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MECREDY, R.C. Rochester Gas & Electric Corp.
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JOHNSON, A.R. Project Directorate I-3

SUBJECT: Clarifies commitment in 890724 response to NRC Bulletin
88-004, "potential Safety-Related Pump Loss."

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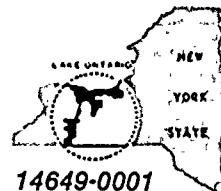
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March 12, 1990

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U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Allen R. Johnson
Project Directorate I-3
Washington, D.C. 20555

Subject: NRC Bulletin 88-04, Potential Safety-Related Pump Loss
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Reference: (a) Letter from B.A. Snow (RG&E) to C. Stahle (NRC),
dated July 7, 1988, Response to NRC Bulletin 88-
04, Potential Safety-Related Pump Loss

Dear Mr. Johnson:

The purpose of this letter is to clarify a commitment made in our July 24, 1989 letter as related to NRC Bulletin 88-04. In that letter RG&E stated that the interim modification completed during the 1989 Refueling Outage, which consisted of the installation of independent 200 gpm recirculation lines for the RHR pumps, removed the concern expressed in the Bulletin of deadheading one pump due to strong/weak pump interaction. RG&E also stated that the modification included provisions at a later time to further increase the recirculation flow capability by installation of a 450 gpm Recirculation System utilizing control valves. This was anticipated to be completed by July 1, 1990. The purpose of this latter phase of the modification was to reduce the potential for long term accelerated wear due to low flow operation during monthly periodic testing. It was established that this was not a required course of action based upon the evaluation performed for Bulletin 88-04 submitted as Attachment B of Reference (a), and the safety evaluation supporting the as-installed modification.

The implementation of this latter phase of this modification has been delayed. The reason for this schedule change is RG&E's plan to perform a revised 10CFR50.46 ECCS analysis that incorporates the dynamic thermal-hydraulic flow phenomena of the Upper Plenum Injection characteristics of the RHR System. This type of analysis has previously been completed for the Point Beach and Prairie Island nuclear power plants, and is underway for Kewaunee Station. This revised analysis incorporates Best Estimate methodology per SECY-83-472, and is expected to lower the accident analysis flowrate requirements of the RHR System. Thus, until the analysis is complete, the optimum RHR recirculation flow will not be known.

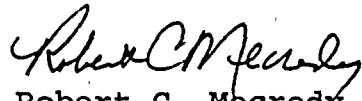
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This revised analysis is presently scheduled for a mid-1990 start date, and is targeted for completion and submittal to the NRC by mid-1991.

As RG&E stated in our July 24, 1989 letter, the strong/weak pump interaction concerns of Bulletin 88-04 have been resolved by RG&E's as-installed modification. We have also established that the 450 gpm Recirculation System previously described is not a required course of action under Bulletin 88-04. Therefore, we are removing this as a commitment to the NRC and the NRC's TAC No. 69918, closed out by the August 16, 1989 SER on Bulletin 88-04, can be considered to remain closed.

Very truly yours,



Robert C. Mecredy
Division Manager
Nuclear Production

GJW\085
Attachment

xc: Mr. Allen R. Johnson (Mail Stop 14D1)
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Washington, D.C. 20555

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Ginna Senior Resident Inspector