

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9407200011      DOC. DATE: 94/07/13      NOTARIZED: NO      DOCKET #  
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G      05000244  
 AUTH. NAME      AUTHOR AFFILIATION  
 MARTIN, J.T.      Rochester Gas & Electric Corp.  
 MECREDY, R.C.      Rochester Gas & Electric Corp.  
 RECIP. NAME      RECIPIENT AFFILIATION

JOHNSON, A.R.      Document Control Branch (Document Control Desk)

SUBJECT: LER 94-008-00: on 940613, "B" safety injection pump was declared inoperable during monthly surveillance test. Caused by broken motor rotor bar. "B" SI pump motor rotor bars were replaced & "B" SI pump motor was reassembled. W/940713 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 9  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: License Exp date in accordance with 10CFR2, 2.109 (9/19/72).      05000244

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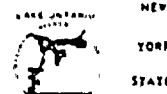
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ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER N.Y. 14649-0001

ROBERT C. MECREDY  
Vice President  
Ginna Nuclear Production

TELEPHONE  
AREA CODE 716 546-2700



July 13, 1994

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Attn: Allen R. Johnson  
Project Directorate I-3  
Washington, D.C. 20555

Subject: LER 94-008, "B" SI Pump Declared Inoperable Due to Broken Motor  
Rotor Bar, Causes Completion of a Plant Shutdown Required by Technical  
Specifications  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (i)  
(A), which requires a report of, "The completion of any nuclear plant shutdown required  
by the plant's Technical Specifications", the attached Licensee Event Report LER 94-008  
is hereby submitted.

This event has in no way affected the public's health and safety.

Very truly yours,

Robert C. Mecredy

xc: U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406

Ginna Senior Resident Inspector

CLF# 2074512 241

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## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) R.E. Ginna Nuclear Power Plant

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TITLE (4) "B" SI Pump Declared Inoperable Due to Broken Motor Rotor Bar, Causes Completion of a Plant Shutdown Required by Technical Specification

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
06	13	94	94	--008--	00	07	13	94	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)		097	20.402(b)			20.405(c)			50.73(a)(2)(iv)	73.71(b)
			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	OTHER
			20.405(a)(1)(iii)		X	50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	

## LICENSEE CONTACT FOR THIS LER (12)

NAME John T. St. Martin - Director, Operating Experience

TELEPHONE NUMBER (Include Area Code)  
(315) 524-4446

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	BQ	MO	W120	Y						

## SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
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## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On June 10, 1994, at approximately 1349 EDST, with the plant at approximately 97% steady state power, the "B" safety injection pump was declared inoperable during performance of the monthly surveillance test. On June 13, 1994, at approximately 1614 EDST, the reactor was placed in the hot shutdown condition to comply with Ginna Technical Specifications for safety injection pump operability requirements.

The underlying cause of the shutdown was inoperability of the "B" safety injection pump, due to a broken motor rotor bar. This event is NUREG-1022 Cause Code (B), "Design, Manufacturing, Construction/Installation."

Corrective action to preclude repetition is outlined in Section V (B).

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

## I. PRE-EVENT PLANT CONDITIONS

On June 10, 1994, the plant was at approximately 97% steady state reactor power. Results and Test personnel were performing the monthly surveillance test of the safety injection (SI) pumps, in accordance with surveillance test procedure PT-2.1M, "Safety Injection System Monthly Test." Testing of the "A" SI pump had just been completed with satisfactory results, and testing of the "B" SI pump was initiated. At approximately 1349 EDST, the "B" SI pump was started for the monthly test. When the motor was started, a few sparks were noted coming from the motor. (As the motor came up to speed, the sparks stopped.) Control Room operators were immediately notified of the sparking condition, and promptly deenergized the "B" SI pump motor. The "B" SI pump was then declared inoperable.

The declaration of inoperability placed the plant into a Technical Specification (TS) Limiting Condition for Operation (LCO) for TS 3.3.1.4. The action statement of TS 3.3.1.4 allows one SI pump to be inoperable for up to 72 hours. If the pump is not operable after 72 hours, the reactor shall be placed in hot shutdown within the following 6 hours and below a reactor coolant system (RCS) temperature of 350 degrees F within an additional 6 hours.

