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ACCESSION NBR: 9302030096      DOC. DATE: 93/01/25      NOTARIZED: NO      DOCKET #  
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G      05000244  
 AUTH. NAME      AUTHOR AFFILIATION  
 BACKUS, W.H.      Rochester Gas & Electric Corp.  
 MECREDY, R.C.      Rochester Gas & Electric Corp.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 92-007-00: on 921224, turbine runback occurred & "A" EDG started & tied into safeguard buses 14 & 18. Caused by loss of power to safeguards. Restored normal power to safeguards buses 14 & 18. W/930125 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 11  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: License Exp date in accordance with 10CFR2,2.109(9/19/72).      05000244

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INTERNAL:	ACNW	2 2	AEOD/DOA	1 1
	AEOD/DSP/TPAB	1 1	AEOD/ROAB/DSP	2 2
	NRR/DET/EMEB 7E	1 1	NRR/DLPQ/LHFB10	1 1
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	NRR/DST/SRXB 8E	1 1	REG FILE 02	1 1
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EXTERNAL:	EG&G BRYCE, J.H	2 2	L ST LOBBY WARD	1 1
	NRC PDR	1 1	NSIC MURPHY, G.A	1 1
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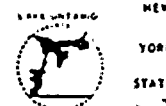
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January 25, 1993

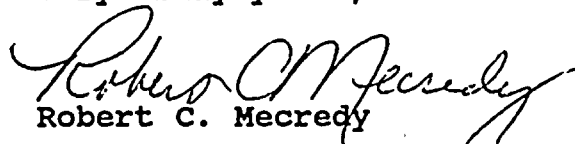
U.S. Nuclear Regulatory Commission  
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Washington, DC 20555

Subject: LER 92-007, Loss of Offsite Power Circuit 751, Due To  
Wind Storm Induced Debris, Causes Automatic Actuation  
of An ESF and RPS System (i.e. Emergency Diesel  
Generator Operation and Turbine Runback)  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report  
System, item (a)(2)(iv), which requires a report of, "any event  
or condition that resulted in manual or automatic actuation of  
any Engineered Safety Feature (ESF), including the Reactor  
Protection System (RPS)", the attached Licensee Event Report LER  
92-007 is hereby submitted.

This event has in no way affected the public's health and  
safety.

Very truly yours,

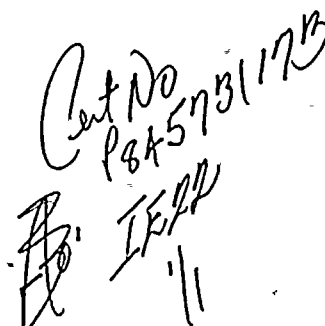
  
Robert C. Mecredy

xc: U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406

Ginna USNRC Senior Resident Inspector

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PDR ADOCK 05000244  
S PDR





## LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) R.E. Ginna Nuclear Power Plant DOCKET NUMBER (2) 0 5 0 0 0 2 4 4 PAGE (3) 1 OF 1 0

TITLE (4) Loss of Offsite Power Circuit 751, Due To Wind Storm Induced Debris, Causes Automatic Actuation of An ESF and RPS System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1	2	2	9	2	0	0	7	0	0	0
1	2	2	4	9	2	9	2	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9)	POWER LEVEL (10)	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vi)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(ix)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
N	01918								X									

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Wesley H. Backus Technical Assistant to the Operations Manager	3 1 5 5 2 4 4 4 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs	
X	E	A	C	O	N	X	9	9	9	N

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On December 24, 1992, at 0435 EST, with the reactor at approximately 98% steady state full power, a turbine runback occurred and the "A" Emergency Diesel Generator started and tied into safeguard buses 14 and 18.

The Control Room operators performed the appropriate actions of AP-TURB.2 (Automatic Turbine Runback) and verified that the "A" Emergency Diesel Generator was operating properly and that safeguards buses 14 and 18 were energized. The plant was subsequently stabilized at approximately 77% full power.

The cause of the event was the loss of power to safeguards buses 14 and 18 due to loss of offsite power 34.5 KV Circuit 751. The underlying cause for the loss of Circuit 751 was determined to be windstorm induced debris intercepting the shield wire and phase conductor causing a fault. (This event is NUREG-1022 (C) Cause Code).

Corrective action was to restore normal power to safeguards buses 14 and 18 from offsite power source 34.5 KV Circuit 767 and to stop the "A" Emergency Diesel Generator and realign it for auto standby.



**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  R.E. Ginna Nuclear Power Plant	DOCKET NUMBER (2)  0   5   0   0   0   2   4   4	LER NUMBER (6)			PAGE (3)		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

**I. PRE-EVENT PLANT CONDITIONS**

The plant was at approximately 98% steady state power with no major operational activities in progress.

