QUALITY ASSURANCE MANUAL GINNA STATION ROCHESTER GAS & ELECTRIC CORPORATION		REV. 0	PAGE 1 OF 111
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APPENDIX B GINNA NUCLEAR POWER STATION INSERVICE INSPECTION PROGRAM FOR THE 1990-1999 INTERVAL	PREPARED BY:	Michael J. Saporito	7-2?-89
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Table of Contents

Table of Contents (Con't)

1.0 INTRODUCTION

1.1 General

This appendix to the Quality Assurance Manual outlines the third interval inservice inspection examination (ISI) requirements for Class 1, Class 2, Class 3 systems and components for Rochester Gas & Electric Corporation's (RG&E) R.E. Ginna Nuclear Power Station (Ginna Station). The third inspection interval begins on January 1, 1990, as permitted by Paragraph IWA-2400 of ASME Code Section XI, the second interval concluded at the end of December 1989.

1.1.1 This program was developed in accordance with the documents identified in Section 1.2 and provides summary information outlining the ISI program for R.E. Ginna for the third inspection interval. This information includes:

a. Class 1, Class 2, and Class 3 systems, Component Supports, High Energy Piping Outside of Containment, Snubbers, Steam Generator Tubes and Tube Sleeve Combinations, Reactor Coolant Flywheels and System Pressure Testing.

b. ASME Section XI examination requirements of Subsections IWA, IWB, IWC, IWD, IWF. Inservice Testing of pumps (IWP) and valves (IWV) is performed in accordance with Appendix C of the Ginna Station Quality Assurance Manual.

   Inservice Inspection and Testing of snubbers is performed in accordance with 1.11 and 5.0 of this Appendix.

c. Method and extent of nondestructive examinations

d. Exemptions

e. Relief requests

f. Regulatory Guides

g. Code Cases

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h. Special requirements

1.1.2 This program is based on the requirements of the 1986 Edition of the American Society for Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Inservice Inspection of Nuclear Power Plant Components" as adopted by the Code of the Federal Regulations, 10 CFR Part 50, May 5, 1988.

1.1.3 This program excludes the controls of the Authorized Nuclear Inservice Inspector, Enforcement Authority, Reporting System and N-Stamp.

1.2 Applicable Documents

RG&E has adopted the following documents as the basis for the third inspection interval and is committed to satisfying their requirements. Any exceptions to these documents are identified and located in the "Relief Requests", Section 7.0 of this document. This program was developed in accordance with these documents:

1.2.1 ASME Boiler and Pressure Vessel Code Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," 1986 Edition.

1.2.2 U.S. Nuclear Regulatory Commission (USNRC) Regulatory Guides

a. 1.14, Rev. 1, "Reactor Coolant Pump Flywheel Integrity"

b. 1.147, Latest Revision, "Inservice Inspection Code Case Acceptability - ASME Section XI, Division 1"

c. 1.150, Rev. 1, "Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations"

d. 1.83, Rev. 1, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes

e. 1.29, Rev. 3, "Seismic Design Classification".

1.2.3 ASME Code Cases

In accordance with 10CFR50.55a, Footnote 6, ASME Section XI Code Cases referenced in Regulatory Guide 1.147, "Inservice Inspection Code Case Acceptability-ASME Section XI, Division 1," may be incorporated into the R.E. Ginna ISI Program. Those Code Cases incluled in Regulatory Guide 1.147 that will be implemented at R.E. Ginna are identified in this section. Each Code Case is preceded by information on the applicable component/area, ASME requirements, and how the Code Case will be implemented.

Paragraph 1.2.3.1 lists those code cases that have been technically reviewed and endorsed, with our without conditions by the NRC in Regulatory Guide 1.147 and will be implemented during the 1990-1999 Inspection Interval.

Use of any subsequent NRC endorsed code cases that are identified in revisions to the Regulatory Guide 1.147 may be incorporated in the program and used during the 1990-1999 Inspection Interval.

1.2.3.1 USNRC Regulatory Guide 1.147 - Approved ASME Section XI Code Cases

Code Case No.	Sect. XI References	Component/Area
N 307-1	IWB-2500-1	Studs and Bolts with Heater Holes
N-416	IWA-4400	Any repaired or replaced Class 2 piping component that cannot be isolated by valves or requires securing safety/ relief valves.
N-401	IWA-2233	Eddy Current Examinations

Code Case No.	Sect. XI References	Component/Area
N-402	IWA-2233	Eddy Current Calibration Standard Material
N-427*	Code Cases	Code Cases in inspection plans.
N-437	IWA-5260	Pressure Measuring Instrumentation used during Class 1, 2,and 3 Pressure Tests.

* As amended by USNRC Regulatory Guide 1.147, May 1988.

1.2.4 R.E. Ginna Updated Final Safety Analysis Report (UFSAR) Section 5.4 - For Class 1

Section 6.6 - For Class 2 and 3

1.2.5 Letter dated March 23, 1981, from Darrell G. Eisenhut, Director, Division of Licensing, USNRC, regarding Technical Specification Revisions for Snubber Surveillance

1.2.5.1 First Addeneda to ASME/ANSI OM-1987, Part 4 Published in 1988.

1.2.6 USAS B31.1.0-1967, "Power Piping" for High Energy Systems

1.2.7 R.E. Ginna Technical Specification 4.2. "Inservice Inspection".

1.2.8 R.E. Ginna Technical Specification 3.13 and 4.14 "Snubbers".

1.2.9 Letter dated February 16, 1989, to Mr. Carl Stahle, USNRC, PWR Project Directorate No. 1, regarding proposed changes to Snubber Technical Specification R.E. Ginna Nuclear Power Plant Docket No. 50-244.

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1.2.10 Rochester Gas and Electric Corporation Mechanical Engineering Specification, ME-256, Titled - "Snubber Inspection and Test Program".

1.3 Inspection Intervals

1.3.1 The inservice inspection intervals for Class 1 components started on January 1, 1970, with the second interval staring on January 1, 1980. The third inspection interval shall start on January 1, 1990.

1.3.2 For Class 2 and Class 3, the first inspection interval started on May 1, 1973; the second on January 1, 1980; and the third shall start on January 1, 1990. The fourth inspection interval for Class 1, Class 2 and Class 3 will follow on January 1, 2000, in accordance with the edition of Section XI applicable at that time.

1.3.3 A 10-year examination plan (Supplement to Appendix B), will describe the distribution of examinations for Class 1, Class 2, and Class 3 components in accordance with the IWB-2400, IWC-2400, IWD-2400 and IWF-2400 of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," 1986 edition.

1.3.4 As allowed by IWA-2430 each inspection interval may be decreased or extended (but not cumulatively) by as much as one year. If R.E. Ginna is out of service continuously for 6 months or more, the inspection interval and associated period during which the outage occurred may be extended for a period of time equivalent to the outage.

1.4 Classification of Components

The Program Plan components and piping have been classified by RG&E for purposes of inservice inspection based on definitions contained in 10CFR50.2 and in Article IWA-9000 of Section XI.

1.5 Responsibility

As specified in Paragraph IWA-1400 of ASME Section XI, RG&E bears the overall responsibility for implementation of an ISI program. Administrative Procedures, NDE Procedures, ISI Plans and Schedules are in place to control and implement these inservice inspection requirements.

1.6 Records

Examination records and documentation of results provide the basis for evaluation and facilitate comparison with previous results and subsequent inspections. In accordance with Section XI, IWA-6000, these records will be maintained for the plant life.

1.7 Examination Methods

Examination methods which will be used to satisfy Code examination requirements have been listed for nonexempt Class 1, Class 2, and Class 3 components, as applicable.

Provided below is a brief explanation of the examination methods which will be performed to satisfy the Code requirements.

Personnel performing nondestructive examinations will be qualified in accordance with written procedures prepared as required by Paragraph IWA-2300 of Section XI. Methods of examination are also described in the applicable sections of this Inservice Inspection Program.

1.7.1 Visual Examination Method

Visual examinations (VT) will be performed in accordance with IWA-2210 of ASME Section XI. IWA-2210 defines the three types of VT examinations as follows:

1.7.1.1 VT-1 examinations are conducted to determine the condition of the part, component or surface examined. The examination shall determine conditions such as cracks, wear, corrosion, erosion, or physical damage on the surfaces of the part or components. This type of examination may be performed by direct or remote methods as defined in IWA-2211.

1.7.1.2 VT-2 examinations are conducted to detect leakage (or abnormal leakage) from pressure-retaining components during system pressure or functional tests as defined in IWA-2212.

1.7.1.3 VT-3 examinations are conducted to determine general mechanical and structural conditions of components and their supports such as the presence of loose parts, debris, or abnormal corrosion products, wear, erosion, corrosion, and the loss of integrity at bolted or welded connections. Examinations are also conducted to determine conditions related to operability of components or devices, such as mechanical and hydraulic snubbers, components, supports, pumps, valves and spring-loaded and constant-weight hangers.

1.7.2 Surface Examination Method

1.7.2.1 A surface examination is performed to detect the presence of discontinuities open to the surface of a material. Techniques for surface examination include either magnetic particle (MT) or liquid penetrant (PT) methods. Surface examinations will be conducted as defined in IWA-2220.

1.7.3 Volumetric Examination Method

1.7.3.1 A volumetric examination is performed to detect the presence of discontinuities in the volume of a material. Such volumetric examinations include radiographic (RT), ultrasonic (UT), and eddy current (ET). Volumetric examinations will be conducted as defined in IWA-2230.

1.7.3.2 Radiography may be performed by utilizing either x-ray or gamma ray techniques.

1.7.3.3 The UT examinations may be performed by utilizing either manual or mechanized UT (Mech UT) techniques in accordance with Appendix I of Section XI and Regulatory Guide 1.150, Rev.1 for the Reactor Vessel Examination only.

1.7.3.4 The ET Method will be utilized in the examination of heat exchanger tubing in accordance with Appendix IV and USNRC Regulatory Guide 1.83.

1.7.4 Alternative Examination Methods

1.7.4.1 Alternative examination methods may be performed to those described in 1.7.1, 1.7.2 and 1.7.3 as allowed in IWA-2240. These may include such things as newly developed techniques, provided that these alternative methods are shown by practical demonstration to be equivalent or superior to those of the specific method to the satisfaction of the Level III NDE Examiner.

1.7.5 Evaluation of Examination Results

1.7.5.1 The evaluation of nondestructive examination results shall be in accordance with Article IWA-3000 of Section XI. All reportable indications will be subject to comparison with previous data to aid in characterization and determination of origin.

Class 1 components containing relevant conditions will be considered acceptable for continued service providing an analytical evaluation performed demonstrates the component's acceptability and is subsequently examined in accordance with the requirements of IWB-3142.4 and IWB-3144(b).

All other components containing relevant conditions will be considered acceptable for continued service providing an evaluation demonstrates the component's acceptability and is subsequently examined in accordance with IWC-3132.3 and IWC-3134(b).

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1.8 Repair Requirements

The repair requirements of this program are applicable to Class 1, 2, 3, high energy piping systems, snubbers and supports as governed by 1.11 and 5.0 of this program.

1.8.1 Performance of Repair

Repairs shall be performed in accordance with the requirements of IWA-4000 and IWF-4000 of ASME Section XI, 1986 Edition or later Edition/Addenda that is approved via 10CFR50.55a. Alternatively, repairs may be performed either to the requirements of the original Construction Code, which the component or system was fabricated to, or to later approved editions of the Construction Code, or later approved editions of ASME Section III along with any Code Cases approved via Regulatory Guide 1.147.

1.8.2 Examination/Test for Repairs

1.8.2.1 Applicable examination requirements of the Construction Code shall be met with the minimum requirement that new welds on pressure retaining components greater than 2 inches in diameter shall receive both surface and 100% volumetric examinations; or surface examination on pressure retaining components less than 2 inches in diameter.

Examinations and testing of snubbers and supports that have been repaired shall be performed in accordance with 1.11 and 5 of this program.

1.8.2.2 The examination shall include the method that detected the flaw, if applicable.

1.8.2.3 Applicable examination requirements of ASME Section XI 1986 Edition shall also be met to serve as a new PSI baseline for future ISI examinations, unless the examination performed under paragraph 1.8.2.1 was conducted under conditions and with equipment and techniques equivalent to those required by Section XI.

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1.8.2.4 A hydrostatic test shall be performed in accordance with ASME Section XI, 1986 Edition, unless specifically exempted by Section XI Article IWA-4400 or Code Case N-416.

1.8.3 Surface flaws in Class 1, 2, or 3, bolts, studs, nuts and ligaments may be removed by mechanical means provided the removal of that flaw does not alter the basic configuration of the item. Bolts, studs, and nuts that have flaws that cannot be removed by mechanical means shall be replaced or reported for evaluation as indicated by Section XI Article IWA-3100.

1.9 Replacement Requirements

Replacement requirements are applicable to Class 1, 2 and 3 and high energy pressure retaining components and piping systems and their supports, unless specifically exempted by ASME Section XI, 1986 Edition, Article IWA-7400.

1.9.1 Replacement Performance

Replacements shall meet the requirements of ASME Section XI, 1986 Edition or a later Edition/Addenda approved via 10CFR50.55a. In addition, replacement items shall meet the requirements of the original Construction Code to which the original part was constructed.

1.9.1.1 A "Like" and "Kind" replacement is an item that meets the original requirements of the design and procurement documents for the item being replaced and does not require reconciliation, re-analysis or changes to the item's design and technical requirements.

1.9.1.2 An approved equivalent is a replacement that will result in a design or technical change to the original requirements based on reconciliation, re-analysis and/or testing per Paragraph 1.9.2.1.

1.9.2 Alternatively items used for replacement may meet all or portions of the requirements later additions of the Construction Code or Section III, when the construction code was not Section III. In order to use this alternate approach, the following additional requirements apply.

1.9.2.1 Reconcile the requirements affecting the design, fabrication and examination of the replacement with the Design Analysis or Design Criteria or other methods of analysis that demonstrates the item is satisfactory for the specified design and operating conditions.

1.9.2.2 Mechanical interfaces, fits and tolerances that provide satisfactory performance are compatible with the system and component requirements.

1.9.2.3 Materials used are compatible with installation and system requirements.

1.9.2.4 If a replacement is because of a failure of the item being replaced, a design evaluation or analysis shall consider the cause of the failure and its impact on other similar items, and the necessary actions to be taken to preclude recurrence.

1.9.2.5 When welding is to be performed as part of the replacement, the rules of Section IX shall be followed to satisfy the requirements of IWA-7320 and IWF-7000.

1.9.2.6 Items identified in IWA 7400 will be exempt from the requirements of the replacement program.

1.9.3 Examination/Test

1.9.3.1 The replacement item shall be examined and pressure tested in accordance with the Construction Code or later Code provided it meets the requirements of 1.9.1 above.

Snubbers shall be examined and tested in accordance to 5.0.

1.9.3.2 Where the attachment of the non-pressure-retaining item is welded to a pressure boundary, the weld shall be examined in accordance with the requirements of 1.8.2.

1.9.3.3 Applicable examination requirements of ASME Section XI, 1986 Edition shall also be met to serve as a new PSI baseline for future ISI examinations, unless the examination performed under 1.9.3.1 was conducted under conditions and with equipment and techniques equivalent to these required by Section XI.

1.9.3.4 Replacements installed by mechanical methods shall be pressure tested at nominal operating pressure, or for Class 1 systems, the pressure associated with 100% rated reactor power.

1.9.4 Reports and Records

Reports and records to the extent required by the construction code and IWA 7520(a) (1 thru 7) as applicable for the replacement shall be completed for all replacements.

1.10 System Pressure Testing

1.10.1 General Requirements

1.10.1.1 Pressure testing shall be conducted on all Class 1, 2, and 3 pressure retaining components in accordance with the requirements of Section XI Articles IWA-5000, IWB-5000, IWC-5000, IWD-5000; and Section XI Table IWB-2500-1 - Examination Category B-P, Table IWC-2500-1 - Examination Category C-H, and Table 2500-1 - Examination Category D-A, D-B and D-C.

In addition, "High Energy" Main Steam and Feedwater Piping shall also be pressure tested on these non-class systems in accordance with the rules of IWC-5000.

1.10.1.2 Pressure tests are conducted at normal operating pressure, or at a pressure 10% to 25% above system pressure. The degree of pressurization and the test boundary depends upon the type of test being

performed. A visual examination (VT-2 method) is performed in concert with the pressure test on pressure retaining components under test pressure. Specific exceptions from achieving Section XI requirements are detailed in the Relief Request Section of this document.

1.10.2 Type of Pressure Tests

The various types of pressure tests which are required during the inspection interval are described in the following:

1.10.2.1. LEAKAGE PRESSURE TEST

This test is limited to Class 1 systems, and is performed subsequent to refueling outages. The boundary subject to test pressurization and the associated VT-2 examination during a leakage pressure test will extend to the pressure retaining components within the system boundary containing pressurized reactor coolant under the plant mode of normal reactor startup.

1.10.2.2 FUNCTIONAL PRESSURE TEST

This test is limited to Class 2 and 3 systems and is performed once each inspection period. The boundary subject to test pressurization and the associated VT-2 examination during a system functional pressure test will include only those pressure retaining components within the system boundary pressurized under the test mode required during the performance of a periodic system/component surveillance test.

1.10.2.3 INSERVICE PRESSURE TEST

This test is limited to Class 2 and 3 systems and is performed once each inspection period. The boundary subject to a test pressurization and the associated VT-2 examination during a system inservice pressure test will include only those pressure retaining components under operating pressure during normal system service.

1.10.2.4 HYDROSTATIC PRESSURE TEST

This test is performed once each inspection interval. The boundary subject to test pressurization and the associated VT-2 examination during a system hydrostatic pressure test includes all Class 1, 2 and 3 components and piping except as exempted by a relief request.