Immediate corrective maintenance was initiated. The "B" SI pump motor was disassembled and inspected. Initial visual inspection by plant and engineering personnel identified that several rotor bars showed crack indications, and one bar appeared to be fully cracked and broken. The rotor was packaged and shipped offsite for repairs.

## II. DESCRIPTION OF EVENT

## A. DATES AND APPROXIMATE TIMES OF MAJOR OCCURRENCES:

- o June 10, 1994, 1349 EDST: The "B" SI pump is declared inoperable.
- o June 13, 1994, 1115 EDST: Commenced a reactor shutdown to hot shutdown, and declared an Unusual Event.

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- o June 13, 1994, 1614 EDST: The reactor is in the hot shutdown condition, with all control rods inserted and both reactor trip breakers open. Event date and time.
- o June 13, 1994, 1614 EDST: Discovery date and time.
- o June 13, 1994, 2157 EDST: The plant is below RCS temperature of 350 degrees F.
- o June 13, 1994, 2159 EDST: Unusual Event terminated.

## B. EVENT:

On the morning of June 13, 1994, the "B" SI pump motor rotor was at an offsite motor repair facility. Detailed inspections performed at the offsite facility confirmed that one (1) bar was broken, and two (2) other bars were cracked. (This broken bar was the cause of the sparking observed during motor start.) Repairs were not able to be completed within the next several hours on June 13, nor was a qualified spare rotor located by this time. The Allowable Outage Time (AOT) of TS 3.3.1.4 required the reactor to be in hot shutdown before 1949 EDST on June 13, 1994, and below a RCS temperature of 350 degrees F before 0149 EDST on June 14. Plant management directed that the reactor be shut down and cooled down within the AOT of the TS LCO.

On June 13, 1994, at approximately 1115 EDST, with the reactor at approximately 97% steady state reactor power, the Shift Supervisor declared an Unusual Event, as required by the Ginna Station Nuclear Emergency Response Plan and procedure EPIP-1.0, "Ginna Station Event Evaluation and Classification." This was based on initiating condition "Loss of Engineered Safety Features", for the specific situation of "initiation of a plant shutdown required by Tech Specs due to loss of Engineered Safety Feature for ECCS". At this time, the Shift Supervisor directed the Control Room operators to initiate the shutdown and cooldown to meet the LCO of TS 3.3.1.4.

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The shutdown and subsequent cooldown were performed in accordance with normal operating procedures O-2.1, "Normal Shutdown to Hot Shutdown" and O-2.2, "Plant Shutdown from Hot Shutdown to Cold Condition." The shutdown proceeded in a normal manner, and the reactor was placed in the hot shutdown condition at approximately 1614 EDST, with all control rods inserted and both reactor trip breakers open. The RCS was cooled down below 350 degrees F at approximately 2157 EDST on June 13, 1994.

With the RCS below a temperature of 350 degrees F, the plant was in a stable condition with no TS LCO action statements in effect for the SI pumps. At approximately 2159 EDST, the Unusual Event was terminated in accordance with emergency plan implementing procedures.

C. INOPERABLE STRUCTURES, COMPONENTS, OR SYSTEMS THAT CONTRIBUTED TO THE EVENT:

The "B" SI pump was inoperable from approximately 1349 EDST on June 10, 1994, until approximately 1518 EDST on June 15, 1994.

D. OTHER SYSTEMS OR SECONDARY FUNCTIONS AFFECTED:

None

E. METHOD OF DISCOVERY:

The completion of the plant shutdown (the event) was preplanned and self-evident. The need for the reactor to be in hot shutdown before 1949 EDST on June 13, 1994, was recognized on June 10. On June 13, it was known that repairs would not be completed within the AOT of TS 3.3.1.4. This was the basis for initiating a reactor shutdown at approximately 1115 EDST on June 13. The completion of the shutdown at approximately 1614 EDST, required by TS, is reportable to the NRC.

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## F. OPERATOR ACTION:

The Control Room operators immediately secured the "B" SI pump by deenergizing the pump motor on June 10, and declared the pump inoperable. On June 13, an Unusual Event was declared. The Control Room operators shut down and cooled down the plant in accordance with normal operating procedures.

The Control Room operators notified the NRC of the initiation of the reactor shutdown and declaration of the Unusual Event at approximately 1151 EDST on June 13, as required by 10 CFR 50.72 (a) (1) (i) and 50.72 (b) (1) (i) (A). All appropriate agencies were notified of the declaration of the Unusual Event.