The offsite power configuration to the plant was as follows:

- o Offsite power 34.5 KV Circuit 751 was feeding the "A" train safeguards buses 14 and 18 through 12A 34.5 KV/4160V transformer and through the safeguards buses 4160V/480V transformers.
- o Offsite power 34.5 KV Circuit 767 was feeding the "B" train safeguards buses 16 and 17 through 12B 34.5 KV/4160V transformer and through the safeguards buses 4160V/480V transformers.

The weather conditions in the area were high winds, blowing snow, and subfreezing temperatures.

**II. DESCRIPTION OF EVENT****A. DATES AND APPROXIMATE TIMES OF MAJOR OCCURRENCES:**

- o December 24, 1992, 0435 EST: Event date and time
- o December 24, 1992, 0435 EST: Discovery date and time
- o December 24, 1992, 0435 EST: Control Room operators verify the "A" Emergency Diesel Generator (D/G) Operation and safeguards buses 14 and 18 energized.
- o December 24, 1992, 0436 EST: Control Room operators perform the appropriate actions of abnormal procedure AP-TURB.2 (Automatic Turbine Runback) and stabilize the plant.
- o December 24, 1992, 0501 EST: Safeguard buses 14 and 18 resupplied from offsite power 34.5 KV Circuit 767.





LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  R.E. Ginna Nuclear Power Plant	DOCKET NUMBER (2)  0   5   0   0   0   2   4   4	LER NUMBER (6)			PAGE (3)		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

- o December 24, 1992, 0510 EST: The "A" Emergency D/G was stopped and realigned for auto standby.
- o December 28, 1992, 0920 EST: Offsite power 34.5 KV Circuit 751 declared operable.

## B. EVENT:

On December 24, 1992, at 0435 EST, with the reactor at approximately 98% steady state full power, the Control Room received numerous annunciator alarms. The Control Room operators immediately determined that the following events had occurred:

- o Safeguards buses 14 and 18 had lost their normal power supply.
- o A turbine runback of approximately 20% full power had occurred.
- o The "A" Emergency D/G had started and was tied to safeguards buses 14 and 18.

The Control Room operators immediately performed the appropriate actions of AP-TURB.2 (Automatic Turbine Runback), transitioned to AP-ELEC.1 (Loss of 12A And/Or 12B Transformer) and verified that the "A" Emergency D/G was operating properly and that safeguards buses 14 and 18 were energized. The plant was subsequently stabilized at approximately 77% full power. The Control Room operators observed that 4160V bus 12A and offsite power 34.5 KV Circuit 751 displayed zero (0) voltage. The Control Room operators performed the appropriate actions of AP-ELEC.1 and contacted Power Control. The Power Control Dispatcher informed them that offsite power 34.5 KV Circuit 751 had tripped at substation 204.



**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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R.E. Ginna Nuclear Power Plant

0500244

92-007-00

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Subsequently at 0452 EST, the Control Room operators, using Equipment Restoration procedure ER-ELEC.1 (Restoration of Offsite Power) energized 4160V Bus 12A from offsite power 34.5 KV Circuit 767. At 0501 EST safeguard buses 14 and 18 were resupplied by offsite power.

At 0510 EST, December 24, 1992, the "A" Emergency D/G was stopped and realigned for auto standby.

At 0920 EST, December 28, 1992, after offsite power 34.5 KV Circuit 751 had been cleared for use, normal offsite power from Circuit 751 was made available to the plant. The current operating alignment utilizing 34.5 KV Circuit 767 is being maintained to supply the safeguards trains. Offsite power 34.5 KV Circuit 751 was maintained as the plant's backup supply of offsite power.

**C. INOPERABLE STRUCTURES, COMPONENTS, OR SYSTEMS THAT CONTRIBUTED TO THE EVENT:**

None.

**D. OTHER SYSTEMS OR SECONDARY FUNCTIONS AFFECTED:**

None.

**E. METHOD OF DISCOVERY:**

The event was immediately apparent due to alarms and indications in the Control Room.



**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

**F. OPERATOR ACTION:**

Following the turbine runback, buses 14 and 18 under-voltage, and the "A" Emergency D/G automatic start, the Control Room operators immediately performed the appropriate actions of AP-TURB.2 (Automatic Turbine Runback) and verified that the "A" Emergency D/G was operating properly and that safeguards buses 14 and 18 were energized.

Subsequently, the Control Room operators restored normal power to bus 14 and 18 and stopped and realigned the "A" Emergency D/G for Auto standby.

**G. SAFETY SYSTEM RESPONSES:**

The "A" Emergency D/G automatically started, displayed proper voltage and frequency and closed into safeguards buses 14 and 18 to supply emergency power.

**III. CAUSE OF EVENT****A. IMMEDIATE CAUSE:**

The automatic turbine runback was caused by a dropped rod signal from the Nuclear Instrument System (NIS) power range N-42 and/or from the Microprocessor Rod Position Indication (MRPI) System.

The automatic actuation of the "A" Emergency D/G was due to undervoltage on safeguards buses 14 and 18.



LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

**B. INTERMEDIATE CAUSES:**

The dropped Rod signal from NIS power range N-42 was due to the momentary loss of the "B" instrument bus which supplied power to N-42. The loss of the "B" instrument bus was due to the momentary loss of MCC-1C which supplied power to the constant voltage transformer for the "B" instrument bus.

The dropped rod signal from the MRPI System was due to the momentary loss of MCC-1K which supplied power to the MRPI System. The loss of MCC-1K was due to the momentary loss of MCC-1C which supplied power to MCC-1K.

The loss of MCC-1C was due to the momentary loss of power to safeguards bus 14.

The loss of power to safeguards bus 14 was due to the loss of offsite power 34.5 KV Circuit 751.

The undervoltage on safeguards buses 14 and 18 was due to the loss of offsite power 34.5 KV Circuit 751.

**C. ROOT CAUSE:**

On January 15, 1993 34.5 KV Circuit 751 was patrolled using bucket trucks. At pole T-424 on Lake Road a burned phase conductor was found.

Based on the above the underlying cause of the loss of offsite power 34.5 KV Circuit 751 was determined to be windstorm induced debris intercepting the shield wire and phase conductor causing a fault. (Wind gusts of up to 33.5 mph)





LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

IV. ANALYSIS OF EVENT

The event is reportable in accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(iv), which requires reporting of, "any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF) including the Reactor Protection System (RPS)", in that the automatic turbine runback was an automatic actuation of the RPS and the starting of the "A" Emergency D/G was an automatic actuation of an ESF system.

An assessment was performed considering both the safety consequences and implications of this event with the following results and conclusions:

- o All reactor control and protection systems performed as designed thus limiting the overall effects of the turbine runback transient.
- o The "A" Emergency D/G operated as designed by starting and supplying emergency power to safeguards buses 14 and 18.
- o Offsite power 34.5 KV Circuit 767 was still in operation supplying power to the "B" train safeguards buses and subsequently was lined up to also supply power to the "A" train safeguards buses as permitted by plant technical specifications.

Based on the above, it can be concluded that the public's health and safety was assured at all times.



LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

V. CORRECTIVE ACTION

## A. ACTION TAKEN TO RETURN AFFECTED SYSTEMS TO PRE-EVENT NORMAL STATUS:

After the plant had been stabilized, safeguards buses 14 and 18 normal power supplies were restored via offsite power 34.5 KV Circuit 767 and the "A" Emergency D/G was stopped and aligned for auto standby.

Subsequently an extensive troubleshooting effort was undertaken to determine the root cause of offsite power 34.5 KV Circuit 751 tripping out at substation 204. This troubleshooting effort included the following:

- o Performing a visual and instrument survey along the entire length of offsite power 34.5 KV Circuit 751 between the plant and substation 204 to ascertain if there were any damaged insulators, tree limbs, or other debris on lines, or evidence of anything that had possibly fallen across the line and burned free. No abnormal conditions were detected.
- o Inspecting substation 204 for any abnormal conditions (i.e. breakers, wires, monitoring equipment, etc.). No abnormal conditions were detected, except a distance monitor did indicate a problem had occurred at a certain distance from Station 204. This area was checked out with no abnormal conditions detected.



LICENSEE EVENT REPORT (LER)  
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TEXT (If more space is required, use additional NRC Form 368A's) (17)

- o Placing a high potential test on Circuit 751 (i.e. approximately 70 KV) to identify possible intermittent abnormal conditions. One abnormal condition was discovered on wire connecting links of two phases of the Circuit. These links were found to have pinholes in the jacket covering. Rochester Gas and Electric Corporation (RG&E) Transmission and Distribution Department believes these pinholes were caused by the high potential test placed on the line. No correlation between the pinholes and the original initiating event could be drawn. The links were subsequently repaired and retested satisfactorily. No other abnormal conditions were detected.
- o On December 28, 1992 at 0920 EST offsite power 34.5 KV. Circuit 751 was declared operable and cleared for use.

**B. ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE:**

As the underlying cause was determined to be windstorm induced debris intercepting the shield wire and phase conductor causing a fault, the following corrective action was taken to prevent recurrence:

- o Tree trimming efforts for 34.5 KV Circuit 751 were reviewed and additional trimming was completed as of January 11, 1993 at pole T-424 on Lake Road and T-25A and T-26A on Slocum Road.

The RG&E Transmission and Distribution Department will continue their practice of Aerial patrol of 34.5 KV Circuit 751 twice per year utilizing thermal imaging and visual assessment of encroachments and general line integrity. Based on those assessments appropriate maintenance activities will be performed.



LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

VI. ADDITIONAL INFORMATION

## A. FAILED COMPONENTS:

The wire connecting links are of unknown origin with no part number or model number.

## B. PREVIOUS LERs ON SIMILAR EVENTS:

A similar LER event historical search was conducted with the following results: No documentation of similar LER events with the same root cause at Ginna Station could be identified. However, LER 91-002 was a similar event with a different root cause.

## C. SPECIAL COMMENTS:

None.