1.10.2.5 PNEUMATIC PRESSURE TEST

This test is limited to Class 2 and 3 systems. The boundary limits subject to test pressurization and the associated VT-2 examination during a system pneumatic pressure test are the same as the associated inservice, functional or hydrostatic pressure test.

1.10.2.6 REPAIR/REPLACEMENT PRESSURE TESTS

The boundary subject to test pressurization and the associated VT-2 examination is limited to the portion repaired or replaced, within the Class 1, 2 or 3 boundary. The specific type of pressure test is either a hydrostatic, pneumatic or a test at operating pressure such as the Leakage, Inservice or Functional. The specific type of test to be performed and the exemptions which apply to repair/replacement pressure testing are described in the Ginna Repair/Replacement program.

1.10.3 Examination Requirements

1.10.3.1 During the conduct of pressure tests, certified VT-2 examination personnel, using RG&E approved NDE VT-2 examination procedure and the associated recording form, will examine the portions of piping under pressurization. The examination and test boundaries are depicted on the color-coded P&ID accompanying the associated test procedures. In some cases, the test boundary extends beyond the examination boundary due to valve location and/or check valve flow direction. In general, personnel will examine for evidence of leakage, inoperative leakage collection systems and evidence of corrosion.

1.10.3.2 Insulation removal during the VT-2 examination is generally not required, however, in accordance with IWA-5242 (a) "systems borated for the purpose of controlling reactivity" shall have insulation removed at bolted connections during conduct of the VT-2 examination. At Ginna, this requirement is considered to be applicable to borated lines only in the primary flow path of piping from the boric acid supply and CVCS Charging to the Reactor Vessel and return through CVCS Letdown, and is not applicable to branch lines connecting to the primary flow path.

Because boric acid is not considered corrosive to austenitic stainless or high alloy materials, insulation removal is only applicable to bolted connections using carbon steel or low alloy bolting materials.

In addition, this requirement is only applicable to those VT-2 examinations performed during a hydrostatic pressure test, since Leakage, Functional and Inservice tests are intended to be non-intrusive type tests.

1.10.4 Test Requirements

1.10.4.1 GENERAL

The contained fluid in the system or fluid added to the system shall serve as the pressurizing or test medium. In steam systems either water or air may be used. Where air is used, the test procedures shall permit the detection and location of through wall leakages in components of the system tested. The temperature of the test medium will be that of the available source unless otherwise specified by the implementing test procedure/document. The test medium will be of a quality which is equal to or better than the system operating medium.

1.10.4.1.1 During conduct of hydrostatic tests all entrained air will be vented from the system except in the following cases:

a. Atmospheric storage tanks
b. 0-15 psi storage tanks

c. Class 1 systems where a mixture of steam, water and noncondensible gases are present in a proportion typical of normal startup conditions.

d. Normal steam systems

1.10.4.1.2 For Leakage, Functional and Inservice tests, the test instrumentation may be permanent plant installed instruments. For hydrostatic tests, any pressure measuring instrument or sensor, analog or digital, including the pressure measuring instrument of the normal operating system instrumentation (such as control room instruments) may be used provided the requirements of Section XI, IWA 5260 and Code Case 437 ("Pressure Measuring Instrumentation used during Class 1, 2, and 3 Pressure Tests"), are met.

1.10.4.2 LEAKAGE PRESSURE TEST REQUIREMENTS

The system leakage pressure test will be conducted at a test pressure which corresponds to the normal operating pressure seen by the system.

The pressure and temperature will be attained at a rate in accordance with the heat-up limitations specified in the Ginna Technical Specifications for the component/piping system being tested.

1.10.4.3 FUNCTIONAL PRESSURE TEST REQUIREMENTS

The system functional pressure test is normally performed during performance of a Periodic Surveillance Test (PT). The test is used to establish the test conditions, associated with normal system operating pressure and temperature, for performance of the system functional pressure test.

1.10.4.4 INSERVICE PRESSURE TEST REQUIREMENTS

The operating pressure and temperature during normal system operation is used during performance of this test.

1.10.4.5 HYDROSTATIC PRESSURE TEST REQUIREMENTS

1.10.4.5.1 General

Test pressure requirements vary among each of the classifications. The minimum test pressure will be maintained for the entire duration of the test, with the exception of tests performed at temperatures greater than 200°F, where the examination phase may be performed at a lower pressure corresponding to 200°F. The test pressure shall not exceed the maximum allowable test pressure of any component within the test boundary.

1.10.4.5.2 The hydrostatic test pressure shall not exceed 106% of test pressure any where within the test boundary, except in cases where the high elevation and low elevation of the system/component cannot be isolated by an isolation valve; and where the pressure at the lowest elevation does not exceed the design pressure of the system/component.

1.10.4.6 The following sections list the test pressure requirements which will be met for each code classification:

1.10.4.6.1 Class 1

The system hydrostatic pressure test is conducted at a test pressure of 1.10 times the system nominal operating pressure (P_o) that is associated with 100% rated reactor power. When system hydrostatic pressure tests are performed at test temperatures (T_t) in excess of 100°F the following table will be used to calculate the reduced test pressure.

Note: Linear interpolation will be used and technical specification heatup/cooldown limits must be observed.

TABLE

Test Temperature	Test Press.(PSIG)
≤199°F	1.10 P_o
200°F - 299°F	1.08 P_o
300°F - 399°F	1.06 P_o
400°F - 499°F	1.04 P_o
500°F	1.02 P_o

1.10.4.6.2 Class 2 and 3

a. The test pressure will be at least 1.10 times the system pressure for systems with a design temperature of 200°F or less, and at least 1.25 times the system pressure for systems with a design temperature above 200°F. The system pressure is equivalent to the lowest pressure setting among the number of safety/relief valves provided for overpressure protection within the boundary of the system to be tested. If safety/relief valves are not installed, the system design pressure will be used as the test pressure.

b. In the case of atmospheric storage tanks, the nominal hydrostatic pressure developed with the tank filled to its design capacity will be acceptable as the test pressure.

c. For 0 to 15 psi storage tanks, the test pressure will be 1.1 times the design pressure of vapor or gas space above liquid level for which overpressure protection is provided by the relief valves. If relief valves are not installed, the test pressure will be equal to 1.1 times the normal operating pressure.

d. For the purpose of the test, open ended portions of a suction or drain line from a storage tank extending to the first shut off valve are considered as an extension of the storage tank.

e. For open ended portions of discharge lines in nonclosed systems (such as containment spray header), a test that demonstrates unimpaired flow will be performed in lieu of a system hydrostatic pressure test.

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1.10.5 Test Implementation

All pressure testing is implemented using both the VT-2 examination procedure and the specific test procedure for the type of test and portion of system being tested.

1.10.6 Scheduling

When using the inspection plan, it should be noted that during a period or refueling outage in which a hydrostatic test is performed on a system or portion(s) of a system, the leakage test (Functional or Inservice) required for that period on the same system or portion of the system, may be deleted from that outage and period.

The hydrostatic test will satisfy the requirements for that leakage test.

1.11 Snubber Inspection and Test Program

1.11.1 General Requirements

1.11.1.1 Inspection and testing of all safety related snubbers shall be conducted to ensure the required operability of these snubbers during a seismic or other event initiating dynamic loads.

1.11.1.2 This program applies only to the snubbers listed in Table 5.

1.11.1.3 This program adheres to the requirements of ASME Section XI, 1986 Edition Article IWF and performed in accordance to the first addenda to ASME/ANSI OM-1987 Part 4 published in 1988.

1.11.1.4 This program complies with the requirements of Technical Specifications Section 3.13 and 4.14. The program is implemented by the requirements of the Mechanical Engineering Specification ME-256 (and will be referred to ME-256 later on in this text), Ginna Station Maintenance Procedures M-40.11 and/or M-40.12 and shall include:

a. Visual Inspection Requirements
b. Visual Inspection Failure Evaluation
c. Visual Inspection Corrective Action and Impact on Examination Frequency.
d. Inspection Documentation
e. Functional Testing Requirements
f. Functional Test Sample
g. Functional Test Failure Analysis
h. Functional Testing Corrective Action
i. Functional Testing Methods
j. Testing Documentation

1.11.2 Visual Inspections

1.11.2.1 The snubbers identified in Table 5 shall be visually inspected based on the results of the previous inspections in accordance with the following schedule:

No. Inoperable Snubbers per Inspection Period	Subsequent Visual* Inspection Period
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
8 or more	31 days ± 25%

The inspection period shall not be lengthened more than one step at a time. Further, mechanical and hydraulic snubbers may be inspected independently in accordance with the above schedule.

1.11.2.2 Visual inspection (VT-3) shall verify (1) that there are no visible indications of impaired operability, (2) visible attachments to the foundation or supporting structure are secure, and (3) in those locations where snubber movement can be manually induced without disconnecting the snubber, that the snubber has freedom of movement and is not frozen up. Snubbers which are defined as inoperable as a result of visual inspections may be determined operable for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the

rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; or (2) the affected snubber is functionally tested in the "as found" condition and determined operable per Paragraph 1.11.3.2. However, when the fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be determined inoperable and cannot be determined operable via functional testing for the purpose of establishing the next visual inspection interval.

1.11.2.3 Snubbers which do not meet the visual inspection (VT-3) requirements of Mechanical Engineering Specification ME-256, shall be reported to Mechanical Engineering (or designee) for visual inspection failure evaluation corrective action and impact on technical specifications Section 3.13. Corrective action may include replacement, modification or repair.

1.11.2.4 Records for the visual inspection (VT-3) shall be maintained on all snubbers listed in the Inservice Inspection Program.

1.11.3 Functional Tests

1.11.3.1 At least once per 18 months during shutdown, at least 10% of snubbers shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria in ME-256, an additional 10 percent of the mechanical and hydraulic snubbers shall be tested until no more failures are found or until "All" snubbers have been functionally tested. If additional sampling is required due to failure of only one type of snubber which has failed the functional testing. The functional testing sample requirements are detailed in the Mechanical Engineering Specification (ME-256) Section 4.8.

1.11.3.2. Functional testing requirements for new installations or spares shall be equal to or more stringent than ME-256 Section 4.7, specifications.

1.11.3.3 Snubbers shall be subjected to a preservice functional test prior to installation using the same test procedures and equipment that is employed during inservice testing or requirements of 1.11.3.2 have been satisfied.

1.11.3.4 Snubber functional test requirement criteria is specified in ME-256 and Ginna Maintenance Procedures M40.11 and/or M40.12 as applicable.

1.11.3.5 Snubbers that do not meet the operability testing acceptance criteria in ME-256 shall be evaluated to determine the cause of the failure. Test failure mode Group(s) shall be established and include all unacceptable snubbers and all other snubbers subject to the same failure mode. This group will remain as defined until corrective action has been completed.

1.11.3.6 Snubbers that have been found unacceptable for the testing acceptance criteria in ME-256 shall be subject to corrective action with its indicated impact on continued testing. Corrective actions for each type of test failure mode group is specified in ME-256 and snubbers will generally be replaced, modified or repaired. For any snubber which fail to meet functional test requirements, an engineering evaluation shall be performed on the components to which the unacceptable snubber is attached. This evaluation shall determine if the components were adversely affected by the unacceptable snubber and to insure that the component remains capable of meeting the designed service requirements.

1.11.3.7 Hydraulic and mechanical snubbers shall each be treated as a different entity for the above surveillance programs.

1.11.3.8 Functional testing examination records shall be maintained on all snubbers listed in the Inservice Inspection Plan.

1.11.4 The visual and functional testing schedule shall be defined in Supplement 1 to this Appendix titled "Snubbers".

1.11.5 Snubber Seal Service Life Monitoring

1.11.5.1 The service life of a snubber is evaluated via manufacturing input and engineering information through consideration of the snubber service conditions and functional design requirements. The only snubber components with service lives not expected to exceed plant life are seal and o-rings fabricated from certain seal materials. Therefore, a seal replacement program is required to monitor snubber seal and o-ring service life to assure snubber operability is not degraded due to exceeding component service life.

1.11.5.2 The seal service life of hydraulic snubbers shall be monitored and seals replaced as required to ensure that the service life is not exceeded between surveillance inspections during a period when the snubber is required to be operable. The seal replacement shall be documented and the documentation shall be retained in accordance with Technical Specification 6.10.2.

1.11.5.3 Records of the service lives of all hydraulic and mechanical snubbers listed in the Inservice Inspection Program including the date at which the service life commences and associated installation and maintenance records will be maintained.

1.12 Steam Generator Tubes and Tube Sleeve Combinations

1.12.1 Repair criteria for steam generator tubes is based on the requirements of USNRC Regulatory Guide 1.121.

1.12.1.1 Steam generator tubes that have imperfections greater than 40 percent through wall as indicated by eddy current, shall be repaired by plugging or sleeving.

1.12.1.2 Steam generator sleeves that have imperfections greater than 30 percent through wall as indicated by eddy current shall be repaired by plugging.

1.12.2 Reporting

1.12.2.1 Within 15 days following the completion of the evaluation of each inservice inspection of steam generator tubes, the number of tubes required by Paragraph 1.12.1 above to the plugged or sleeved in each steam generator shall be reported to the Commission in a Special Report pursuant to Technical Specification 6.9.3.4.

1.12.2.2 The complete results of the steam generator tube inservice inspection shall be submitted to the Nuclear Regulatory Commission in a Special Report pursuant to Technical Specification 6.9.3.4 within 12 months following the completion of the inspection. This Special Report shall include:

(a) Number and event of tubes inspected.

(b) Location and percent of wall-thickness penetration for each indication of an imperfection, and

(c) Identification of tubes plugged or sleeved.

1.12.2.3 If the number of tubes in a generator falling into categories a or b below exceeds the criteria, then results of the inspection shall be considered a Reportable Event pursuant to 10 CFR 50.73. Oral notification to the NRC Staff shall be accomplished within 48 hours, but no sooner than the next normal working day after the final review of the eddy current results. A written followup report shall provide a description of investigations conducted to determine the course of the tube degradation and corrective measures taken to preclude recurrence. Categories (a) and (b) are:

(a) More than 10 percent of the total tubes inspected are degraded (imperfections greater than 20 percent of the nominal wall thickness). However, previously degraded tubes must exhibit at least 10 percent further wall penetration to be included in this calculation.

(b) More than 1 percent of the total tubes inspected are degraded (imperfections greater than the repair limit).

2.0 CLASS 1 PROGRAM PLAN

2.1 Basis for Preparation

2.1.1 Preparation of the Class 1 ISI program plan was based on the requirements of Articles IWB-1000 and IWB-2000 of Section XI. These articles provide rules and guidelines for exemptions, inspection schedules, and examination requirements for Class 1 pressure retaining components and their integral attachments.

2.2 Components Subject to Examination

2.2.1 Based on the requirements of Section XI, the following Class 1 system's nonexempt pressure retaining components and their integral attachments will be subject to examination during the third inspection interval:

2.2.1.1 Vessels
Reactor Pressure Vessel
Pressurizer
Steam Generators (Primary Side)
Regenerative Heat Exchanger

2.2.1.2 Piping
Reactor Coolant System
Pressurizer Surge Line
Residual Heat Removal System
Safety Injection System
Pressurizer Spray System
Pressurizer Safety and Relief System
Charging Lines
Letdown Lines
Drain Lines

2.2.1.3 Pumps
Reactor Coolant Pumps

2.2.1.4 Valves
Safety Injection System
Pressurizer Spray System
Residual Heat Removal System
Pressurizer Safety and Relief System
Charging Lines
Letdown Lines

2.3 Extent and Frequency of Examinations

2.3.1 Class 1 components, as listed in Table 1 shall be examined to the extent and frequency required in Table IWB-2500-1 and Figures IWB 2500 thru IWB 2500-20 of Section XI.

[illegible]

2.4 Exemption Criteria

2.4.1 In accordance with IWB-1220, certain Class 1 components are exempt from examination. The following criteria were applied to exempt components from surface and volumetric examinations in accordance with Section XI:

Exemption Criteria	Code Reference
Piping of 1 inch nominal pipe size (NPS) and smaller, except for steam generator tubing	IWB-1220(b)(1)
Components and their connections in piping of 1 inch NPS and smaller	IWB-1220(b)(2)

2.5 Examination of Reactor Coolant Pump Flywheels
The Reactor Coolant Pump Flywheels shall be examined per the recommendations of Regulatory Guide 1.14, Rev.1 using both surface and volumetric examination methods. These examinations shall be scheduled in the Class 1 section of the ISI program plan.

2.6 Inservice Inspection Program Plan (Supplement 1 to Appendix B)

This plan provides the examination requirements for Class 1 components per the 1986 Edition of Section XI. These requirements shall be satisfied during the third inspection interval. The plan also shows the results of examinations performed in the previous two intervals.

A detailed description of the contents of the Class 1 Examination Plan can be found in the "Introduction", this immediately precedes the tables and the isometric and component drawings in the plan. From this plan an examination schedule is extracted for implementation of the examinations for each outage of the third interval.

3.0 CLASS 2 PROGRAM PLAN

3.1 Basis for Preparation

3.1.1 Preparation of the Class 2 ISI program plan was based on the requirements of Articles IWC-1000 and IWC-2000 of Section XI. These articles provide rules and guidelines for exemptions, inspection schedule, and examination requirements for Class 2 pressure retaining components and their integral attachments.

3.2 Components Subject to Examination

3.2.1 Based on the requirements of Section XI, nonexempt pressure-retaining components and their integral attachments for the following Class 2 systems will be subject to volumetric and/or surface examination during the third inspection interval:

3.2.1.1 Vessels
Steam Generators (Secondary Side)
Non-regenerative Heat Exchanger
Seal Water Return Filter
Seal Water Injection Filter
Residual Heat Exchangers A&B
Pulse Dampener

3.2.1.2 Piping
Safety Injection System
Containment Spray System
Residual Heat Removal System
Main Steam System
Feedwater System

3.3 Extent and Frequency of Examinations

3.3.1 Class 2 components, as listed in Table 2 shall be examined to the extent and frequency required in Table IWC 2500-1 and Figures IWC 2500-1 thru IWC 2500-13 of Section XI.