The Control Room operators notified the NRC and all appropriate agencies after the termination of the Unusual Event.

## G. SAFETY SYSTEM RESPONSES:

None

## III. CAUSE OF EVENT

## A. IMMEDIATE CAUSE:

The immediate cause of the reactor shutdown was the need to meet the TS LCO of TS 3.3.1.4 for one SI pump inoperable. The "B" SI pump had been declared inoperable at approximately 1349 EDST on June 10, 1994, and the AOT of TS 3.3.1.4 required that, with one SI pump inoperable, the reactor be placed in hot shutdown before 1949 EDST on June 13, 1994.

## B. INTERMEDIATE CAUSE:

The intermediate cause of the "B" SI pump being declared inoperable was due to sparks coming from the motor during motor starting.

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C. ROOT CAUSE:

The underlying cause of the sparks coming from the motor of the "B" SI pump was determined to be a broken motor rotor bar.

The cracking that leads to broken bars is due to fatigue caused by the number of stress cycles imposed on the rotor bars. The principal contributor to stress cycles in the motor rotor (squirrel cage) is the magnetic stresses experienced during motor starting. The frequency of motor starts could significantly reduce the expected service life of the motor rotors. This event is NUREG-1022 Cause Code (B), "Design, Manufacturing, Construction/Installation".

IV. ANALYSIS OF EVENT

This event is reportable in accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (i) (A), which requires a report of, "The completion of any nuclear plant shutdown required by the plant's Technical Specifications."

An assessment was performed considering both the safety consequences and implications of this event with the following results and conclusions:

- o There were no operational or safety consequences or implications attributed to the "B" SI pump motor rotor sparking because pump motor failure did not occur. In addition, the sparking did not continue after the start of the pump. The "B" SI pump motor was immediately deenergized and the pump was declared inoperable.
- o The plant continued to operate in accordance with the LCO of TS 3.3.1.4 for the next 72 hours. The cracked rotor bar was not repairable within this 72 hour AOT, so the reactor was placed in hot shutdown within the following 6 hours and below a RCS temperature of 350 degrees F within an additional 6 hours per TS. The reactor was operated within the requirements of the TS at all times.



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Based on the above, it can be concluded that the public's health and safety was assured at all times.

## V. CORRECTIVE ACTION

## A. ACTION TAKEN TO RETURN AFFECTED SYSTEMS TO PRE-EVENT NORMAL STATUS:

- o The "B" SI pump motor rotor bars were replaced at an offsite facility. The "B" SI pump motor was reassembled, tested, and returned to service.
- o The issue of motor rotor bar cracking was recently discussed in Westinghouse Technical Bulletin NSD-TB-94-01-R0, "Motor Rotor Bar Cracking", dated 03/17/94. Some Westinghouse Life-line A series and CSP series motors may exhibit rotor bar cracking. This Westinghouse Technical Bulletin (WTB) had been assessed as part of the Ginna Operational Assessment Program, prior to this event. Motors in use at Ginna, of the frame sizes of concern in the WTB, were identified and listed at the time of the assessment. This list of Ginna motors was referred to, and vibration analysis was conducted for selected motors. Results of this analysis indicated that the "C" SI pump had possible cracked rotor bars. The "C" SI pump motor was disassembled and visually inspected. No visual evidence of rotor bar cracking was identified. The "C" SI pump motor was reassembled, tested, and returned to service.

## B. ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE:

The Predictive Maintenance Program is being enhanced to use improved diagnostic vibration analysis methods. Diagnostic testing will be performed periodically to trend for motor inconsistencies, including rotor bar cracking.

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**VI. ADDITIONAL INFORMATION**

**A. FAILED COMPONENTS:**

The failed component is the rotor for the "B" SI pump motor. This motor is a Model ABDP, frame size 509US, 440 VAC, 3555 RPM, 400 amp, 350 horsepower motor, manufactured by Westinghouse Electric Corporation.

**B. PREVIOUS LERS ON SIMILAR EVENTS:**

A similar LER event historical search was conducted with the following results: No documentation of similar LER events with the same root cause at Ginna Nuclear Power Plant could be identified.

**C. SPECIAL COMMENTS:**

None