

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9004130181 DOC.DATE: 90/03/28 NOTARIZED: NO DOCKET #  
 FACIL:50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244  
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 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 90-002-00:on 900226,fire watch patrol hourly patrol  
 performed in wrong areas due to personnel error.

W/9 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:License Exp date in accordance with 10CFR2,2.109(9/19/72). 05000244

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March 23, 1990

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555


Subject: LER 90-002, Fire Watch Patrol Performed Technical  
Specification Hourly Patrol in the Wrong Areas Due to  
Personnel Error.  
R. E. Ginna Nuclear Power Plant  
Docket No. 50-244

In accordance with 10CFR50.73, Licensee Event Report System, Item  
(a)(2)(i)(B), which requires reporting of "Any Operation Or  
Condition Prohibited by the Plant's Technical Specifications", the  
attached Licensee Event Report LER 90-002 is being submitted.

This LER is being submitted because an Hourly Fire Watch Patrol  
patrolled the wrong area and did not meet the Ginna Technical  
Specification 3.14 requirements for hourly patrol for two (2)  
Technical Specifications systems.

This event in no way affected the public health and safety.

Very truly yours,

  
Robert C. Mecredy  
Division Manager  
Nuclear Production

xc: U. S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406

Ginna USNRC Senior Resident Inspector

9004130181 900328  
PDR ADOCK 05000244  
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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
R. E. Ginna Nuclear Power Plant	0 5 0 0 0 2 4 4	9 0	- 0 0 2	- 0 0	0 2	OF	0 5

TEXT (If more space is required, use additional NRC Form 306A's) (17)

I. INITIAL PLANT CONDITION

The plant was at approximately 98% steady state full power with no major activities in progress.

II. DESCRIPTION OF EVENT

## A. DATES AND APPROXIMATE TIMES FOR MAJOR OCCURRENCES:

- February 26, 1990, 1544 EST: Discovery date and time.
- February 26, 1990, 1551 EST: Fire Watch correctly performed hourly patrol of "A" and "B" Battery Rooms, in compliance with Technical Specifications.
- February 26, 1990, 1744 EST: Fire Detection System which was inoperable due to construction activities was restored to service.

## B. EVENT:

- On February 26, 1990, at approximately 1544 EST with the Reactor at approximately 98% full power, it was determined by the Fire Watch Patrol that she was touring the "A" and "B" Diesel Generator Rooms instead of the "A" and "B" Battery Rooms.
- On February 26, 1990, at approximately 1551 EST, a Fire Watch was sent to the "A" and "B" Battery Rooms to perform the hourly tour.

## C. INOPERABLE STRUCTURES, COMPONENTS OR SYSTEM THAT CONTRIBUTED TO THE EVENT:

- Inoperable Fire Detection System - Pyrotronics

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

## D. OTHER SYSTEMS OR SECONDARY FUNCTIONS AFFECTED:

None

## E. METHOD OF DISCOVERY:

This event was discovered by the responsible Fire Watch during an afternoon turn-over meeting.

## F. OPERATOR ACTION:

The control room operators were notified by the Safety Coordinator at which time they performed the following:

- . Initiated an A-25.1 Ginna Station Event Report
- . Notified Duty Engineer
- . Notified Shift Technical Advisor (STA)
- . Notified Plant Superintendent
- . Notified Nuclear Regulatory Commission Resident Inspector

## G. SAFETY SYSTEM RESPONSES:

. None

III. CAUSE OF EVENT

## A. IMMEDIATE CAUSE:

Technical Specifications fire detection system was not Fire Watch toured hourly.

## B. ROOT CAUSE:

The root cause was determined to be unintentional human error.

IV. ANALYSIS OF EVENT

Hourly Fire Watch tours of two (2) Technical Specifications areas with inoperable fire detection systems were not conducted so Technical Specifications 3.14.1 and 3.14.1.1 were not met. This event is reportable in accordance with 10CFR50.73, Licensee Event Report System, Item (a)(2)(i)(B), which requires reporting of, "Any Operation Or Condition Prohibited by the Plant's Technical Specifications".

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)			PAGE (3)		
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R. E. Ginna Nuclear Power Plant	0 15 0 0 0 24 4	9 0	- 0 0 2	- 0 0	0 4	OF	05

TEXT (If more space is required, use additional NRC Form 366A's) (17)

An assessment was performed considering the safety of this event with the following results and conclusions:

There were no operational or safety consequences or implications attributed to the inoperable fire detection system because:

Inoperable Fire Detection System "A" Battery Room:

The fire area for the "A" Battery Room has a maximum permissible fire load of 160,000 BTU's/Sq. Ft. with an actual fire load of 44,021 BTU's/Sq. Ft. The actual fire load of 44,021 BTU's/Sq. Ft. is considered low (<80,000 BTU's/Sq. Ft.). The wall separating the "A" Battery Room from the Turbine Building is a 2 hour rated wall. The wall separating the "A" Battery Room from the "B" Battery Room is a 2 hour rated wall. The wall separating the "A" Battery Room from the Air Handling Room is a 2 hour rated wall.

Inoperable Fire Detection System "B" Battery Room:

The fire area for the "B" Battery Room has a maximum permissible fire load of 160,000 BTU's/Sq. Ft. with an actual fire load of 52,656 BTU's/Sq. Ft. The actual fire load of 52,656 BTU's/Sq. Ft. is considered low (<80,000 BTU's/Sq. Ft.). The wall separating the "B" Battery Room from the Turbine Building is a 2 hour rated wall. The wall separating the "B" Battery Room from the "A" Battery Room is a 2 hour rated wall.

These two fire areas/zones (BR1A-1 and BR1B-1) have fire extinguishing equipment within the area and there are hose stations adjacent to them.

The fire detection system for the area adjacent to the "A" and "B" Battery Room in the Turbine Building basement was operable.

The fire suppression system for the area adjacent to the "A" Battery Room in the Air Handling Room was operable.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES: 8/31/85

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TEXT (If more space is required, use additional NRC Form 364A's) (17)

No work was performed in either of the "A" or "B" Battery Rooms.

Based on the above, it can be concluded that the public's health and safety was assured at all times.

## V. CORRECTIVE ACTION

## A. ACTION TAKEN TO RETURN INOPERABLE FIRE DETECTION SYSTEM TO OPERABLE STATUS:

- Fire System reconnect was performed under plant procedure SC-3.16.2.4 Fire Signaling System/Component Disconnection - Reconnection
- System declared operable on February 26, 1990 at 1744 EST.

## B. ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE:

- As the root cause was determined to be a personnel error, the Fire Watch Supervisor has been instructed to follow-up the assignments given to fire watches to ensure they are going to the right area. Index cards have also been issued to the Fire Watch to ensure they have written instructions on where their patrols are to be conducted.

VI. ADDITIONAL INFORMATION

## A. FAILED COMPONENTS:

None

## B. PREVIOUS LER'S ON SIMILAR EVENTS:

A similar LER event historical search was conducted with the following results: No documentation of similar LER events with the same root cause at Ginna Station could be identified.

## C. SPECIAL COMMENTS:

During the five (5) hours in which the assigned Fire Watch inspected the incorrect area, Security entered the area of concern ten (10) times.