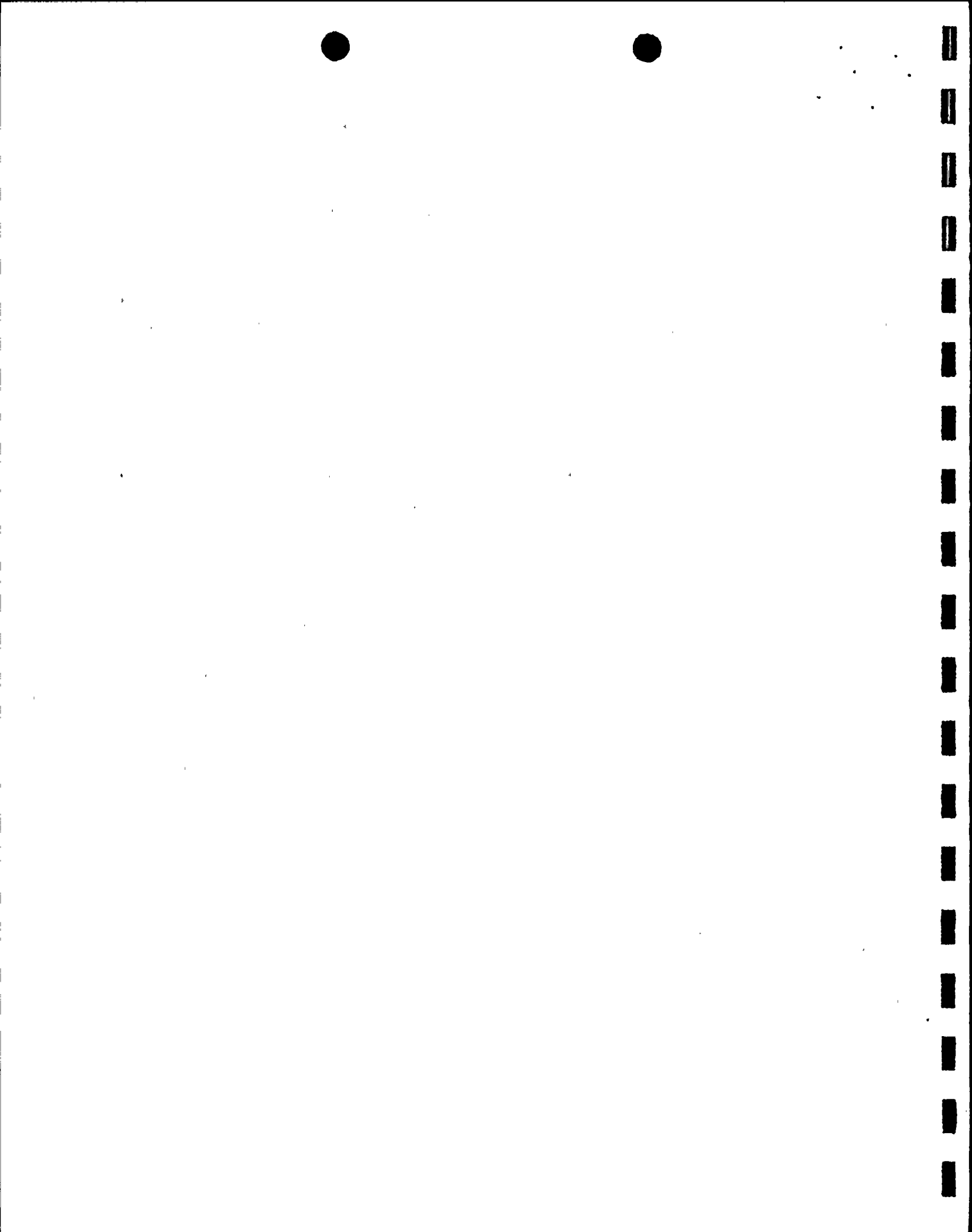
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Aadland, J.D., "Babcock & Wilcox Owners' Group 177-Fuel Assembly Reactor Vessel and Surveillance Program Materials Information," BAW-1820, Babcock & Wilcox, Lynchburg, Virginia, December 1984.*

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* This report is available from B&W Nuclear Technologies, Inc., Lynchburg, Virginia.



