

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9209090062 DOC. DATE: 92/08/31 NOTARIZED: NO DOCKET #
FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
AUTH. NAME AUTHOR AFFILIATION
MECREDY, R. C. Rochester Gas & Electric Corp.
RECIP. NAME RECIPIENT AFFILIATION
JOHNSON, A. R. Project Directorate I-3

SUBJECT: Submits list of unanticipated difficulties which will delay
submittal of final results of PRA to 930801.

DISTRIBUTION CODE: A011D COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2
TITLE: Generic Ltr 88-20 re Individual Plant Evaluations

NOTES: License Exp date in accordance with 10CFR2.2.109(9/19/72). 05000244

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD1-3 PD	1 1	JOHNSON, A	1 1
INTERNAL:	ACRS HOUSTON, M.	1 1	AEOB/DSP/TPAB	1 1
	NRR HERNAN, R.	1 1	NRR/DREP/PRAB	1 1
	REG 1	1 1	REG 2	1 1
	REG 3	1 1	REG 4	1 1
	REG FILE	1 1	RES FLACK, J.	6 6
	RES MITCHELL, J.	1 1	RES/DSIR/SAIB/B	7 7
EXTERNAL:	NRC PDR	1 1	NSIC	1 1

Cont NO
p034044130

NOTE TO ALL "RIDS" RECIPIENTS:

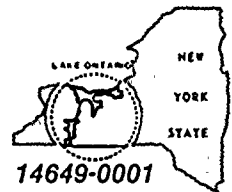
PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK.
ROOM P1-37 (EXT. 504-2065) TO ELIMINATE YOUR NAME FROM DISTRIBUTION
LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 27 ENCL 27





ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER N.Y. 14649-0001



ROBERT C. MECREDY
Vice President
Ginna Nuclear Production

TELEPHONE
AREA CODE 716 546-2700

August 31, 1992

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Generic Letter 88-20
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

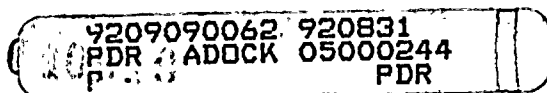
Dear Mr. Johnson:

In RG&E's October 27, 1989 response to Generic Letter 88-20, Supplement 1, we noted that we would anticipate submitting the final results of our Probabilistic Risk Assessment to the NRC on or before September 1, 1992. This schedule is not able to be met, based on a number of unanticipated difficulties:

- 1) The intensive nature of data-gathering to support Ginna-specific accident sequences and mitigation alternatives required substantially more effort than expected;
- 2) Personnel changes, both within RG&E and our contractor, required substantial resource reallocation and additional training time;
- 3) Use of plant-specific component failure and demand data, as well as selection and quantification of Ginna-specific initiating events, has required that additional resources be allocated to our PRA staff;
- 4) Additional engineering, training, operations, maintenance, and licensing personnel are being used in review efforts in order to most accurately reflect the current Ginna Station configuration.
- 5) Development of a state of the art component/sub-component data base that will be useful in future fire analyses as well as the current internal flooding analyses has required significant in house engineering resources beyond the original project estimates; and,
- 6) Conduct of the project within a quality assurance program has caused development of a project support structure that is far more extensive than original estimates.

Cont No P03A044-130

*A011
10*

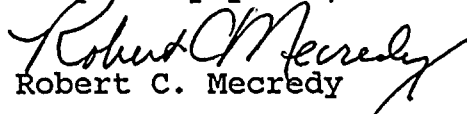


040013

040013

This additional RG&E and contractor work has extended the duration of our focused Level 2 PRA. We now anticipate our draft report to be generated by April 29, 1993, with a submittal to NRC by August 1, 1993. We are continuing to expend significant personnel and monetary resources to assure an accurate and detailed risk assessment of Ginna Station. We have currently completed our Level 1 PRA to the point of beginning the final quantification process. We have also made significant progress on Level 2. Containment structural analysis is complete, and the containment event trees are awaiting the results of Level 1 quantification. Please note that, to date, we have not identified any risk outliers or problems requiring extensive modifications or other changes; rather, the additional work effort and time will be used to complete a state of the art living PRA that can be of considerable future value in ensuring the continued safe, efficient operation of Ginna. We will continue to keep you apprised of our progress in this area.

Very truly yours,


Robert C. Mecredy

GJW/249

xc: Mr. Allen R. Johnson (Mail Stop 14D1)
Project Directorate I-3
Washington, D.C. 20555

U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

US NRC Ginna Senior Resident Inspector

James W. Smith