3.4 Exemption Criteria

IWC-1220 of Section XI provides the exemption criteria for Class 2 components. The following criteria were used to exempt Class 2 components from surface and volumetric examinations in accordance with IWC-1220.

3.4.1.1 The following components (or parts of components) of Residual Heat Removal (RHR), Emergency Core Cooling (ECC), and Containment Heat Removal (CHR), systems (or portions of systems) are exempt from the volumetric and surface examination requirements of IWC-2500:

(a) vessels, piping, pumps, valves and other components 4 inches NPS and smaller in all systems except in high pressure safety injection systems of pressurized water reactor plants;

(b) vessels, piping, pumps, valves, and other components 1-1/2 inches NPS and smaller in high pressure safety injection systems of pressurized water reactor plants;

(c) component connections 4 inches NPS and smaller (including nozzles, socket fittings, and other connections) in vessels, piping, pumps, valves, and other components of any size in all systems except in high pressure safety injection systems of pressurized water reactor plants;

(d) component connections 1-1/2 inches NPS and smaller (including nozzles, socket fittings and other connections) in vessels, piping, pumps, valves and other components of any size in high pressure safety injection systems of pressurized water reactor plants;

(e) vessels, piping, pumps, valves, other components, and component connections of any size in statically pressurized, passive (i.e., no pumps) safety injection systems of pressurized water reactor plants; and

(f) piping and other components of any size beyond the last shutoff valve in open ended portions of systems that do not contain water during normal plant operating conditions.

3.4.1.2 The following components (or parts of components) of systems (or portions of systems) other than RHR, ECC and CHR systems are exempt from the volumetric and surface examination requirements of IWC-2500:

(a) vessels, piping, pumps, valves and other components 4 inches NPS and smaller;

(b) component connections 4 inches NPS and smaller (including nozzles, socket fittings and other connections) in vessels, piping, pumps, valves and other components of any size;

(c) vessels, piping, pumps, valves, other components and component connections of any size in systems or portions of systems that operate (when the system function is required) at a pressure equal to or less than 275 psig and at a temperature equal to or less than 200°F; and

(d) piping and other components of any size beyond the last shutoff valve in open ended portions of systems that do not contain water during normal plant operating conditions.

3.5 Inservice Inspection Program Plan (Supplement 1 to Appendix B)

This plan provides the examination requirements for Class 2 IWC components per the 1986 Edition of Section XI and R.E. Ginna Technical Specifications. These requirements shall be satisfied during the third inspection interval. The plan also shows the results of examinations performed in the previous two intervals.

A detailed description of the contents of the Class 2 Examination Plan can be found in the "Introduction". This immediately precedes the tables and the isometric and components drawings in the plan. From the plan an examination schedule is extracted for implementation of the examinations for each outage of the third interval.

4.0 CLASS 3 PROGRAM

4.1 Basis for Preparation

4.1.1 Preparation of the Class 3 ISI program was based on the requirements of Articles IWD-1000 and IWD-2000 of Section XI. These articles provide rules and guidelines for exemptions, inspection schedules, and examination requirements for Class 3 pressure retaining components and their integral attachments.

4.2 Components Subject to Examination

4.2.1 Based on the requirements of Section XI, nonexempt pressure-retaining components and their integral attachments for the following Class 3 systems will be subject to examination during the third inspection interval:

4.2.1.1 Systems

a. Auxiliary Feedwater System
b. Residual Heat Removal System
c. Component Cooling System
d. Service Water Cooling System
e. Spent Fuel Pool Cooling

4.2.1.2 "Other Class 3 systems are not subject to the examination requirements of ASME Section XI because they do not meet the system function requirements of Examination Categories D-A, D-B and D-C, where:

D-A Systems in support of Reactor Shutdown Function

D-B Systems in support of Emergency Core Cooling, Containment Heat Removal, Atmosphere Clean-up and Reactor Residual Heat Removal.

D-C Systems in support of Residual Heat Removal from Spent Fuel Storage Pool".

4.3 Extent and Frequency of Examinations

4.3.1 Class 3 components, as listed in Table 3 shall be examined to the extent and frequency required in Table IWD 2500-1 and Figure IWD 2500-1 of Section XI.

4.4 Exemption Criteria

4.4.1 In accordance with IWD-1220, certain Class 3 components are exempt from examination. The following exemption criteria was applied to Class 3 systems as specified in IWD-1220:

Exemption Criteria	Section XI Reference
Integral attachments of supports and restraints to components that are 4 inches NPS and smaller within the system boundaries of Examination Categories D-A, D-B, and D-C shall be exempt from the VT-3 examination, except for Auxiliary Feedwater.	IWD-1220.1
Integral attachments of supports and restraints to components exceeding 4" nominal pipe size may be exempted from the visual examination VT-3 of Table IWD-2500-1 provided:	IWD-1220.2

(a) the components are located in systems (or portions of systems) whose function is not required in support of reactor residual heat removal, containment heat removal, and emergency core cooling; and

(b) the components operate at a pressure of 275 psig or less and at a temperature of 200 degrees F or less.

4.5 Inservice Inspection Program Plan (Supplement 1 to Appendix B)

This plan provides the examination requirements for Class 3 IWD components per the 1986 Edition of Section XI. These requirements shall be satisfied during the third inspection interval. The plan also shows the results of examinations performed in the previous two intervals.

A detailed description of the contents of the Class 3 Examination Plan can be found in the "Introduction". This immediately precedes the tables and the isometric and component drawings in the plan. From the plan an examination schedule is extracted for implementation of the examinations for each outage of the third interval.

5.0 CLASS 1, CLASS 2, AND CLASS 3 COMPONENT SUPPORTS

5.1 Basis for Preparation

5.1.1 Preparation of the component support ISI program was based on the requirements of Articles IWF-1000 and IWF-2000 of Section XI. These articles provide rules and guidelines for exemptions, inspection schedules, and examination requirements for Class 1, Class 2, and Class 3 component supports. Inservice test requirements and VT-3 inspection requirements for snubbers shall be conducted in accordance with this program and Technical Specifications and Mechanical Specifications which implements the requirements of Article IWF-5000.

5.2 Components Subject to Examination

5.2.1 Based on the requirements of Section XI, nonexempt component supports for the Class 1, Class 2 and Class 3 systems identified in Sections 2.2, 3.2 and 4.2 of this plan will be subject to examination during the third inspection interval. The component supports requiring examination shall be as follows:

5.2.1.1 Plate and Shell - Type Supports

Supports which are fabricated from plate and shell elements, such as vessel skirts and saddles, and are normally subjected to a biaxial stress.

5.2.1.2 Linear-Type Supports

Supports acting under essentially a single component of direct stress. Such elements may also be subjected to shear stress. Examples of such structural elements are: tension and compression struts; beams and columns subjected to bending; trusses; frames; arches; rings; and cables.

5.2.1.3 Component Standard Supports

A support assembly consisting of one or more generally mass-produced units usually referred to as catalog items. Examples of such items are shown in Figure IWF-1210-1 of Section XI.

5.3 Extent and Frequency of Examination

5.3.1 Component supports selected for examination shall be those components required to be examined under the requirements of 2.3, 3.3 and 4.3, IWF-2500. The inservice test requirements of Article IWF-5000 shall be satisfied by the requirements of 1.11.

5.3.2 In addition, snubbers identified in Table 5 will be functionally tested at the frequency required in 1.11.3.

5.3.3 On piping systems where a piping seismic analysis boundary is beyond the safety class boundary, the component supports within the portion between those two boundaries shall be examined. The extent of these examinations shall be consistent with the examination requirements of that safety class system.

5.3.4 High Energy Piping Component Supports shall be examined to IWC-2500 Category C-C and to IWF-2500 in an Augmented Program.

5.4 Exemptions

5.4.1 ASME Section XI, 1986 Edition, does not contain defined exemption criteria for component supports.

5.4.2 Exemption criteria specified in IWB, IWC and IWD have been used.

5.5 Inservice Inspection Program Plan (Supplement 1 to Appendix B)

This plan provides the Class 1, Class 2 and Class 3 component supports shall be incorporated into the appropriate Class 1, Class 2 or Class 3 section of the examination plan. These requirements shall be satisfied during the third inspection interval. The plan also shows the results of examinations performed in the previous two intervals.

6.0 AUGMENTED ISI PROGRAM FOR HIGH ENERGY PIPING

6.1 Basis For Examination

6.1.1 The inservice inspection program for high-energy piping outside of containment, as established in Rochester Gas and Electric Corporation's report "Effect of Postulated Pipe Breaks Outside the Containment Building," dated October 29, 1973, provides for the examination of all circumferential butt welds at design break locations and at discontinuity locations where a failure would result in unacceptable consequences. This inspection program consists of volumetric examinations utilizing the radiographic or ultrasonic methods and magnetic particle and visual examinations of these welds; these types of examinations are used to detect any change in condition or development of service induced flaws in advance of a potential failure. Surveillance of these welds by the inspection program provides assurance that the design basis or consequential Main Steam or Feedwater breaks will not occur.

6.2 Inservice Inspection Program Plan

6.2.1 The 10-year inspection plan consists of three periods of 3, 4 and 3 years each, respectively. During each period of the 10-year interval, the inspection plan provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences.

6.3 Welds Subject to Examination

6.3.1 Table 6 of this plan lists all the welds by category and system. This table defines the inspection plan for the design basis break location welds, and the consequential break location welds.

6.4 Evaluation

6.4.1 The evaluation of the examinations of the high-energy piping welds outside of containment will be in accordance with Section XI IWC 3130 for visual examinations and IWC 3120 for magnetic particle examinations. Evaluations of radiographic results in accordance with the acceptance criteria for radiographic examinations referenced in USAS B31.1.0-1967, "Power Piping" and "ASME Section XI." Ultrasonic Examinations performed in accordance with the requirements of Appendix I and III and the acceptance criteria in USAS B31.1.0 - 1967 (Radiography).

7.0 RELIEF REQUESTS

7.1 Relief Requests

In accordance with 10CFR50.55a(g)(5)(iv), RG&E has requested relief from those ASME Section XI requirements that have been determined impractical for certain areas. This section summarizes each Relief Request submitted to the NRC for consideration. Each request provides information on the component for which relief is requested, ASME requirements, proposed alternate method, and other pertinent information, as needed. Additional Relief Requests will be submitted to the NRC as appropriate.

Table 7 presents a summary of the enclosed Relief Requests followed by the Relief Requests.

Table 1

SCHEME
CODE CLASS

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
The ASME Section XI Item No. and Category of the component are listed in these columns.		Each type of examination area is listed in this column.	The NDE method required to satisfy Code requirements is listed in this column.	This column provides information regarding the number and/or percent of examinations required to be performed for the inspection interval.	This column provides information specific to examination techniques and examination areas.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
REACTOR PRESSURE VESSEL					
B1.10 B1.11 B1.12	B-A	Circumferential and Longitudinal Shell Welds	Volumetric	100% of one circumferential weld to be examined at structural discontinuity in the beltline region. Examinations may be performed at or near the end of the inspection interval.	Examination of circumferential shell welds will be performed with UT techniques. There are no longitudinal welds at R. E. Ginna.
B1.20 B1.21 B1.22	B-A	Circumferential and Meridional Head Welds	Volumetric	The accessible length of one circumferential weld of each head to be examined. Examinations may be performed at or near the end of each inspection interval for the lower head.	Lower head and closure head welds will be examined utilizing UT techniques. There are no meridional welds at R. E. Ginna.
B1.30	B-A	Shell-to-Flange Weld	Volumetric	100% of the weld to be examined. At least 50% of the weld shall be examined from the flange face by the end of the first inspection period and the remainder by the end of the third inspection period.	The shell-to-flange weld will be examined from the vessel seal surface and from the vessel wall inside surface with UT.
B1.40	B-A	Head-to-Flange Weld	Volumetric and Surface	100% of the weld to be examined.	The head-to-flange weld will be examined with UT and surface examination techniques when the head is removed.
B1.50 B1.51	B-A	Repair Welds	N/A		No repair welds at R. E. Ginna.
B3.90 B3.100	B-D B-D	Nozzle-to-Vessel Welds and Nozzle Inside Radius Section	Volumetric	100% of nozzles. At least 25% but not more than 50% of the nozzles shall be examined by the end of the first period and the remainder by the end of the interval.	The nozzle-to-vessel welds and nozzle inside radius sections will be examined with UT.
B4.10 B4.11 B4.12 B4.13	B-E	Partial Penetration Welds: Vessel Nozzles Control Rod Drive Nozzles Instrumentation Nozzles	Visual (VT-2)	At least 25% of each group of welds of comparable size and function to be examined.	Examination will be performed during the system hydrostatic test. VT examinations will be performed in accordance with IWA-5240.

42-52D

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
B5.10 B5.20 B5.30	B-F	Pressure-Retaining Dissimilar Metal Welds	N/A		No dissimilar metal welds on RPV at R. E. Ginna.
B6.10	B-G-1	Closure Head Nuts >2 Inches in Diameter	Surface	100% of nuts to be examined. Examination may be performed at or near the end of the inspection interval.	Nuts will be examined with MT when removed for refueling.
B6.20	B-G-1	Closure Head Studs, in Place, >2 Inches in Diameter	Volumetric	100% of studs to be examined. Examination may be performed at or near the end of the inspection interval.	Closure stud examinations may be performed "in place." Examinations should be scheduled when studs are removed to reduce radiation exposure and allow the most thorough examination. Code Case N-307-1 shall apply.
B6.30	B-G-1	Closure Head Studs, When Removed, >2 Inches in Diameter	Volumetric and Surface	100% of studs to be examined. Examination may be performed at or near the end of the inspection interval.	The studs will be examined with UT and MT. Code Case N-307-1 shall apply.
B6.40	B-G-1	Threads in Flange >2 Inches in Diameter	Volumetric	100% of threaded holes to be examined. Examination may be performed at or near the end of the inspection interval.	The threads in flange will be examined from the flange face with UT.
B6.50	B-G-1	Pressure-Retaining Closure Washers and Bushings >2 Inches in Diameter	Visual (VT-1)	All washers and bushings to be examined upon stud removal. Examination may be performed at or near the end of the inspection interval.	
B7.10	B-G-2	Pressure-Retaining Bolting ≤2 Inches in Diameter	Visual (VT-1)	All bolts, studs, and nuts to be examined.	The bolting may be examined in place under tension or when disassembled or removed.
B7.80	B-G-2	CRD Housings Bolts, Studs, and Nuts	N/A		No pressure-retaining CRD housing bolting at R. E. Ginna.
B8.10	B-H	Integral Attachments for RPV	N/A		No integrally welded attachments on RPV at R. E. Ginna that meet the requirements of Category B-H.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
B13.10	B-N-1	Vessel Interior	Visual (VT-3)	Accessible areas to be examined during each inspection period.	
B13.50	B-N-2	Interior Attachments Within Beltline Region	Visual (VT-1)	Accessible attachment welds to be examined. Examinations may be completed at or near the end of the inspection interval.	
B13.60	B-N-2	Interior Attachments Beyond Beltline Region	Visual (VT-3)	Accessible attachment welds to be examined. Examinations may be completed at or near the end of the inspection interval.	
B13.70	B-N-3	Core-Support Structure	Visual (VT-3)	With core-support structure removed, all accessible surfaces to be examined. Examinations may be completed at or near the end of the inspection interval.	
B14.10	B-O	Control Rod Drive Housing Welds	Volumetric or Surface	Welds in 10% of the peripheral CRD housings to be examined. Examinations may be performed at or near the end of the inspection interval.	The CRD housing welds will be examined with PT.
B15.10	B-P	All Pressure-Retaining Boundaries for Vessel Components	Visual (VT-2)	All components to be examined during system leakage test. Examinations to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examinations will be performed in accordance with IWA-5240.
B15.11	B-P	All Pressure-Retaining Boundaries for Vessel Components	Visual (VT-2)	All components to be examined during system hydrostatic test. Examinations to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examinations will be performed in accordance with IWA-5240.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PRESSURIZER					
B2.10 B2.11 B2.12	B-B	Circumferential and Longitudinal Shell-to-Head Welds	Volumetric	The upper and lower head-to-shell welds to be examined. One foot of one longitudinal weld intersecting each head-to-shell weld to be examined.	Examination will be performed with UT techniques.
B2.20 B2.21 B2.22	B-B	Circumferential and Meridional Head Welds	Volumetric	There are no pressurizer head welds at R. E. Ginna.	
B3.110 B3.120	B-D	Nozzle-to-Vessel Welds and Nozzle Inside Radius Sections	Volumetric	All nozzle-to-vessel welds and inside radius sections to be examined. At least 25% but not more than 50% of the nozzles shall be examined by the end of the first period and the remainder by the end of the inspection interval.	Examinations will be performed with UT techniques.
B4.20	B-E	Heater Penetration Welds	Visual (VT-2)	All pressurizer heater penetration welds shall be examined during system hydrostatic test in accordance with IWB-5222 at or near the end of the inspection interval.	VT examination will be performed in accordance with IWA-5240.
B5.40	B-F	Nozzle-to-Safe End Dissimilar Metal Welds ≥4 Inches NPS	Surface and Volumetric	All butt welds to be examined.	Examinations will be performed with PT and UT techniques.
B5.50	B-F	Nozzle-to-Safe End Dissimilar Metal Welds <4 Inches NPS	Surface	All butt welds to be examined.	Examinations will be performed with PT techniques.
B5.60	B-F	Nozzle-to-Safe End Socket Welds	N/A		No dissimilar metal socket welds on pressurizer at R. E. Ginna.
B6.60 B6.70 B6.80	B-G-1	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No bolting on pressurizer >2 inches in diameter at R. E. Ginna.

