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 MECREDY,R.C. Rochester Gas & Electric Corp. *See*  
 RECIP.NAME RECIPIENT AFFILIATION *BAW-2166*  
 JOHNSON,A.R. Project Directorate I-3

SUBJECT: Forwards response to GL 92-01,Rev 1 re reactor vessel  
 structural integrity.B&W Rept BAW-2166, "B&WOG Response to  
 GL 92-01" also encl.

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 TITLE: Generic Letter 92-01 Responses (Reactor Vessel Structural Integrity 1

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July 2, 1992

U.S. Nuclear Regulatory Commission  
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Washington, D.C. 20555

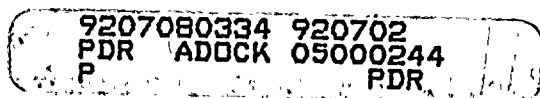
Subject: Reactor Vessel Structural Integrity, 10CFR50.54(f)  
Response to Generic Letter 92-01 Revision 1  
R. E. Ginna Nuclear Power Plant  
Docket No. 50-244

Dear Mr. Johnson:

Generic Letter 92-01 was issued to obtain information needed to assess compliance with requirements and commitments regarding reactor vessel integrity in view of certain concerns raised in the (NRC) staff's review of reactor vessel integrity for the Yankee Nuclear Power Station. The requirements are set forth in Appendix G and H of 10CFR Part 50. In 10 CFR 50.61 the NRC also provided fracture toughness requirements for protecting pressurized water reactors against pressurized thermal shock events. RG&E has also committed, Reference (a), in response to Generic Letter 88-11, "NRC Position on Radiation Embrittlement of Reactor Vessel Materials and its Impact on Plant Operations", to utilize the methodology in Regulatory Guide 1.99 Revision 2, "Radiation Embrittlement of Reactor Vessel Materials", to predict the effects of neutron irradiation as required by Paragraph V.A of 10 CFR 50 Appendix G.

For the Ginna Plant a discussion and evaluation of the Reactor Vessel Materials Program, Fracture Toughness Requirements for Protection against Pressurized Thermal Shock Events (10 CFR 50.61), and Charpy Upper Shelf Energy (USE) has been documented in Reference (b). The NRC evaluation of the Technical Specification Pressure/Temperature limits relative to the neutron irradiation embrittlement on each beltline material in the reactor vessel is documented in Reference (c). The RG&E Reactor Vessel Life Attainment Plan, Reference (d), contains a description and results concerning Irradiation Embrittlement Prediction and Ginna Specific Evaluation of Pressurized Thermal Shock, Flux Reduction Goal, and Upper Shelf Fracture Toughness.

RG&E has been a member of the B&W Owners Group and participated in the preparation of the Owners Group response to Generic Letter 92-01 for the 16 plants in the B&W Owners Group Reactor Vessel Working Group. The B&W Owners Group Response to Generic Letter 92-01 has been submitted to the NRC in Report BAW-2166 by Reference (e), and



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contains individual responses to the Generic Letter for each of these plants including the Ginna Plant.

The individual Generic Letter 92-01 requests and our response to each request is included in Attachment A. Also enclosed herewith (Attachment B) is a copy of that portion of BAW-2166 that is applicable to the Ginna Plant.

Very truly yours,

  
Robert C. Mecredy

GAH\232

Attachment

Subscribed and sworn to before me  
on this 2nd day of July, 1992



MARIE C. VILLENEUVE  
Notary Public, State of New York  
Monroe County  
Commission Expires October 31, 1992

xc: Mr. Allen R. Johnson (Mail Stop 14D1)  
Project Directorate I-3  
Washington, D.C. 20555

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US NRC Ginna Senior Resident Inspector

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