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SUBJECT: Forwards table listing projected reactor temp PTS for reactor vessel beltline matls,per Rev 2 to Reg Guide 1.99. Info on table indicates that reactor temp PTS maintained below screening criteria for all cases up to 32 EFPYs.

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July 30, 1991

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: Regulatory Guide 1.99, Revision 2 - PTS Screening
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Johnson:

Recently attention has been focused on the issue of reactor vessel weld materials and their susceptibility to neutron embrittlement. The recent revision to 10CFR50.61 requires the use of Regulatory Guide 1.99, Revision 2 to perform the calculations of the projected RT_{PTS} for the Ginna Station reactor vessel beltline materials. Although no specific actions are required for Ginna Station at this time, Westinghouse has performed these calculations for RG&E and the results are being submitted for your information. The attached Table 1 and Figure 1 are extracted from the "Rochester Gas and Electric Reactor Vessel Life Attainment Plan," March 1990. Table 1 indicates that the RT_{PTS} is maintained below the PTS screening criteria, as required, for all cases up to 32 Effective Full Power Years.

Very truly yours,

Robert C. Mecredy

Robert C. Mecredy

GJW/179
Attachment

xc: Mr. Allen R. Johnson (Mail Stop 14D1)
Project Directorate I-3
Washington, D.C. 20555

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Ginna Senior Resident Inspector

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TABLE 1
PROJECTED RT_{PTS} FOR THE R. E. GINNA REACTOR VESSEL BELTLINE MATERIALS

COMPONENT	EFPY	PROJECTED FLUENCE (1,2)	RT_{PTS} (REG. GUIDE 1.99, REV. 2) (°F)	PTS SCREENING CRITERIA (°F)
INT. SHELL FORGING	27.36	2.93	111	270
LOW SHELL FORGING	27.36	2.93	114	270
CIRC. WELD	27.36	2.93	283	300
INT. SHELL FORGING	32	3.38	112	270
LOW SHELL FORGING	32	3.38	115	270
CIRC. WELD INT. SHELL	32	3.38	288	300

(1) Fluence is in 10^{19} n/cm^2.

(2) Projected fluence from Congedo, T. V., Heinecke, C. C., Rens, T. E., Weaver, M., "R. E. Ginna Reactor Vessel Fluence and RT_{PTS} Evaluation," WCAP=11026, December 1985.

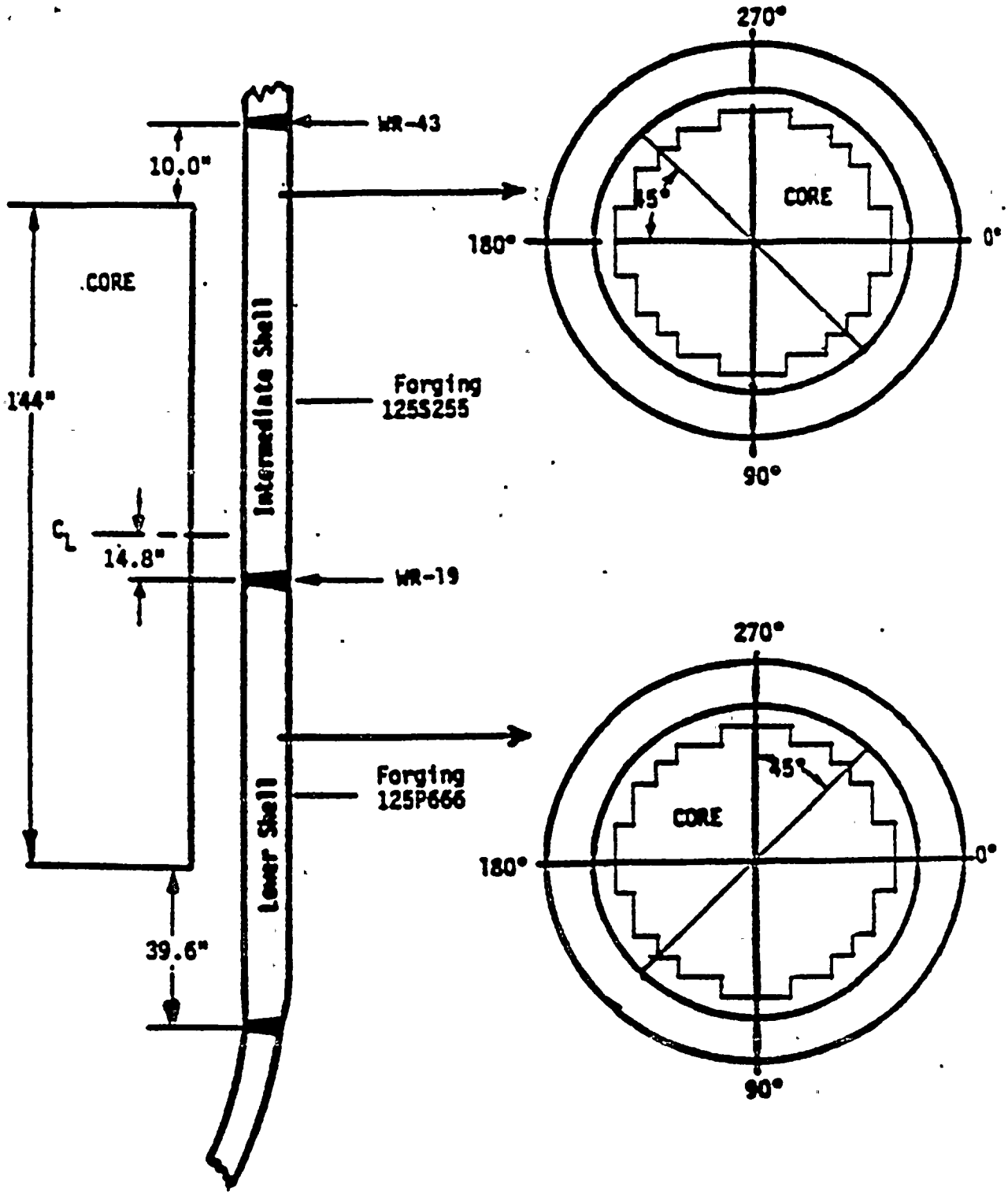

FIGURE 1

IDENTIFICATION AND LOCATION OF BELTLINE REGION MATERIAL FOR THE GINNA REACTOR VESSEL