42-52B

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PRESSURIZER (Cont'd)					
B7.20	B-G-2	Bolts, Studs, and Nuts ≤2 Inches in Diameter	Visual (VT-1)	All bolts, studs, and nuts to be examined.	Bolting examinations may be performed in place under tension or when disassembled or removed.
B8.20	B-H	Integrally Welded Attachments	Volumetric or Surface	Pressurizer support skirt to be examined.	Examination will be performed per Figure IWB-2500-14.
B15.20	B-P	Pressure-Retaining Boundaries for Vessel Components	Visual (VT-2)	All pressurizer components to be examined during system leakage test. Examination to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examination will be performed in accordance with IWB-5240.
B15.21	B-P	Pressure-Retaining Boundaries for Vessel Components	Visual (VT-2)	All pressurizer components to be examined during system hydrostatic test. Examination to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examination will be performed in accordance with IWA-5240.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
STEAM GENERATORS					
B2.30 B2.31 B2.32	B-B	Circumferential and Meridional Welds in the Lower Head	Volumetric		There are no steam generator lower head circumferential or meridional welds.
B2.40	B-B	Tubesheet-to-Head Weld	Volumetric	The tubesheet-to-head weld will be examined. The examination will be limited to one of the two steam generators.	Examination will be performed with UT techniques.
B3.130 B3.140	B-D	Nozzle-to-Vessel Welds and Nozzle Inside Radius Sections	Volumetric	All nozzle inside radius sections will be examined. At least 25% but not more than 50% of the nozzles shall be examined by the end of the first inspection period and the remainder by the end of the inspection interval.	Examinations will be performed with UT techniques. There are no steam generator nozzle-to-vessel welds.
B5.70 B5.80 B5.90	B-F	Nozzle-to-Safe End Dissimilar Metal Welds	Surface and Volumetric	All nozzle safe-end dissimilar metal welds shall be examined.	Examinations will be performed with PT and UT techniques at R. E. Ginna.
B6.90 B6.100 B6.110	B-G-1	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No bolting on steam generators >2 inches in diameter at R. E. Ginna.
B7.30	B-G-2	Bolts, Studs, and Nuts ≤2 Inches in Diameter	Visual (VT-1)	All bolts, studs, and nuts will be examined.	Examinations may be performed in place under tension or when removed.
B8.30	B-H	Integrally Welded Attachments	N/A		There are no steam generator integrally welded attachments. IWB-2500-13.
B15.30	B-P	Pressure-Retaining Boundaries	Visual (VT-2)	All steam generator components to be examined during system leakage test. Examinations to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examinations will be performed in accordance with IWA-5240.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
STEAM GENERATORS (Cont'd)					
B15.31	B-P	Pressure-Retaining Boundaries	Visual (VT-2)	All steam generator components to be examined during system hydrostatic test. Examinations to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examinations will be performed in accordance with IWA-5240.
B16.20	B-Q	Steam Generator Tubing	Volumetric	The tubing in the hot leg side, U-bend portion, and optionally cold leg side will be examined.	Examination requirements, examination method, and the extent and frequency of examination shall be in accordance with plant Technical Specifications.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
HEAT EXCHANGERS					
B2.50 B2.51 B2.52	B-B	Head Welds	Volumetric	One weld per head will be examined.	Examination will be performed with UT techniques.
B2.60	B-B	Tubesheet-to-Head Welds	Volumetric	Each tubesheet-to-head weld will be examined.	Examination will be performed with UT techniques.
B2.70	B-B	Longitudinal Welds	N/A		There are no RHE longitudinal welds at R. E. Ginna.
B2.80	B-B	Tubesheet-to-Shell Welds	Volumetric	Each tubesheet-to-shell weld will be examined.	Examination will be performed with UT techniques.
B3.150 B3.160	B-D B-D	Nozzle-to-Vessel Welds and Nozzle Inside Radius Section	Volumetric	All nozzle-to-shell and nozzle inside radius sections will be examined. At least 25% but not more than 50% of the nozzles shall be examined by the end of the first inspection period and the remainder by the end of the interval.	Examination will be performed with UT techniques.
B5.100 B5.110 B5.120	B-F	Pressure-Retaining Dissimilar Metal Welds	N/A	No heat exchanger dissimilar welds at R. E. Ginna.	
B6.120 B6.130 B6.140	B-G-1	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No heat exchanger bolting at R. E. Ginna.
B7.40	B-G-2	Bolts, Studs, and Nuts ≤2 Inches in Diameter	N/A		No heat exchanger bolting at R. E. Ginna.
B8.40	B-H	Integrally Welded Attachments	Volumetric or Surface	Integrally welded attachments in one RHE support shall be examined.	Examination will be performed with PT or UT techniques.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
HEAT EXCHANGERS (Cont'd)					
B15.40	B-P	Pressure-Retaining Boundaries	Visual (VT-2)	All RHE components will be examined during system leakage test. Examinations to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examinations will be performed in accordance with IWA-5240.
B15.41	B-P	Pressure-Retaining Boundaries	Visual (VT-2)	All RHE components will be examined during system hydrostatic test. Examinations to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examinations will be performed in accordance with IWA-5240.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PIPING					
B5.130	B-F	Dissimilar Metal Welds ≥4 Inches NPS	Volumetric and Surface	All butt welds to be examined.	The welds will be examined with UT and PT.
B5.140	B-F	Dissimilar Metal Welds <4 Inches NPS	Surface	All butt welds to be examined.	The welds will be examined with PT.
B5.150	B-F	Dissimilar Metal Socket Welds	N/A		No dissimilar metal socket welds at R. E. Ginna.
B6.150 B6.160 B6.170	B-G-1	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No pressure-retaining bolting >2 inches in diameter at R. E. Ginna.
B7.50	B-G-2	Pressure-Retaining Bolting ≤2 Inches in Diameter	Visual (VT-1)	All bolts, studs, and nuts to be examined.	The bolting may be examined in place under tension or when disassembled or removed.
B9.10 B9.11	B-J	Circumferential Pipe Welds ≥4 Inches NPS	Volumetric and Surface	25% of the required circumferential butt welds to be examined. See Note 1 at end of Table 1 for selection criteria.	The piping welds will be examined with UT and PT.
B9.12	B-J	Longitudinal Pipe Welds ≥4 Inches NPS	Volumetric and Surface	Longitudinal welds that adjoin scheduled circumferential welds are to be examined. One pipe diameter not to exceed 12 inches of each longitudinal weld length required.	The piping welds will be examined with UT and PT.
B9.20 B9.21	B-J	Circumferential Welds <4 Inches NPS	Surface	25% of the required circumferential butt welds to be examined. See Note 1 at end of Table 1 for selection criteria.	The piping welds will be examined with PT.
B9.22	B-J	Longitudinal Pipe Welds <4 Inches NPS	Surface	Longitudinal welds that adjoin scheduled circumferential welds are to be examined. One pipe diameter of each longitudinal weld length required.	The piping welds will be examined with PT.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PIPING (Cont'd)					
B9.30 B9.31	B-J	Branch Pipe Connection Welds ≥4 Inches NPS	Volumetric and Surface	25% of the required branch connection joints to be examined. See Note 1 at end of Table 1 for selection criteria.	The branch connection welds will be examined with UT and PT.
B9.32	B-J	Branch Pipe Connection Welds <4 Inches NPS	Surface	25% of the required branch connection joints to be examined. See Note 1 at end of Table 1 for selection criteria.	The branch connection welds will be examined with PT.
B9.40	B-J	Socket Welds	Surface	25% of the required socket welds to be examined. See Note 1 at end of Table 1 for selection criteria.	The socket welds will be examined with PT.
B10.10	B-K-1	Integrally Welded Attachments	Volumetric or Surface	All required supports to be examined. See Note 2 at end of Table 1 for selection criteria.	The integrally welded attachments will be examined with PT or UT as applicable.
B15.50	B-P	All Pressure-Retaining Boundaries for Piping Components	Visual (VT-2)	All components to be examined during system leakage test. Examination to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examination will be performed in accordance with IWA-5240.
B15.51	B-P	All Pressure-Retaining Boundaries for Piping Components	Visual (VT-2)	All components to be examined during system hydrostatic test. Examination to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examination will be performed in accordance with IWA-5240.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PUMPS					
B6.180	B-G-1	Bolts and Studs, >2 Inches in Diameter	Volumetric	All bolts and studs to be examined.	The bolting will be examined with UT. Code Case N-307-1 shall apply. The bolting may be examined in place under tension or when disassembled or removed.
B6.190	B-G-1	Flange Surface for Bolting >2 Inches in Diameter When Connection Is Disassembled	Visual (VT-1)	Examination includes 1 inch annular surface of flange around each stud hole surface when disassembled.	
B6.200	B-G-1	Nuts, Bushings, and Washers >2 Inches in Diameter	Visual (VT-1)	All nuts, bushings, and washers to be examined when disassembled.	
B7.60	B-G-2	Pressure-Retaining Bolting ≤2 Inches in Diameter	Visual (VT-1)	All bolts, studs, and nuts to be examined.	The bolting may be examined in place under tension or when disassembled or removed.
B10.20	B-K-1	Integrally Welded Attachments	Volumetric or Surface	All required attachments to be examined. See Note 2 at end of Table 1 for selection criteria.	Welded attachments will be examined with PT or UT as applicable.
B12.10	B-L-1	Pump Casing Welds	Volumetric	Reactor coolant pump casing welds on one pump to be examined.	Item exempt from examination per Relief #4..
B12.20	B-L-2	Internal Surfaces of Pump Casings	Visual (VT-3)	One reactor coolant pump to be examined when disassembled.	Pump casing internal surface will be examined with visual techniques when disassembled.
B15.60	B-P	Pressure-Retaining Boundaries for Pump Components	Visual (VT-2)	All components to be examined during system leakage test. Examination to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examinations will be performed in accordance with IWA-5240.

42-52B

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PUMPS (Cont'd)					
B15.61	B-P	Pressure-Retaining Boundaries for Pump Components	Visual (VT-2)	All components to be examined during system hydrostatic test. Examination to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examinations will be performed in accordance with IWA-5240.
N/A	N/A	Pump Flywheels	Surface and Volumetric	Each reactor coolant pump flywheel to be examined per Reg. Guide 1.14. High-stress areas to be examined each period with full examination at the end of interval.	Examinations will be performed with MT and UT techniques.

Table 1

INSERVICE INSPECTION PROGRAM
CLASS 1 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To be Examined	Examination Methods	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
VALVES					
B6.210 B6.220 B6.230	B-G-1	Bolting >2 Inches in Diameter	N/A		No valve bolting >2 inches in diameter at R. E. Ginna.
B7.70	B-G-2	Bolting ≤2 Inches in Diameter	Visual (VT-1)	All bolts, studs, and nuts to be examined.	The bolting may be examined in place under tension or when disassembled or removed.
B10.30	B-K-1	Integrally Welded Attachments	Volumetric or Surface	All required attachments to be examined. See Note 2 at end of Table 1 for selection criteria.	Welded attachments will be examined with PT or UT as applicable.
B12.30	B-M-1	Pressure-Retaining Welds in Valve Bodies <4 Inches NPS	Surface	One valve in each group of valves that is of the same construction and similar function to be examined.	Valve welds will be examined with PT.
B12.40	B-M-1	Pressure-Retaining Welds in Valve Bodies ≥4 Inches NPS	N/A		No pressure-retaining welds in valve bodies ≥4 inches NPS at R. E. Ginna.
B12.50	B-M-2	Internal Surfaces of Valve Bodies on Valves >4 Inches NPS	Visual (VT-3)	One valve in each group of valves that is of the same construction and similar function to be examined.	
B15.70	B-P	All Pressure-Retaining Boundaries for Valve Components	Visual (VT-2)	All components to be examined during system leakage test. Examination to be performed in accordance with IWB-5221 prior to plant startup after each refueling outage.	VT examinations will be performed in accordance with IWB-5240.
B15.71	B-P	All Pressure-Retaining Boundaries for Valve Components	Visual (VT-2)	All components to be examined during system hydrostatic test. Examination to be performed in accordance with IWB-5222 at or near the end of the inspection interval.	VT examinations will be performed in accordance with IWA-5240.

42-52D

Table 1 - NOTES

(1) Examinations shall be distributed throughout the piping system to satisfy the extent required by ASME Section XI 1974 Edition with Addenda through Summer 1975 as permitted by 10CFR50.55a(b)(2)(ii). Examinations selected shall be distributed in such a manner to include:

(a) Terminal ends in each pipe or branch run connected to vessels.

(b) Terminal ends and joints in each pipe or branch run connected to other components.

(c) All dissimilar metal welds.

(d) Additional piping welds so that the total number of circumferential butt welds (or branch connection or socket welds) selected for examination equals 25% of the circumferential butt welds (or branch connection or socket welds) in the reactor coolant piping system. This total does not include welds excluded by IWB-1220. These additional welds may be located in one loop.

(2) Examination is limited to those integrally welded attachments that meet the following conditions:

(a) the attachment is on the outside surface of the pressure-retaining component;

(b) the attachment provides component support as defined in NF-1110;

(c) the attachment base material design thickness is 5/8 inch or greater; and

(d) the attachment weld joins the attachment either directly to the surface of the component or to an integrally cast or forged attachment to the component.

Examinations include the welded attachments of piping required to be examined by Examination Category B-J and the welded attachments to associated pumps and valves integral to such piping.

Table 2

SCHEME
CODE CLASS

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
The ASME Section XI Item No. and Category of the component are listed in these columns.		Each type of examination area is listed in this column.	The NDE method required to satisfy Code requirements is listed in this column.	This column provides information regarding the number and/or percent of examinations required to be performed for the inspection interval.	This column provides information specific to examination techniques and examination areas.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS

Item No.	ExaminationCategory	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PRESSURE VESSELS					
C1.10	C-A	Shell Circumferential Welds	Volumetric	100% of each weld to be examined (applies only to welds at gross structural discontinuities). For multiple vessels of similar design size and service, examinations may be limited to one vessel.	The welds will be examined with UT.
C1.20	C-A	Head Circumferential Welds	Volumetric	100% of each head-to-shell weld to be examined. For multiple vessels of similar design size and service, examinations may be limited to one vessel.	The welds will be examined with UT.
C1.30	C-A	Tubesheet-to-Shell Weld	Volumetric	100% of each weld to be examined. For multiple vessels of similar design size and service, examinations may be limited to one vessel.	The welds will be examined with UT.
C2.10 C2.11	C-B	Nozzles in Vessels ≤1/2-Inch Nominal Thickness Nozzle-to-Shell (or Head) Weld	Surface	All nozzles at terminal ends of piping runs which are selected for examination under Categories C-F-1 and C-F-2. 100% of each weld to be examined. Manways and hand holes excluded. For multiple vessels of similar design, size, and service, examinations may be limited to one vessel.	The welds will be examined with MT or PT as applicable.
C2.20 C2.21	C-B	Nozzles Without Reinforcing Plate in Vessels >1/2-Inch Nominal Thickness Nozzle-to-Shell (or Head) Weld	Surface and Volumetric	All nozzles to be selected at terminal ends of piping runs selected for examination under Categories C-F-1 and C-F-2. 100% of each weld to be examined. Manways and hand holes excluded. For multiple vessels of similar design, size, and service, examinations may be limited to one vessel.	The welds will be examined with UT and MT or PT as applicable.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	ExaminationCategory	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PRESSURE VESSELS (Cont'd)					
C2.22	C-B	Nozzle Inside Radius Section	Volumetric	100% of each area to be examined. Manways and hand holes are excluded. For multiple vessels of similar design, size, and service, examinations may be limited to one vessel.	The nozzle inside radius section will be examined with UT.
C2.30 C2.31 C2.32 C2.33	C-B	Nozzles with Reinforcing Plate in Vessels >1/2-Inch Nominal Thickness	N/A		There are no nozzles with reinforcing plate in vessels >1/2-inch nominal thickness at R. E. Ginna.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	Examinatio nCategory	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PRESSURE VESSELS (Cont'd)					
C3.10	C-C	Integrally Welded Attachments	Surface	100% of each weld to be examined. Attachments whose base material is 3/4 inch or greater to be selected. Where multiple vessels are provided with a number of similar attachments, the attachments may be distributed among the vessels. For multiple vessels of similar design and service, examinations may be limited to one vessel.	The welded attachments will be examined with MT or PT as applicable.
C4.10	C-D	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No pressure-retaining vessel bolting >2 inches in diameter at R. E. Ginna.
C7.10	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for vessels to be examined during system pressure test. Examinations to be performed in accordance with IWC-5221 for each inspection period.	VT examinations will be performed in accordance with IWA-5240.
C7.20	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for vessels to be examined during system hydrostatic test. Examinations to be performed in accordance with IWC-5222 at or near the end of each inspection interval or during same inspection periods of each interval.	VT examinations will be performed in accordance with IWA-5240.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PIPING					
C3.20	C-C	Integrally Welded Attachments	Surface	100% of each weld to be examined. Attachments whose base material is 3/4 inch or greater to be selected. Selection limited to those components selected under Examination Categories C-F-1 and C-F-2.	The welds will be examined with MT or PT as applicable.
C4.20	C-D	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No pressure-retaining bolting >2 inches in diameter at R. E. Ginna.
C5.10 C5.11 C5.12	C-F-1	Piping Welds in Austenitic Stainless Steel or High-Alloy Piping ≥3/8-Inch Nominal Wall Thickness for Piping >4 NPS, Circumferential and Longitudinal	Surface and Volumetric	100% of each circumferential and 2.5t of each longitudinal weld requiring examination. See Note 1 at end of Table 2 for selection criteria.	The welds will be examined with UT and PT.
C5.20 C5.21 C5.22	C-F-1	Piping Welds in Austenitic Stainless Steel or High-Alloy Piping ≥1/5-Inch Nominal Wall Thickness for Piping ≥2 NPS and ≤4 NPS, Circumferential and Longitudinal	Surface and Volumetric	100% of each circumferential and 2.5t of each longitudinal weld requiring examination. See Note 1 at end of Table 2 for selection criteria.	The welds will be examined with UT and PT.
C5.30	C-F-1	Socket Welds in Austenitic Stainless Steel or High-Alloy Piping	Surface	100% of each weld requiring examination. See Note 1 at end of Table 2 for selection criteria.	The welds will be examined with PT.
C5.40 C5.41 C5.42	C-F-1	Pipe Branch Connections in Austenitic Stainless Steel or High-Alloy Piping ≥2 NPS, Circumferential and Longitudinal	Surface	100% of each circumferential and 2.5t of each longitudinal weld requiring examination. See Note 1 at end of Table 2 for selection criteria.	The welds will be examined with PT.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PIPING (Cont'd)					
C5.50 C5.51 C5.52	C-F-2	Piping Welds in Carbon or Low-Alloy Steel ≥3/8-Inch Nominal Wall Thickness for Piping >4 NPS, Circumferential and Longitudinal	Surface and Volumetric	100% of each circumferential and 2.5t of each longitudinal weld requiring examination. See Note 2 at end of Table 2 for selection criteria. In addition, 100% of the main steam and main feedwater welds located outside containment and traversing safety areas shall be examined per Technical Specifications.	The welds will be examined with UT and MT.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PIPING (Cont'd)					
C5.60 C5.61 C5.62	C-F-2	Piping Welds in Carbon or Low-Alloy Steel >1/5-inch Nominal Wall Thickness for Piping ≥2 NPS and ≤4 NPS, Circumferential and Longitudinal	N/A		No carbon or low-alloy steel nonexempt welds in this category at R. E. Ginna.
C5.70	C-F-2	Socket Welds in Carbon or Low-Alloy Steel	N/A		No carbon or low-alloy steel nonexempt welds in this category at R. E. Ginna.
C5.80 C5.81 C5.82	C-F-2	Pipe Branch Connections in Carbon or Low-Alloy Steel ≥2 NPS, Circumferential and Longitudinal	Surface	100% of each circumferential and 2.5t of each longitudinal weld requiring examination. See Note 2 at end of Table 2 for selection criteria.	The welds will be examined with MT.
C7.30	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for piping to be examined during system pressure test. Examination to be performed in accordance with IWC-5221 for each inspection period.	VT examinations will be performed in accordance with IWA-5240.
C7.40	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for piping to be examined during system hydrostatic test. Examination to be performed in accordance with IWC-5222 at or near the end of each inspection interval or during same inspection periods of each interval.	VT examinations will be performed in accordance with IWA-5240. Reference Relief Request No. 4.