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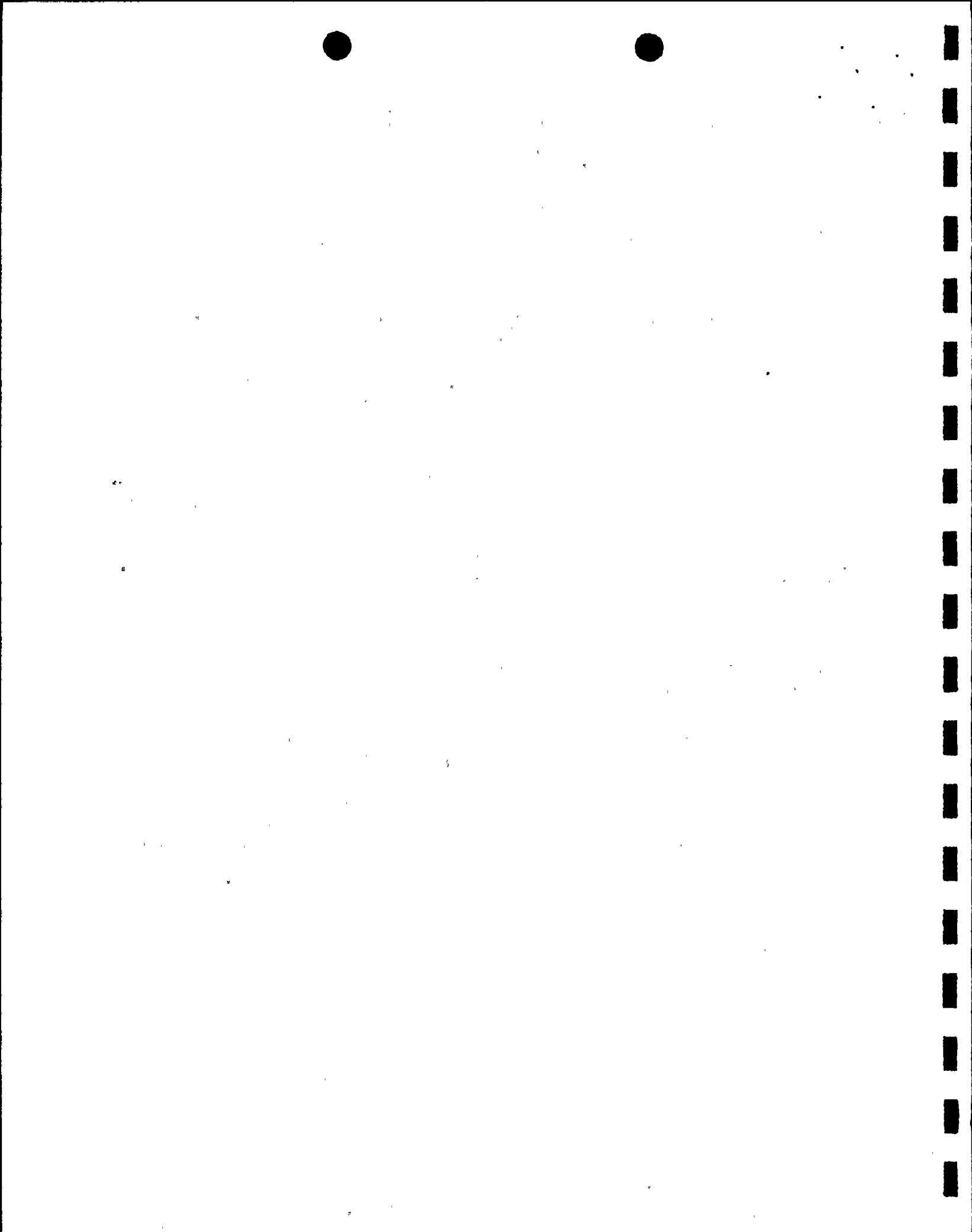
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Van Der Sluys, W.A., et al., "An Investigation of Mechanical Properties and Chemistry Within a Thick MnMoNi Submerged Arc Weldment," EPRI NP-373, Prepared by Babcock & Wilcox for the Electric Power Research Institute, Palo Alto, California, February 1977.

Letter from J. W. Williams, Florida Power & Light Company, to D. G. Eisenhut, Nuclear Regulatory Commission, Turkey Point Units 3 & 4, Docket Nos. 50-250 & 50-251, Pressurized Thermal Shock - Reactor Vessel Materials Data, dated February 10, 1984.

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Nanstad, R.K., and R.G. Berggren, "Irradiation Effects on Charpy Impact and Tensile Properties of Low Upper Shelf Welds, HSSI Series 2 and 3," NUREG/CR-5696, Prepared by Oak Ridge National Laboratory for the U.S. Nuclear Regulatory Commission, Washington, DC, August 1991.

Nanstad, R.K., et al., "Chemical Composition and RT_{NDT} Determinations for Midland Weld WF-70," NUREG/CR-5914, Prepared by Oak Ridge National Laboratory for the U.S. Nuclear Regulatory Commission, Washington, DC, December 1992.

Letter from S. A. Varga, Nuclear Regulatory Commission, to J. W. Williams. Florida Power & Light Company, Subject: Evaluation of Reactor Vessel Materials Data for Turkey Point Plant Units 3 and 4 Reactor Vessels, dated April 26, 1984.

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Chicots, J.M., et al., "Analysis of Capsule S from the Rochester Gas and Electric Corporation R.E. Ginna Reactor Vessel Radiation Surveillance Program," WCAP-13902, Westinghouse Electric Corporation, Pittsburgh, Pennsylvania, December 1993.