42-52B

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PUMPS					
C3.30	C-C	Integrally Welded Attachments	N/A		100% of each weld to be examined. Attachments whose base material is 3/4 inch or greater to be selected. Selection limited to those components selected under Examination Category C-G. No pumps meet this criteria at R. E. Ginna.
C4.30	C-D	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No pressure-retaining pump bolting >2 inches in diameter at R. E. Ginna.
C6.10	C-G	Pressure-Retaining Welds in Pump Casings	N/A		No pressure-retaining pump welds at R. E. Ginna.
C7.50	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for pumps to be examined during pressure test. Examination to be performed in accordance with IWC-5221 for each inspection period.	VT examinations will be performed in accordance with IWA-5240.
C7.60	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for pumps to be examined during system hydrostatic test. Examination to be performed in accordance with IWC-5222 at or near the end of each inspection interval or during same inspection periods of each interval.	VT examinations will be performed in accordance with IWA-5240.

Table 2

INSERVICE INSPECTION PROGRAM
CLASS 2 COMPONENTS (Cont'd)

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
VALVES					
C3.40	C-C	Integrally Welded Attachments	Surface	100% of each weld to be examined. Attachments whose base material is 3/4 inch or greater to be selected. Selection limited to those components selected under Examination Category C-G.	The welds will be examined with MT or PT as applicable.
C4.40	C-D	Pressure-Retaining Bolting >2 Inches in Diameter	N/A		No pressure-retaining bolting >2 inches in diameter at R. E. Ginna.
C6.20	C-G	Pressure-Retaining Welds in Valve Bodies	Surface	100% of welds in components in each piping run examined under Examination Categories C-F-1 and C-F-2. In case of multiple valves of similar design, size, function, and service in a system, the examination of only one valve is required.	The welds will be examined with MT or PT as applicable.
C7.70	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for valves to be examined during system pressure test. Examination to be performed in accordance with IWC-5221 for each inspection period.	VT examinations will be performed in accordance with IWA-5240.
C7.80	C-H	Pressure-Retaining Components	Visual (VT-2)	All pressure-retaining boundaries for valves to be examined during system hydrostatic test. Examination to be performed in accordance with IWC-5222 at or near the end of each inspection interval or during same inspection periods of each interval.	VT examinations will be performed in accordance with IWA-5240.

Table 2 - NOTES

(1) The welds selected for examination shall include 7.5%, but not less than 28 welds, of all austenitic stainless steel or high-alloy welds not exempted by IWC-1220. (Some welds not exempted by this Case are not required to be nondestructively examined per Examination Category C-F-1. These welds, however, shall be included in the total weld count to which the 7.5% sampling rate is applied.) The examinations shall be distributed as follows:

(a) the examinations shall be distributed among the Class 2 systems prorated, to the degree practicable, on the number of nonexempt austenitic stainless steel or high-alloy welds in each system (i.e., if a system contains 30% of the nonexempt welds, then 30% of the nondestructive examinations required by Examination Category C-F-1 should be performed on that system);

(b) within a system, the examinations shall be distributed among terminal ends [see Note (3)] and structural discontinuities [see Note (4)] prorated, to the degree practicable, on the number of nonexempt terminal ends and structural discontinuities in that system; and

(c) within each system, examinations shall be distributed between line sizes prorated to the degree practicable.

(2) The welds selected for examination shall include 7.5%, but not less than 28 welds, of all carbon or low-alloy welds not exempted by IWC-1220. (Some welds not exempted by this Case are not required to be nondestructively examined per Examination Category C-F-2. These welds, however, shall be included in the total weld count to which the 7.5% sampling rate is applied.) The examinations shall be distributed as follows:

(a) the examinations shall be distributed among the Class 2 systems prorated, to the degree practicable, on the number of nonexempt carbon or low-alloy welds in each system (i.e., if a system contains 30% of the nonexempt welds, then 30% of the nondestructive examinations required by Examination Category C-F-2 should be performed on that system);

(b) within a system, the examinations shall be distributed among terminal ends [see Note (3)] and structural discontinuities [see Note (4)] prorated, to the degree practicable, on the number of nonexempt terminal ends and structural discontinuities in that system; and

(c) within each system, examinations shall be distributed between line sizes prorated to the degree practicable.

(d) Only those welds showing reportable preservice transverse indications need to be examined for transverse reflectors.

(3) Terminal ends are the extremities of piping runs that connect to structures, components (such as vessels, pumps, valves), or pipe anchors, each of which acts as a rigid restraint or provides at least two degrees of transitional restraint to piping thermal expansion.

(4) Structural discontinuities include pipe weld joints to vessel nozzles, valve bodies, pump casings, pipe fittings (such as elbows, tees, reducers, flanges, etc., conforming to ANSI B16.9), and pipe branch connections and fittings.

Table 3

SCHEME
CODE CLASS

Item No.	Examinatio nCategory	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
The ASME Section XI Item No. and Category of the component are listed in these columns.		Each type of examination area is listed in this column.	The NDE method required to satisfy Code requirements is listed in this column.	This column provides information regarding the number and/or percent of examinations required to be performed for the inspection interval.	This column provides information specific to examination techniques and examination areas.

Table 3

INSERVICE INSPECTION PROGRAM
CLASS 3 COMPONENTS

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
D1.10 D2.10 D3.10	D-A D-B D-C	Pressure-Retaining Components	Visual (VT-2)	All components to be examined during system pressure or system hydrostatic test. Examination to be performed in accordance with IWD-5221 for each inspection period and performed once each interval in accordance with IWD-5223.	VT examinations will be performed in accordance with IWA-5240.
D1.20 through D1-.60 D2.20 through D2-.60 D3.20 through D3-.60	D-A D-B D-C	Integral Attachments of Supports and Restraints, Hydraulic Snubbers, Spring, Constant Load, and Shock Absorbers	Visual (VT-3)	All required attachments to be examined during each inspection interval. For multiple components in a system of similar design, function, and service, the integral attachment of only one of the multiple components shall be examined. The integral attachments selected shall correspond to those support components selected for examination in accordance with IWF-2510(b).	The integral attachments will be examined.

Table 4

SCHEME
CODE CLASS

Item No.	Examinatio nCategory	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
The ASME Section XI Item No. and Category of the component are listed in these columns.		Each type of examination area is listed in this column.	The NDE method required to satisfy Code requirements is listed in this column.	This column provides information regarding the number and/or percent of examinations required to be performed for the inspection interval.	This column provides information specific to examination techniques and examination areas.

Table 4

INSERVICE INSPECTION PROGRAM

CLASS 1, CLASS 2, AND CLASS 3 COMPONENT SUPPORTS

Item No.	Examination Category	Components and Parts To Be Examined	Examination Method	Examination Requirements for Third Inspection Interval	Examination Technique/Examination Area Comments
PLATE AND SHELL TYPE SUPPORTS, LINEAR TYPE SUPPORTS, AND COMPONENT STANDARD SUPPORTS					
F1.10 through F1-.40 F2.10 through F2-.40 F3.10 through F3-.50	F-A F-B F-C	Mechanical Connections to Pressure-Retaining Components and Building Structure; Weld Connections to Building Structure; Weld and Mechanical Connections at Intermediate Joints in Multiconnected Integral and Nonintegral Supports; and Component Displacement Settings of Guides and Stops, Misalignment of Supports, Assembly of Support Items; Spring Type Supports; Constant Load Type Supports; Shock Absorbers; Hydraulic Type Snubbers	Visual (VT-3)	Component supports to be selected for examination are the supports of the nonexempt Class 1, 2, and 3 components scheduled to be examined. Examination boundaries established in accordance with IWF-1300. Examinations may be performed during normal system operation or plant outages.	Functional testing of snubber type support components shall be performed in accordance with Technical Specifications and in accordance with requirements of this program.

TABLE 5

Safety Related Hydraulic Snubber

Snubber No.	System Snubber Installed on and Analysis Line Number	Accessible or Inaccessible (A or I
AFU-205	Standby Auxiliary Feedwater	A
AFU-208	Standby Auxiliary Feedwater	A
AFU-209	Standby Auxiliary Feedwater	A
AFU-224	Standby Auxiliary Feedwater	A
AFU-225	Standby Auxiliary Feedwater	A
AFU-226	Standby Auxiliary Feedwater	A
AFU-229	Standby Auxiliary Feedwater	A
AFU-109	Auxiliary Feedwater - 500	A
FWU-3	Feedwater - 100	A
FWU-5	Feedwater - 100	A
FWU-21	Feedwater - 301	A
FWU-44	Feedwater - 300	A
N-601	Pressurizer Relief Discharge	A
N-602	Pressurizer Relief Discharge	A
N-604	Pressurizer Relief Discharge	A
N-605	Pressurizer Relief Discharge	A
N-607	Pressurizer Relief Discharge	A
N-608	Pressurizer Relief Discharge	A
N-615	Pressurizer Relief Discharge	A
N-616	Pressurizer Relief Discharge	A
MSU-8	Main Steam - 200	I
PS-2	Pressurizer PORV Discharge	A
PS-4	Pressurizer PORC Discharge	A
PS-5	Pressurizer PORV Discharge	A
PS-6	Pressurizer PORC Discharge	A
PS-8	Pressurizer PORV Discharge	A
PS-9	Pressurizer PORC Discharge	A
PS-10	Pressurizer PORV Discharge	A
PS-11	Pressurizer PORC Discharge	A

TABLE 5 (Con't)

Safety Related Mechanical Snubbers

Snubber No.	System Snubber Installed on and Analysis Line Number	Accessible or Inaccessible (A or I
AFU-3	Auxiliary Feedwater	A
AFU-31	Auxiliary Feedwater	A
AFU-34	Auxiliary Feedwater	A
AFU-52	Auxiliary Feedwater	A
AFU-75	Auxiliary Feedwater	A
AFU-98	Auxiliary Feedwater	A
AFU-101	Auxiliary Feedwater	A
AFU-103 (East)	Auxiliary Feedwater	A
AFU-103 (West)	Auxiliary Feedwater	A
AFW-111	Auxiliary Feedwater	A
AFW-123	Auxiliary Feedwater	A
AFW-124	Auxiliary Feedwater	A
AFU-227	Standby Auxiliary Feedwater	A
BDU-16	Steam Generator Blowdown - 200	I
CCU-43	Component Cooling - 450	I
CCU-57	Component Cooling - 700	I
CCU-71	Component Cooling - 600	I
CVU-26	Chemical and Volume Control - 730	I
CVU-46	Chemical and Volume Control - 300	I
CVU-49	Chemical and Volume Control - 700	I
CVU-80	Chemical and Volume Control - 100	I
CVU-103	Chemical and Volume Control - 500	I
CVU-104	Chemical and Volume Control - 500	I
CVU-131	Chemical and Volume Control - 500	I
CVU-186	Chemical and Volume Control - 250	I
CVU-345	Chemical and Volume Control - 900	I
CVU-351	Chemical and Volume Control - 900	I
CVU-372	Chemical and Volume Control - 1000	I
CVU-550	Chemical and Volume Control - 1000	I
FWU-8	Feedwater - 200	A
FWU-12	Feedwater - 200	A
FWU-15	Feedwater - 301	A
FWU-17	Feedwater - 301	A

TABLE 5 (Con't)

Safety Related Mechanical Snubbers

Snubber No.	System Snubber Installed on and Analysis Line Number	Accessible or Inaccessible (A or I
FWU-18	Feedwater - 301	A
FWU-20	Feedwater - 301	A
FWU-23	Feedwater - 301	A
FWU-24	Feedwater - 301	A
FWU-26	Feedwater - 301	A
FWU-32	Feedwater - 300	A
FWU-38	Feedwater - 300	A
FWU-39	Feedwater - 300	A
FWU-40	Feedwater - 300	A
FWU-42	Feedwater - 300	A
FWU-47	Feedwater - 300	A
FWU-48	Feedwater - 300	A
FWU-51	Feedwater - 300	A
FWU-52	Feedwater - 300	A
FWU-54	Feedwater - 300	A
FWU-57	Feedwater - 300	A
MSU-2	Main Steam - 100	I
MSU-3	Main Steam - 100	I
MSU-7 (Top)	Main Steam - 200	I
MSU-7 (Bottom)	Main Steam - 200	I
MSU-12	Main Steam - 300	A
MSU-13 (East)	Main Steam - 300	A
MSU-13 (West)	Main Steam - 300	A
MSU-15 (North)	Main Steam - 300	A
MSU-15 (South)	Main Steam - 300	A
MSU-16 (North)	Main Steam - 300	A
MSU-16 (South)	Main Steam - 300	A
MSU-18 (North)	Main Steam - 300	A
MSU-18 (South)	Main Steam - 300	A
MSU-19 (North)	Main Steam - 300	A
MSU-19 (South)	Main Steam - 300	A
MSU-22	Main Steam - 300	A
MSU-25	Main Steam - 300	A
MSU-26 (Top)	Main Steam - 300	A
MSU-26 (Bottom)	Main Steam - 300	A
MSU-27	Main Steam - 300	A

TABLE 5 (Con't)

Safety Related Mechanical Snubbers

Snubber No.	System Snubber Installed on and Analysis Line Number	Accessible or Inaccessible (A or I
MSU-29	Main Steam - 300	A
MSU-31	Main Steam - 300	A
MSU-32	Main Steam - 300	A
MSU-38	Main Steam - 300	A
MSU-39	Main Steam - 300	A
MSU-40	Main Steam - 300	A
MSU-44	Main Steam - 300	A
MSU-55	Main Steam - 300	A
MSU-57	Main Steam - 300	A
MSU-58	Main Steam - 300	A
MSU-60	Main Steam - 300	A
MSU-72	Main Steam - 120	A
MSU-74	Main Steam - 120	A
MSU-75	Main Steam - 120	A
MSU-78	Main Steam - 120	A
MSU-80	Main Steam - 120	A
MSU-82	Main Steam - 120	A
MSU-84 (East)	Main Steam - 120	A
MSU-84 (West)	Main Steam - 120	A
MSU-85	Main Steam - 120	A
RHU-8	Residual Heat Removal - 100	A
RHU-30	Residual Heat Removal - 2500	A
RHU-33	Residual Heat Removal - 2500	A
RHU-36	Residual Heat Removal - 350	A
RHU-51	Residual Heat Removal - 400	A
RHU-53	Residual Heat Removal - 400	A
RHU-61	Residual Heat Removal - 400	A
RHU-63 (North)	Residual Heat Removal - 300	A
RHU-63 (South)	Residual Heat Removal - 300	A
RHU-69	Residual Heat Removal - 300	A
RHU-71 (North)	Residual Heat Removal - 300	A
RHU-71 (South)	Residual Heat Removal - 300	A
RHU-72	Residual Heat Removal - 300	A
RHU-75	Residual Heat Removal - 300	A
RHU-92	Residual Heat Removal - 300	A
RHU-109	Residual Heat Removal - 450	A

TABLE 5 (Con't)

Safety Related Mechanical Snubbers

Snubber No.	System Snubber Installed on and Analysis Line Number	Accessible or Inaccessible (A or I
RHU-110	Residual Heat Removal - 450	A
RHU-119	Residual Heat Removal - 450	A
RHU-123	Residual Heat Removal - 450	A
SIU-3	Safety Injection - 100	A
SIU-47	Safety Injection - 200	A
SIU-52	Safety Injection - 200	A
SWU-254	Service Water - 1850	A
SWU-308	Service Water - 1400	A
SWU-309	Service Water - 1400	A
SWU-370	Service Water - 1500	A
SGA-7	Steam Generator "A"	A
SGA-8	Steam Generator "A"	A
SGB-3	Steam Generator "B"	A
SGB-4	Steam Generator "B"	A

TABLE 6

HIGH ENERGY WELD OUTSIDE CONTAINMENT

1) MAIN STEAM LOOP A - OUTSIDE CV

Design Basis Break		Consequential Break	
30A-MS-600-1A	D	30A-MS-600-1A	D2
30A-MS-600-1A	D1	30A-MS-600-1A	D3
30A-MS-600-1A	F1	30A-MS-600-1A	E
36-MS-600-1	L1	30A-MS-600-1A	E1
		30A-MS-600-1A	E2
		30A-MS-600-1A	F
		30A-MS-600-1A	G
		30A-MS-600-1A	G1
		30A-MS-600-1A	G2
		30A-MS-600-1A	H
		30A-MS-600-1A	J
		30A-MS-600-1A	K
		30A-MS-600-1A	L

2) MAIN STEAM LOOP B - OUTSIDE CV

Design Basis Break		Consequential Break	
30B-MS-600-1B	D	30B-MS-600-1B	D1
30B-MS-600-1B	H1	30B-MS-600-1B	D2
30B-MS-600-1B	J1	30B-MS-600-1B	E
36-MS-600-1	P1	30B-MS-600-1B	J
		30B-MS-600-1B	K
		30B-MS-600-1B	L
		30B-MS-600-1B	L1
		30B-MS-600-1B	L2
		30B-MS-600-1B	M
		30B-MS-600-1B	N
		30B-MS-600-1B	O
		30B-MS-600-1B	P

TABLE 6 (Con't)

3) MAIN STEAM - TURBINE BUILDING

Design Basis Break		Consequential Break	
36-MS-600-1	L2	36-MS-600-1	L4
		24A-MS-600-1A	A
		24A-MS-600-1A	B
		24A-MS-600-1A	B1
		24A-MS-600-1A	C1
		24A-MS-600-1A	D
		24A-MS-600-1A	D1
		24B-MS-600-1B	A
		24B-MS-600-1B	B
		24B-MS-600-1B	B1
		24B-MS-600-1B	C
		24B-MS-600-1B	C1
		24B-MS-600-1B	D

4) FEEDWATER - TURBINE BUILDING

Design Basis Break		Consequential Break	
20-FW-900-1	M3	20-FW-900-1	J
		20-FW-900-1	K1
		20-FW-900-1	L
		20-FW-900-1	L1
		20-FW-900-1	M
		20-FW-900-1	M1
		20-FW-900-1	M2
		8-FW-900-1	A
		8-FW-900-1	B
		8-FW-900-1	C
		8-FW-900-1	D
		8-FW-900-1	E
		8-FW-900-1	F
		8-FW-900-1	G
		8-FW-900-1	H

TABLE 6 (Con't)

5) FEEDWATER LOOP A - OUTSIDE CV

Design Basis Break		Consequential Break	
20-FW-900-1	M4	14A-FW-900-1A	T2
20-FW-900-1	U1	14A-FW-900-1A	T3
20-FW-900-1	U2	14A-FW-900-1A	U7
14A-FW-900-1A	AA	14A-FW-900-1A	U5
		14A-FW-900-1A	U6
		14A-FW-900-1A	V
		14A-FW-900-1A	VA
		14A-FW-900-1A	VB
		14A-FW-900-1A	V1
		14A-FW-900-1A	V2
		14A-FW-900-1A	V2A
		14A-FW-900-1A	V2B
		14A-FW-900-1A	W
		14A-FW-900-1A	X
		14A-FW-900-1A	Y
		14A-FW-900-1A	Z
		14A-FW-900-1A	Z1
		14A-FW-900-1A	Z2
		14A-FW-900-1A	Z3
		14A-FW-900-1A	Z4

TABLE 6 (Con't)

6) FEEDWATER LOOP B - OUTSIDE CV

Design Basis Break		Consequential Break	
20-FW-900-1	A1	14B-FW-900-1B	F3
20-FW-900-1	F4	14B-FW-900-1B	F5
14B-FW-900-1A	N	14B-FW-900-1B	F1
14B-FW-900-1A	V	14B-FW-900-1B	F2
		14B-FW-900-1B	G
		14B-FW-900-1B	G2
		14B-FW-900-1B	G1
		14B-FW-900-1B	G3
		14B-FW-900-1B	G4
		14B-FW-900-1B	H
		14B-FW-900-1B	HA
		14B-FW-900-1B	HB
		14B-FW-900-1B	H1
		14B-FW-900-1B	H2
		14B-FW-900-1B	H2A
		14B-FW-900-1B	H2B
		14B-FW-900-1B	J
		14B-FW-900-1B	K
		14B-FW-900-1B	L
		14B-FW-900-1B	M
		14B-FW-900-1B	N1
		14B-FW-900-1B	O
		14B-FW-900-1B	01
		14B-FW-900-1B	P

[illegible]

TABLE 7

SUMMARY OF RELIEF REQUESTS

Relief Request Number	Section XI Reference	Component	ASME Requirement for Which Relief is Requested	Reason for Relief Request	Proposed Alternate Examination
1	IWB-2500-1 Cat. B-A	RPV Shell to Flange Weld	Volumetric Examination during two different periods.	To perform all examinations associated with the Shell-to-Flange during the same period.	Perform all examinations associated with the Shell-to-Flange at or near the end of the interval.
2	IWB-2500-1 Cat. B-D	RPV Nozzle-to-Vessel Welds	Volumetric examination during two different periods.	To perform all examinations associated with the Nozzle-to-Vessel welds during the same period.	Perform all examinations associated with the Nozzle-to-Vessel welds at or near the end of the interval.
3	IWA-1400	Authorized Inspection Agency.	Use of "Authorized Inspection Agency"	New York State has not endorsed ASME Codes and does not have an Authorized Inspection Agency.	Use R.E. Ginna Quality Assurance Program.
4	IWB-2500-1 Cat. B-L-1 B-L-2	Reactor Coolant Pump Case Welds and Intervals.	Volumetric examination of case welds and visual of internals.	Pump material and configuration.	Hydrostatic test, surface and visual exams of outside surfaces.
5	IWB-2500-1 Cat. B-M-2	Class 1 Valves Greater than NPS 4.	Visual Examination of valve internals.	Excessive radiation exposure and historical reliability of valves.	Examine valve internals when disassembled for maintenance.
6	IWD-2500-1 Cat. DB	Radioactive Waste Holdup Tank.	Visual Examination at hydrostatic pressure.	Tank will be rendered inoperative during tests.	Perform visual examination each period at normal operating pressure.
7	IWC-5222(a)	Charging Pumps	Visual Examination at 3420 psig Hydrostatic Pressure.	Pumps have maximum pressure limit on the seals.	Perform Hydro Test and visual at 2400 psig.
8	IWC-5222(a)	Valves PCV 430 and PCV 431C	Visual Examination at Hydrostatic Pressure.	Valve diaphragms cannot withstand Test Pressure.	Hydrostatic Test to Flex Connection operate diaphragm per vale test requirements and perform inservice visual examination once per period.

TABLE 7 (Con't)

SUMMARY OF RELIEF REQUESTS

Relief Request Number	Section XI Reference	Component	ASME Requirement for Which Relief is Requested	Reason for Relief Request	Proposed Alternate Examination
9	IWC-5222(a)	Secondary Side of steam generator and associated main steam piping.	Visual Examination at Hydrostatic Pressure.	RG&E adopted pressure differential limitation of 800 psig to prevent primary side tube sheet cladding separation.	Hydrostatic Test at 1.10 times instead of 1.25 Psv setting and perform visual examination.
10	IWD-5223(a)	Standby Auxiliary Pump Recircul. Line from AOV 9710A, AOV 9710B and their associated downstream flow orifices.	Visual Examination at Hydrostatic Pressure.	Pressure Reduction Flow Orifice requires removal and blank-off. System piping does not provide isolation to condensate supply tank. Significant tank reduction would be required for removal and is considered impractical.	Perform Inservice Visual Examination once per period.
11	IWD-5223(a)	Boric Acid Filter and associated piping between Valves 347, 348A and 349A.	Visual Examination at Hydrostatic Pressure.	Test Pressure required will exceed limits for safe working pressure on Boric Acid Filter Housing Flange Gaskets.	Perform Inservice Visual Examination once per period.
12.1	IWD-5223(a)	Air Start for Diesel Generator including Receiver Tanks and Piping.	Visual Examination at Hydrostatic Pressure.	Air Start Pressure Test would require termination prior to reaching engine skid to preclude air to air start motors, leaving portion of piping untestable.	Perform Inservice Visual Examination once per period and once each quarter a pressure decay test performed on air receiver.
12.2	IWD-5223(a)	Fuel Oil Transfer Pumps and associated piping to terminals at Oil Storage Tank.	Visual Examination at Hydrostatic Pressure.	Requires isolation of Diesel Oil Storage and Dry Tank where no no means of isolation is provided at Transfer pump discharge piping with tanks vented to atmosphere.	Perform system functional Testing with associated visual Visual Examination once per period.

TABLE 7 (Con't)

SUMMARY OF RELIEF REQUESTS

Relief Request Number	Section XI Reference	Component	ASME Requirement for Which Relief is Requested	Reason for Relief Request	Proposed Alternate Examination
12.3	IWD-5223(a)	Jacket Cooling Water System and associated piping to terminals at Water Expansion Tanks.	Visual Examination at Hydrostatic Pressure.	Cooling Water Expansion Tank vented to atmosphere requiring isolation with vast piping involved will be unable to pressurize.	Perform System Functional Testing with associated Visual Examination once per period.
13	IWC-5222(a)	Non-ISI classified systems penetrating primary containment.	Visual Examination at Hydrostatic Pressure.	Requirements should be based on the containment system design not the associated process system design requirement.	Test in accordance with 10CFR50 Appendix J with the safety function the line performs in accordance with Technical Specifications, surveillance requirements, in addition, perform Inservice Visual Examination on exposed portions.
14	IWD-5223(a)	Class 3 Portion of the service water system.	Visual Examination at Hydrostatic Pressure.	Hydrostatic Testing is impractical due to system design that is open-ended and employing butterfly valves that were not designed to provide a leaktight boundary.	Perform Inservice Visual Examination once per period.
15	IWC-2500 Cat. C-F-1 & C-F-2 Items C5.10 and C5.50, resp.	Class 2 Piping Welds < 3/8" nominal wall thickness for piping > NPS4.	Piping Welds < 3/8" nominal wall thickness do not require surface and volumetric examinations.	At a minimum terminal connection welds of identified exempted welds per Items C5.10 and C5.50 of Table IWC-2500 should be examined.	Surface and Volumetric Examinations on terminal connection welds of identified exempted welds shall be performed to the requirements of IWC-2500-1.
16	IWB-2500 Cat. B-K-1 Item B10.10 and IWC-2500, Cat. C-C, Item C3.20.	Class 1 & 2 Integral Attach. on piping specifically support Attachments.	Volumetric or surface examination on Class 1 Integral Attachments is required on Base Attachment ≥ 5/8". Class 2 requires a surface examination on Base Attachments ≥ 3/4".	Integral Attachments on Class 1 & 2 support attachments should have a surface examination to insure safety and system integrity.	On Class 1 & 2 Integral Attachments, a surface examination shall be performed on support attachments to IWB and IWC-2500 requirements.

RELIEF REQUEST NO. 1
DEFER RPV EXAMINATIONS TO END OF INTERVAL

I. Components for Which Relief is Requested:

The component for which relief is requested is the Reactor Pressure Vessel (RPV) Shell-to-Flange Weld.

II. ASME Requirement from Which Relief is Requested:

Table IWB-2500-1, Examination Category B-A, requires that the RPV Shell-to-Flange weld be examined during the first and third periods in conjunction with the nozzle examinations, with at least 50 percent examined during the first period and the remainder by the end of the third period. The required Shell-to-Flange examination is impractical if performed during the periods specified as it can only be accomplished from the flange surface.

III. Proposed Alternate Method:

During the first two inspection intervals, 100 percent of the accessible length of the RPV welds including the Shell-to-Flange weld were examined at or near the end of the interval when the entire examination could be performed from both the flange surface and the vessel wall. This is a more practical approach in that the required examinations from both surfaces can be performed at the same time. During the third interval, 100 percent of the accessible length of all RPV welds including the shell-to-flange weld will be performed at or near the end of the interval when all the required examinations can be performed at the same time.

RELIEF REQUEST NO.2

I. Components for Which Relief is Requested:

The components for which relief is requested are the RPV Nozzle-to-Vessel welds and Nozzle Inside Radius Sections.

II. ASME Requirements for Which Relief is Requested:

Table IWB-2500-1, Examination Category B-D, Item B3.90, Nozzle-to-Vessel welds allows partial deferral. "If examinations are conducted from inside the component and the nozzle weld is examined by straight beam ultrasonic method from the nozzle bore, the remaining examinations required to be conducted from the shell inside diameter may be performed at or near the end of each interval."

Examination Category B-D, Item 3.100 Nozzle Inside Radius Sections, does not allow deferral to the end of the interval, and requires (footnote 2) 25 percent to 50 percent of the nozzles to be examined during the first period, with the remainder to be examined at the end of the interval.

Examinations from the nozzle bore and nozzle inside radius examinations can only be performed on two (outlets) of the six major nozzles without removal of the core barrel. The mechanized examination of the two accessible nozzle and inside radius sections is quite expensive, and the nozzle-to-vessel examination is only a partial examination from the nozzle bore. From a technical position considering the progress which is being made in ultrasonic examination equipment and techniques and for the correlation of data obtained from the bore with that obtained from the shell, it is highly desireable to perform both examinations at the same time.

RELIEF REQUEST NO.2 (Con't)

III. Proposed Alternate Method:

Rochester Gas & Electric (RG&E) proposes to perform both nozzle-to-vessel examinations (from the nozzle bore and from the shell inside diameter) at or near the end of the interval. The nozzle inside radius examinations will also be performed at this time. This more practical approach will allow all the required examinations to be performed at the same time on all the nozzles and nozzle inside radii.

RELIEF REQUEST NO. 3

I. Examination Requirement for Which Relief is Requested:

The ASME Boiler and Pressure Vessel Code, Section XI, 1986 Edition, IWA-1400(f), requires an arrangement with an Authorized Inspection Agency to provide inspection services. In addition, the Code requires that certain administrative functions be performed by the "Enforcement Authority" and "Authorized Nuclear Inservice Inspector."

II. Proposed Alternative:

Ginna Station is located in the state of New York. This state has not endorsed ASME Codes and therefore does not provide administrative organization and controls such as "Enforcement Authority," "Authorized Nuclear Inservice Inspector" and "Reporting Systems." However, Ginna Station's Quality Assurance Program does provide equivalent administrative control. Therefore, RG&E requests that Ginna's Station Quality Assurance Program be used in lieu of Code administrative functions.

Rochester Gas & Electric's program for the inservice inspection, governed by the R.E. Ginna Station Quality Assurance Manual, contains the requirements and responsibilities for implementation of the program and procedures. The procedures have been prepared and approved by the responsible organizations within Rochester Gas & Electric (e.g., Ginna Station, Engineering, Materials Engineering and Inspection Services, Electric Meter and Laboratory and Purchasing).

Approved procedures will be implemented to control the standards for examination evaluation. These procedures include the identifications of the organization performing the inspection, description of the method of inspection to be used, acceptance and rejection criteria, and requirements for providing evidence of completion and certification of the inspection activity.

RELIEF REQUEST NO.3 (Con't)

In addition, procedures are developed by Ginna Station to prescribe the disposition of nonconformances. The procedures implemented for the repairs, the retest procedures and the test results will be reviewed by the Plant Operating Review Committee. The members of this committee include technically qualified staff personnel.

Examination techniques have been established in accordance with written requirements and incorporated into written procedures. Qualifications for nondestructive test personnel are in compliance with Regulatory Guide 1.58, "Qualification of Nuclear Power Plant Inspection, Examination and Testing Personnel."

Records and reports of the inservice inspection will be developed and maintained by Rochester Gas and Electric and include such items as examination plans and schedules, examination of results and corrective actions.

The functions of the authorized nuclear inservice inspector, namely their review and verification of inservice examinations, personnel qualification and equipment certification during the annual outages at Ginna Station will be performed by personnel of the Hartford Steam Boiler Inspection and Insurance Company. The qualifications of the inspectors, inspections specialists and inspection agency are in compliance with the Code.

RELIEF REQUEST NO.4

I. Components for Which Relief is Requested:

Each of the 27.5 inch diameter recirculation loops at R.E. Ginna has a Class 1 Reactor Coolant Pump. The function of these two pumps is to provide forced circulation through the RPV core during normal reactor operation.

II. Code Requirement for Which Relief is Requested:

Table IWB-2500-1, Examination Categories B-L-1 and B-L-2 require volumetric examination of casing welds and visual examination of internal pressure boundary surfaces of one pump case in each of the pump groups performing similar system functions each inspection interval. These examinations are impractical for the reactor coolant pumps at Ginna Station and relief is, therefore, requested.

A. Supporting Information

1. The two reactor coolant pumps (RCP) for R.E. Ginna are Westinghouse Model 93 pumps. Each pump casing is fabricated by welding four stainless steel (SA351 CF8) castings together. Thus, there are 3 circumferential pressure-retaining welds that are to be volumetrically inspected in accordance with Category B-L-1.

2. The unsuitability of ultrasonic examination was demonstrated during the "A" reactor coolant pump examination in 1980. An attempt was made to determine the wall thicknesses using ultrasonic examination, the casing welds must be inspected using the miniature linear accelerator (MINAC).

3. Radiographic examination using the MINAC was performed on the R.E. Ginna "A" RCP during the Spring 1981 refueling outage. In addition, the same type of examination has been performed at several other sites.

RELIEF REQUEST NO.4 (Con't)

This examination was performed by placing the MINAC inside the pump casing and placing the film on the outside of the pump. To perform the examination, the pump was completely disassembled. Disassembly to this extent is far beyond any disassembly expected for this examination. Also, insulation on the casing exterior was removed for film placement.

Additionally, the pump bowl must be dry for installation of the MINAC. Therefore, all fuel assemblies were removed from the reactor vessel and the vessel water level lowered to below the nozzles. Complete disassembly of the pump was also required to conduct the VT-1 examination in accordance with Category B-L-2.

4. No problems have been found with the welds at R.E. Ginna or other sites. Additionally, no problems have been found during the Category B-L-2 visual examination. The visual examination was conducted at R.E. Ginna by using the video camera on the MINAC.

   The whole body exposure to personnel during the Spring 1981 directly attributable to the RCP "A" examinations 93,067 millirem. This does not include the dose received during the complete core unload to get the plant in condition for the RCP disassembly.

RELIEF REQUEST NO.4 (Con't)

5. The nuclear industry has been successfully applying leak-before-break concepts to primary loop and Class 1 auxiliary piping systems of commercial nuclear power plants. Currently, the analyses supporting such concepts comes under the review of the Nuclear Regulatory Commission by General Design Criteria-4 (GDC-4).

   There are eight different models of RCP's in Westinghouse-type PWRs . Model 93 methodology used in the analyses is consistent with that recommended in NUREG 1061, Vol. 3 and GDC-4. A finite element stress analysis model for the Model 93 pump was developed.

   The RCP casings are cast stainless steel. The chemistries of each heat of material used in the pumps were used to determine the fracture toughness. The phenomenon of thermal aging was addressed.

   The program successfully demonstrates that leak-before-break analyses are applicable to all primary pump casings of all Westinghouse design PWRs for which the screening loads are reasonably applicable and the fracture toughness are known.

6. We believe that performing a volumetric examination of the Ginna Station Unit 1 RCP casing welds and a visual examination of the interior pressure retaining surface of one pump during the third 10-year inspection period does not provide an increase in safety and expected radiation exposure. The following items have been considered:

   a. Visual examination (VT-2) of the exterior of all pumps during the hydrostatic pressure test required by Table IWB 2500-1 Category B-P.

RELIEF REQUEST NO.4 (Con't)

b. Perform a visual examination (VT-1) of the external surfaces of the welds of one pump casing.

c. Perform a visual examination (VT-3) of the internal surfaces each time pump disassembly is required for maintenance.

d. Perform an evaluation to demonstrate the safety and serviceability of the pump casing. The evaluation will include:

(i) Establishing material properties including fracture toughness values.

(ii) Performing a stress analysis of the structure.

(iii) Reviewing of the operating history of the structure

(iv) Selection of locations for postulating flaws

(v) Determination of a flaw size resulting in the detectable leak rate

(vi) Establishing the stability of the selected flaw

(vii) Demonstration that a postulated through-wall flaw which yields detectable leakage remains stable for all design loadings, with a margin of 2 on flaw size

RELIEF REQUEST NO.4 (Con't)

NOTE: In making this assessment, thermal aging embrittlement and any other processes which may degrade the properties of the pump casing during service will be considered.

RELIEF REQUEST NO. 5

I. Components for Which Relief is Requested:

Class 1 valves requiring valve body internal VT-3 examination.

Size (In.)	Valve No.	MFG/Type	Line No.
10	842A	Darling/Check	10-SI2-2501-A
10	842B	Darling/Check	10A-SI2-2501-B
10	867A	Darling/Check	10-SI2-2501-B
10	867B	Velan/Gate	10A-SI2-2501-B
10	700	Velan/Gate	10A-RCO-2501-A
10	701	Velan/Gate	10A-AC7-2501-A
10	720	Velan/Gate	10A-AC7-2501-B
10	721	Velan/Gate	10A-AC7-2501-B
6	853A	Velan/Gate	6A-RC-2501-A
6	853B	Velan/Gate	6A-RC-2501-B
6	852A	Velan/Check	6A-RC-2501-A
6	852B	Velan/Check	6A-RC-2501-B

II. ASME Requirement for Which Relief is Requested

Table IWB-2500-1, Examination Category B-M-2, requires an internal VT-3 examination on at least one valve within each group of valves that are of the same size, constructional design (such as globe, gate or check valves) and manufacturing method, that perform similar functions in the system. This relief request is based on the following points:

1. to complete the subject examination, unnecessary expenditures of man-hours and manrem are required with essentially no compensating increase in plant safety, and

RELIEF REQUEST NO.5 (Con't)

2. the structural integrity afforded by valve casing material utilized will not significantly degrade over the lifetime of the valve.

Based on data compiled from a plant similar in age and design to Ginna Station, it is expected that approximately 100 manhours and 5 manrem exposure would be required to disassemble, inspect, and reassemble these vales. Performing this visual examination under such adverse conditions, high doses rate (30-40 R/hr), and poor as-cast surface conditions, realistically provides little additional information as to the valve's casing integrity.

The valves material, a high-strength cast stainless steel (ASTM A351-CF8), is widely used in the nuclear industry and has performed extremely well. The presence of some delta ferrite (typically 5% or more) substantially increases resistance to intergranular stress corrosion cracking. The delta ferrite also helps the material to resist pitting corrosion in chloride containing environments.

RG&E feels that adequate safety margins are inherent in the basic valve design and that the public's health and safety will not be adversely affected by not performing a visual examination of the valve internal pressure boundary surfaces. Additionally, this visual examination adds little or no value to the overall safety of the plant and subjects plant personnel to unnecessary radiation exposure. Therefore, a request for relief from this requirement is sought.

RELIEF REQUEST NO.5 (Con't)

III. Proposed Alternative Method:

As stated above, RG&E does not believe that the visual examination required each ten-year interval is warranted. However, as standard maintenance practice dictates, when these valves are disassembled for maintenance purposes, a visual examination of the internals and internal pressure boundary surfaces will be performed, to the extent practical.

[illegible]

RELIEF REQUEST NO.6

I. Component for Which Relief is Requested:

The radioactive waste hold-up tank in the waste disposal system provides a means of storing contaminated water that has been used in the operation of the nuclear power plant. The waste disposal system and waste hold-up tank may be required to function in all modes of reactor operation including cold shutdown and refueling.

II. ASME Requirement for Which Relief is Requested:

Table IWD 2500-1, Examination Category D-B, Item No. D2.10, requires VT-2 examination of the waste hold-up tank at hydrostatic testing levels (at least 1.10 system pressure) during each interval as well as VT-2 examinations at nominal operating pressure during each period.

The design of the waste disposal system is such that contaminated water is stored in the waste holdup tank until such time as the level of contamination is below the limits for discharge. At this time the holdup tanks may be reavailable for use by emptying the stored liquid.

Several important systems within the chemical volume and control system drain into the waste disposal system hold-up tanks. These are the volume and control tank drains, reactor coolant letdown system, reactor coolant drain tank discharge, and the demineralizer system drains.

If the tank was to be hydrostatically tested by filling it with water and pressurizing to 1.10 system pressure, the hold-up tank would be rendered useless. The plant would then be potentially put into an unsafe condition for any abnormal plant function and if startup occurred without a holdup tank being available.

RELIEF REQUEST NO.6 (Con't)

Since this hold-up tank constantly stores liquid, any degradation of the tank material would show up prior to it becoming a problem. RG&E believes that hydrostatically testing the rad-waste hold-up tank puts Ginna's plant in an unsafe condition and therefore a request for relief from this requirement is sought.

III. Proposed Alternative Method:

A Visual VT-2 examination shall be performed once every period with the system at normal operating pressure to verify continue structural integrity.

[illegible]

RELIEF REQUEST NO.7

I. Components for which Relief is Requested:

CVCS, Three Charging Pumps and Discharge Piping to Discharge Isolation Valves.

II. ASME Requirements for which Relief is Requested:

IWC-5222(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. This corresponds to a test pressure of 3420 psig.

The charging pumps have a minimum hydrostatic test pressure limitation on the seals of 2400 psig, as specified by the pump manufacturer. As a result, the pumps and associated discharge piping to the first isolation valves cannot be tested to the required Code Test Pressure.

III. Proposed Alternate Method:

During the hydrostatic test and associated VT-2 examination, the charging pumps and associated discharge piping to the first isolation valves will be tested at a pressure of 2400 psig.

RELIEF REQUEST NO.8

I. Components for Which Relief is Requested:

RCS Overpressure Protection Nitrogen Accumulator System Valves PCV 430 and PCV 431C

II. ASME Requirements for Which Relief is Requested:

IWC-5222(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. This corresponds to a test pressure of 137.5 psig.

The diaphragms in the operators of the subject valves are only designed to withstand a maximum pressure of 105 psig, and therefore cannot be tested to the required Code test pressure.

III. Proposed Alternate Method:

The RCS overpressure nitrogen accumulator system will be tested to the Code requirements up to the flex connection to the valve operator. Operability of the diaphragm and operator is verified by valve testing requirements. In addition, an inservice pressure test at operating pressure will be performed once each inspection period on the piping, including the diaphragm.

RELIEF REQUEST NO.9

I. Components for Which Relief is Requested:

Main Steam Secondary Side of Steam Generator and Downstream Piping to Class Boundary.

II. ASME Requirement from Which Relief is Requested:

IWC-5222(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. Since the design temperature of the Main Steam system is greater than 200°F, the test pressure is required to be 1.25 times Psv, or 1356 psig.

A pressure differential limitation of 800 psig between the primary and secondary side of the Steam Generator has been adopted. This was established early in plant life due to the experiences of some plants with primary side tube sheet cladding separation. To maintain this 800 psig differential, and the required pressure on the secondary side, the primary system must be heated up to a minimum of 160°F which would result in a problem with heat balance and a potential operational problem during implementation of the test procedure. The administrative controls necessary to assure a proper and safe test and the complexity required for the test procedure result in a situation that should be minimized.

In addition to the Section XI volumetric and surfaces examination requirements, the piping is part of the augmented inspection program since it falls within the high energy break criteria.

RELIEF REQUEST NO.9 (Con't)

A letter was submitted to Dennis L. Ziemann Chief Operating Reactor Branch #2, USNRC, Dated: November 8, 1979, requesting relief.

Subject: System Pressure Test Restriction for Steam Generator and associated Feedwater and Main Steam piping, R.E. Ginna Nuclear Power Plant #1, Docket No. 50-244.

III. Proposed Alternate Method:

Test the secondary side of the Steam Generator and associated Main Steam piping at a pressure of 1194 psig, which corresponds to 1.10 times the Psv setting.

These components are inside containment and any significant leakage would be detected by various leakage monitoring systems during plant operation.

RELIEF REQUEST NO.10

I. Component for Which Relief is Requested:

Feedwater, Standby Auxiliary Pump recirculation line between AOV 9710A, AOV 9710B and their associated downstream flow orifices.

II. ASME Requirement from Which Relief is Requested:

IWD-5223(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. For systems (or portions of systems) not provided with safety or relief valves, the system design pressure Pd shall be substituted for Psv.

In order to hydrotest this piping to Section XI requirements, the pressure reducing flow orifices downstream of AOV 9710A & B would require removal and blank flanges installed. System piping does not provide an isolation valve between the orifices and the Condensate Supply Tank. A significant reduction in tank level would be required to facilitate orifice removal, which is considered to be impractical.

III. Proposed Alternate Method:

The Class 3 portion of this piping shall be VT-2 examined at operational discharge pressure during functional testing which is performed once each period.

RELIEF REQUEST NO. 11

I. Component for Which Relief is Requested:

CVCS, Boric Acid Filter (CSFLBA) and all piping between valves 347, 348A and 349A.

II. ASME Requirement from Which Relief is Requested:

IWD-5223(a) System Hydrostatic Test: The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. For systems (or portions of systems) not provided with safety or relief valves, the system design pressure Pd shall be substituted for Psv.

The hydrostatic test pressure necessary to satisfy Section XI requirements will exceed the limits for the safe working pressure on the Boric Acid Filter housing flange gaskets.

III. Proposed Alternate Method:

The Boric Acid Filter and associated piping shall be VT-2 examined, at full operational pressure during inservice testing which shall be performed once each period.

RELIEF REQUEST NO. 12

I. Component for Which Relief is Requested:

Emergency Diesel Generation:

1. Starting Air including receiver tanks and associated piping.
2. Fuel Oil Transfer pumps, suction and discharge lines including miscellaneous lines terminating at oil storage tanks.
3. Jacket Cooling Water system including miscellaneous line terminating at cooling water expansion tanks.

II. ASME Requirement from Which Relief is Requested:

IWD-5223(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. For systems (or portions of systems) not provided with safety or relief valves, the system design pressure Pd shall be substituted for Psv.

Only portions of the piping associated with the components identified above are capable of being pressure tested.

The Air Start System pressure test would require termination prior to reaching the engine skid to preclude administrating air to the Air Start Motors. This would leave that portion of piping between the Air Start Motors to the first isolation, prior to reaching the engine skid, untestable.

RELIEF REQUEST NO. 12 (Con't)

The Diesel Fuel Oil Transfer system would require flange connection disassembly and the installation of blind flanges to isolate the Diesel Oil Storage Tank, which is vented to atmosphere, and the Day Tank where no means is provided to isolate the transfer pump discharge piping at a point close to the day tank. Additionally, the overflow piping form the day tank to the storage tank, which is identified as Class 3, has not isolation valves installed and is vented to the atmosphere.

The Jacket Cooling Water System would require isolating the Cooling Water Expansion Tank, due to vents to the atmosphere, which would include most of the piping subject to pressure testing. Due to the amount of piping within the class boundary which is unable to be pressurized, testing in accordance with Section XI requirements would not prove system integrity over and above the existing Surveillance Inservice and Functional Testing.

III. Proposed Alternate Method:

Inservice Testing shall be performed on the Air Start System at least once each period in accordance with the requirements of Section XI. Additionally, once each quarter a pressure decay test shall be performed on the air receiver to verify check valve operability in the reverse direction for the air receiver inlet check.

System Functional Testing shall be performed at least once each period on the Diesel Fuel Oil Transfer and Jacket Cooling Water Systems in accordance with the requirements of Section XI.

In addition to the testing discussed above, Technical Specifications 6.4.1 requires surveillance testing to be performed on a monthly basis. Such as, verifying operability of the fuel oil transfer pumps and verifying that the diesel starts from normal standby conditions.

[illegible]

RELIEF REQUEST NO. 13

I. Components for Which Relief is Requested:

Non-ISI classified systems, which do not carry radioactive gases or fluids, that contain line penetrating primary containment.

II. ASME Requirement from Which Relief is Requested:

IWC-5222(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested.

The safety function of these lines is to become part of the containment isolation system during periods when containment isolation is required. Therefore, the pressure testing requirements should be based on the containment system design not the associated process system design requirements.

III. Proposed Alternate Method:

Test these lines in accordance with 10CFR50 Appendix J, Reactor Containment Leakage Testing for Water Cooled Power Reactor commensurate with the safety function the line performs in accordance with Technical Specification, Surveillance requirements. Additionally, at least once each period exposed portions of the lines penetrating primary containment will be examined during normal system operation.

RELIEF REQUEST NO. 14

I. Component for Which Relief is Requested:

Service Water, All pressure retaining components within the Class 3 portion of the Service Water System.

II. ASME Requirement for Which Relief is Requested:

IWD-5223(a) System Hydrostatic Test; The system hydrostatic test pressure shall be at least 1.10 times the system pressure Psv for systems with Design Temperature of 200°F (93C) or less, and at least 1.25 times the system pressure Psv for systems with Design Temperature above 200°F (93C). The system pressure Psv shall be the lowest pressure setting among the number of safety or relief valves provided for overpressure protection within the boundary of the system to be tested. For systems (or portions of systems) not provided with safety or relief valves, the system design pressure Pd shall be substituted for Psv.

Rochester Gas and Electric believes that the hydrostatic test requirement for the service water system is impractical due to system design which dictates the use of an open-ended test. The portion of the system downstream of the heat exchanger is also open-ended and cannot be hydrostatically tested. The remaining section of the system is only isolatable by means of butterfly valves which were not designed to provide a leak-tight boundary. With the system as such it would be impractical to expect the leakages other than at the valves could be detected.

The ample margin in cooling capacity inherently provided by system design does not dictate the need for an essentially leak-tight boundary. Since the system is in constant operation, its integrity is continually monitored. Thorough inspection of the system each period at the full operating pressure is adequate to detect any gross failures in the system without degrading system safety or availability.

RELIEF REQUEST NO. 14 (Con't)

III. Proposed Alternate Method:

Pressure retaining components within the operational boundary will receive an inservice test at operating pressure and an associated VT-2 examination each period during the interval.

RELIEF REQUEST NO. 15

I. Component for Which Relief is Requested:

Class 2, IWC 2500-1 Table Examination Category C-F-1 and C-F-2, Items C5.10 and C5.50.

II. ASME Requirements for Which Relief is Requested:

Category C-F-1 and C-F-2 (Items C5.10 and C5.50, respectively) for piping welds $\geq$ 3/8 inches nominal wall thickness for piping > NPS4, a surface and volumetric examination is required on 100% of each weld requiring examination at each inspection interval.

III. Proposed Alternate Method:

Rochester Gas and Electric believes as a minimum that the terminal connection welds of identified exempted welds (< 3/8" nominal wall) should be examined to the requirements of IWC 2500-1 Table, Category C-F-1 and C-F-2, Items c5.10 and c5.50 respectively. These examinations are identified in the Class 2 Allocation Tables as Augmented Examinations and also are included under the category C-F-1 and C-F-2. In the Program Plan Tables (Supplement 1 to Appendix B). These are identified as C-F-1 or C-F-2 followed by "----". These components are also noted in the instruction field.

RELIEF REQUEST NO. 16

I. Components for Which Relief is Requested:

Class 1 and Class 2 Integral Attachments on Piping specifically to Support Attachments.

II. ASME Requirements for Which Relief is Requested:

For Class 1, Integral Attachments on piping as indicated in IWB-2500-1, Category B-K-1, Item B10.10, requires volumetric or surface examination be performed on Base Attachment Thickness $\geq$ 5/8". For Class 2, Integral Attachments on piping as indicated in IWC-2500-1, Category C-C, Item C3.20, requires a surface examination be performed on Base Attachments $\geq$ 3/4".

It has been felt that support attachments to the pressure boundary such as gussets and stanchions should have a surface examination performed to insure the safety and integrity of the Class 1 and Class 2 Systems.

III. Proposed Alternate Method:

Surface examinations shall be performed on Integral Attachments on piping specifically support attachments once per interval in accordance with IWB-2500-1 and IWC-2500, B-K-1 and C-C, respectively.

QUALITY ASSURANCE MANUAL GINNA STATION ROCHESTER GAS & ELECTRIC CORPORATION		REV. 0	PAGE 1 OF 93A
	EFFECTIVE DATE: January 1, 1990		
		SIGNATURE	DATE
TITLE: ATTACHMENT A APPENDIX B GINNA NUCLEAR POWER STATION INSERVICE INSPECTION PROGRAM FOR THE 1990-1999 INTERVAL	PREPARED BY:	Michael J. Szymanski	7-20-89
	QUALITY ASSURANCE REVIEW	C R Anderson	7-20-89
	APPROVED BY:	John F. Smith	7/20/89

ATTACHMENT A

Line List Introduction

In the following Section; Class 1, 2 and 3 Line Lists have been identified and grouped into their associated section. P&ID drawings were reviewed in respect to their class and systematically identified with unique line numbers. Line numbers were then identified with required ASME Section XI Code Information and reviewed according to their exemption status.

Lines that were exempted have their corresponding code exemption stated. Lines not possessing code exemption have been identified with the type of examination required by the code. In all cases, the system and associated P&ID drawing number can be located in the upper left when viewing the Line List Tables.

The following line lists have been grouped by class for convenience. P&ID Drawings to support these line lists are located in Attachment B.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: CHEMICAL & VOLUME CONTROL
P&ID No: 33013-1265

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2-CHS-2501	RC-300;CVC-700	2-AS-1001	A-11	A376	2.00	0.344		SUR			Y	CVCS AUX SPRAY FROM 9313 TO PRZ SPRAY
2A-CHS-2501	CVC-700;-701	2-ACH-1002	A-27	A376	2.00	0.344		SUR			Y	CVCS ALT. CHARGING TO HOT LEG B
2B-CHS-2501			A-25	A376	2.00	0.344		SUR			Y	CVCS CHARGING TO COLD LEG B
2C-CHS-2501	CVC-400;-401	2-ACH-1001	A-26	A376	2.00	0.344		SUR			Y	CVCS ALT. CHARGING TO COLD LEG A
.75A-CHS-2501-A		N/A		A376	0.75	0.113	IWB-1220(b)				Y	RCP A TO VALVE 388A
.75A-CHS-2501-B		N/A		A376	0.75	0.113	IWB-1220(b)				Y	RCP B TO VALVE 388B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: CHEMICAL & VOLUME CONTROL
P&ID No: 33013-1264

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-CH6-2501	CVC-730	2-LD-1002	A-23	A376	2.00	0.344		SUR			Y	CVC LETDOWN FROM 427 TO RHE
2B-CH6-2501		2-LD-1003	A-24	A376	2.00	0.344		SUR			Y	RHE TO 200A, 200B, 202
.75A-CH6-2501		N/A		A376	0.75	0.113	IWB-1220(b)				Y	VALVE 310 TO ELHE
.75B-CH6-2501		N/A		A376	0.75	0.113	IWB-1220(b)				Y	ELHE TO VALVE 123

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: REACTOR COOLANT-LOOP A
P&ID No: 33013-1260

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
31-RC-2501-A		LINE A	A-3	CCSS	31.00	2.500		SUR/VOL			Y	CROSS OVER FROM SGA TO RCP A
29-RC-2501-A		LINE A	A-3	CCSS	29.00	2.500		SUR/VOL			Y	HOT LEG FROM RPV TO SG A
27.5-RC-2501-A		LINE A	A-3	CCSS	27.50	2.400		SUR/VOL			Y	COLD LEG FROM RCP A TO RPV
10A-RCO-2501-A	RHR-2500	10-AC-1004	A-15	A376	10.00	1.000		SUR/VOL			Y	RH FROM HOT LEG A TO 700
6A-RCO-2501-A	RHR-100	6-AC-1003	A-14	A376	6.00	0.718		SUR/VOL			Y	RHR FROM VALVE 852A TO 853A
4A-RCO-2501-A	RHR-100	4-AC-1003	A-14	A376	4.00	0.531		SUR/VOL			Y	RHR FROM VALVE 853A TO RPV
2B-RCO-2501-A		2-LD-1001	A-22	A376	2.00	0.000		SUR			Y	DRAIN FROM CROSS OVER LEG TO VALVE 541
.75A-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	CROSS OVER LEG A INSTRUMENTATION
.75B-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	CROSS OVER LEG A INSTRUMENTATION
.75C-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	CROSS OVER LEG A INSTRUMENTATION
.75D-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	CROSS OVER LEG A INSTRUMENTATION
.75E-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	LETDOWN FROM CROSS OVER LEG
.75F-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	LETDOWN FROM CROSS OVER LEG
.75G-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	RPV TO VENT
.75H-RCO-2501-A		N/A		A376	0.75	0.000	IWB-1220(b)				Y	RPV TO VENT

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: REACTOR COOLANT-LOOP B
P&ID No: 33013-1260

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
31-RC-2501-B		LINE B	A-38	CCSS	31.00	2.500		SUR/VOL			Y	CROSS OVER FROM SG B TO RCP B
29-RC-2501-B		LINE B	A-38	CCSS	29.00	2.500		SUR/VOL			Y	HOT LEG FROM RPV TO SG B
27.5-RC-2501-B		LINE B	A-38	CCSS	27.50	2.400		SUR/VOL			Y	COLD LEG FROM RCP B TO RPV
6A-RCO-2501-B	RHR-100	6-AC-1002	A-18	A376	6.00	0.718		SUR/VOL			Y	RHR FROM VALVE 852B TO 853B
4A-RCO-2501-B	RHR-100	4-AC-1002	A-18	A376	4.00	0.531		SUR/VOL			Y	RHR FROM VALVE 853B TO RPV
2A-RCO-2501-B		2-DR-1002	A-23A	A376	2.00	0.344		SUR/SUR			Y	LETDOWN FROM CROSSOVER LEG TO VALVE 427
1A-RCO-2501-B		N/A		A376	1.00	0.133	IWB-1220(b)				Y	HOT LEG TO SAMPLING
.75A-RCO-2501-B		N/A		A376	0.75	0.113	IWB-1220(b)				Y	CROSS OVER LEG INSTRUMENTATION
.75B-RCO-2501-B		N/A		A376	0.75	0.113	IWB-1220(b)				Y	CROSS OVER LEG INSTRUMENTATION
.75C-RCO-2501-B		N/A		A376	0.75	0.113	IWB-1220(b)				Y	CROSS OVER LEG INSTRUMENTATION
.75D-RCO-2501-B		N/A		A376	0.75	0.113	IWB-1220(b)				Y	CROSS OVER LEG INSTRUMENTATION

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: REACTOR COOLANT-PRESSURIZER
P&ID No: 33013-1258

RGLE Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
10-RC8-2501	RC-200	10 PRZ SURGE	A-38	A376	10.00	1.000		SUR/VOL			Y	PRZ SURGE LINE
4A-RC8-2501-A		4-RC-273	A-13	A376	4.00	0.531		SUR/VOL			Y	PRZ RELIEF PRZ TO PCV 434
4A-RC8-2501-B		4-RC-273	A-13	A376	4.00	0.531		SUR/VOL			Y	PRZ RELIEF PRZ TO PCV 435
4C-RC8-2501		4-RC-1005	A-12	A376	4.00	0.531		SUR/VOL			Y	PRZ RELIEF PRZ.TO RELIEF MANIFOLD
3A-RC8-2501		3-RC-1005	A-12	A376	3.00	0.438		SUR			Y	PRZ RELIEF MANIFOLD TO VALVES 515 & 431C
3A-RC8-2501-A	RC-300	3-RC-1000	A-9	A376	3.00	0.438		SUR			Y	PRZ SPRAY FROM LOOPS TO 431A & B
3A-RC8-2501-B	RC-300	3-RC-1001	A-9	A376	3.00	0.438		SUR			Y	PRZ SPRAY FROM LOOPS TO 431A & B
3B-RC8-2501		3-RC-1006	A-12	A376	3.00	0.438		SUR			Y	PRZ RELIEF MANIFOLD TO VALVES 516 & 430
3C-RC8-2501	RC-300	3-RC-1000	A-9	A376	3.00	0.438		SUR			Y	PRZ SPRAY FROM 431A & B TO PRZ HEAD
.75A-RC8-2501		N/A		A376	0.75	0.219	IWB-1220(b)				Y	PRZ RELIEF TO VALVE 950
.75A-RC8-2501-A		N/A		A376	0.75	0.219	IWB-1220(b)				Y	PRZ SPRAY BY-PASS AT 518
.75A-RC8-2501-B		N/A		A376	0.75	0.219	IWB-1220(b)				Y	PRZ SPRAY BY-PASS AT 517
.75B-RC8-2501		N/A		A376	0.75	0.219	IWB-1220(b)				Y	SURGE SAMPLE AT VALVE 952

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: RESIDUAL HEAT REMOVAL
P&ID No: 33013-1247

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
10A-AC7-2501-A	RHR-2500	10-AC-1004	A-15	A376	10.00	1.000		SUR/VOL			Y	RH FROM VALVE 700 TO VALVE 701
10A-AC7-2501-B	SI-200	10-AC-1001	A-14	A376	10.00	1.000		SUR/VOL			Y	SI FROM VALVE 720 TO COLD LEG B

QUALITY ASSURANCE MANUAL
GINNA STATION

TITLE:
ATTACHMENT A
APPENDIX B
GINNA NUCLEAR POWER STATION
INSERVICE INSPECTION PROGRAM
FOR THE 1990-1999 INTERVAL

REV. 0

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 1
System: SAFETY INJECTION
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
10A-SI2-2501-A	SI-200	10-SI-1004	A-16	A376	10.00	1.000		SUR/VOL			Y	SI FROM ACCUM TO COLD LEG B
10A-SI2-2501-B	SI-100	10-SI-1005	A-17	A376	10.00	1.000		SUR/VOL			Y	SI FROM ACCUM TO COLD LEG A
2A-SI2-2501	SI-110	2-SI-1001	A-19	A376	2.00	0.344		SUR			Y	878J B TO 10" SI ACCUM DUMP
2B-SI2-2501	SI-210	2-SI-1002	A-21	A376	2.00	0.344		SUR			Y	SI PUMP A TO HOT LEG B
2C-SI2-2501	SI-110,-111	2-SI-1001	A-20	A376	2.00	0.344		SUR			Y	SI PUMP B TO HOT LEG A
2D-SI2-2501	SI-210	2-SI-1002	A-16	A376	2.00	0.344		SUR			Y	SI PUMP A TO 10" SI ACCUM DUMP
.75A-SI2-2501-A		N/A		A376	0.75	0.113	IWB-1220(b)				Y	ACCUM A TO VALVE 839B
.75A-SI2-2501-B		N/A		A376	0.75	0.113	IWB-1220(b)				Y	ACCUM B TO VALVE 840B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: AUXILIARY COOLING SPENT FUEL
P&ID No: 33013-1248

RGLE Line No.	GILBERT Line No.	SURI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2I-CN-601	CVC-900		B-35	A312	2.00	0.154	IWC-1222(A)		250	350	Y	THIS LINE ACCOUNTED FOR ON 33013-1264.
2J-CN-601	CVC-900		B-35	A312	2.00	0.154	IWC-1222(A)		250	350	Y	THIS LINE ACCOUNTED FOR ON 33013-1264.

QUALITY ASSURANCE MANUAL
GINNA STATION

TITLE: ATTACHMENT A
APPENDIX B
GINNA NUCLEAR POWER STATION
INSERVICE INSPECTION PROGRAM
FOR THE 1990-1999 INTERVAL

REV. 0

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: AUXILIARY COOLANT
P&ID No: 33013-1246

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
6A-AC-152	CC-200			A53	6.00	0.280	IWC-1222(C)		90	70	N	VALVE 813 TO PEN 131.
6B-AC-152	CC-220			A53	6.00	0.280	IWC-1222(C)		90	70	N	PEN 130 TO VALVE 814.
4A-AC-152	CC-450		B-29	A53	4.00	0.237	IWC-1222(A)		90	70	N	VALVE 750B IN CONT TO RDCR PEN 128.
4B-AC-152	CC-625		B-30	A53	4.00	0.237	IWC-1222(A)		90	70	N	VALVE 750A TO RDCR BEFORE PEN 127 IN CON
3A-AC-152	CC-220			A58	3.00	0.216	IWC-1222(A)		90	70	N	PEN 125 TO VALVE 759B.
3B-AC-152	CC-220			A58	3.00	0.216	IWC-1222(A)		90	70	N	PEN 126 TO VALVE 759A.
3C-AC-152	CC-330 & CC-450		B-29	A53	3.00	0.216	IWC-1222(A)		90	70	N	RDCR BEFORE PEN 128 IN CONT TO VAL 749B.
3D-AC-152	CC-625 & CC-330		B-30	A53	3.00	0.216	IWC-1222(A)		90	70	N	RDCR BEFORE PEN 127 IN CONT TO VAL 749A.
2A-AC-152				A53	2.00	0.154	IWC-1222(A)		90	70	N	PEN 124 TO VALVE 743A.
2B-AC-152	CC-220			A53	2.00	0.154	IWC-1222(A)		90	70	N	PEN 124 TO VALVE 745.
.75A-AC-152	CC-300			A53	0.75	0.113	IWC-1222(A)		90	70	N	3C-AC-152 TO PIPE CAP. (VALVE 2741).
.75B-AC-152	CC-300			A53	0.75	0.113	IWC-1222(A)		90	70	N	3D-AC-152 TO PIPE CAP. (VALVE 2621).
.75C-AC-152	CC-200			A53	0.75	0.113	IWC-1222(A)		90	70	N	6A-AC-152 TO PIPE CAP. (VALVE 2724).
.75D-AC-152	CC-220			A53	0.75	0.113	IWC-1222(A)		90	70	N	6B-AC-152 TO PIPE CAP. (VALVE 2726).

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: AUXILIARY FEEDWATER
P&ID No: 33013-1237

RG&E Line No.	GILBERT Line No.	SW&I Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3A-FW-900-1A	AFW-500	FW-1001	B-11	A106	3.00	0.300	IWC-1222(A)		715	505	Y	THIS LINE ACCOUNTED FOR ON 33013-1236.
3A-FW-900-1B	AFW-400	FW-1005	B-13	A106	3.00	0.300	IWC-1222(A)		715	505	Y	THIS LINE ACCOUNTED FOR ON 33013-1236.
3B-FW-900-1A	AFW-500	FW-1001	B-11	A106	3.00	0.300	IWC-1222(A)		715	505	Y	THIS LINE ACCOUNTED FOR ON 33013-1236.
3B-FW-900-1B	AFW-400	FW-1005	B-13	A106	3.00	0.300	IWC-1222(A)		715	505	Y	THIS LINE ACCOUNTED FOR ON 33013-1236.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CONTAINMENT SPRAY
P&ID No: 33013-1261

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
4C-SI-301	CS-150			A312	4.00	0.237	IWC-1221(F)		205	70	N	3A-SI-301 TO 6I-SI-301.
4D-SI-301	CS-800		B-47		4.00	0.000	IWC-1221(A)		205	70	-	CS PUMP TO 6J-SI-301
4E-SI-301	CS-250			A312	4.00	0.237	IWC-1221(F)		205	70	N	6X4 RDCR ON 3B-SI-301.
4F-SI-301	CS-250			A312	4.00	0.237	IWC-1221(F)		205	70	N	4X3 RDCR TO 6X4 RDCR ON 6P-SI-301.
3-SI-151					3.00	0.000	IWC-1221(E)		30	70	-	RUST TO FIRST FLANGE.
3A-SI-301	CS-150			A312	3.00	0.216	IWC-1221(F)		205	70	N	4B-SI-301 TO 4C-SI-301.
3B-SI-301	CS-250			A312	3.00	0.216	IWC-1221(F)		205	70	N	4X3 RDCR ON 4E-SI-301 TO 4X3 RDCR.
3C-SI-301					3.00	0.000	IWC-1221(A)				-	VALVE 863B TO SPRAY ADDITIVE TANK.
2-SI-151					2.00	0.154	IWC-1221(E)		30	70	-	RUST TO VALVE 893A.
2A-SI-151					2.00	0.154	IWC-1221(E)		30	70	-	RUST TO VALVE 893B.
2B-SI-151					2.00	0.154	IWC-1221(E)		30	70	-	RUST TO VALVE 894B.
2C-SI-151					2.00	0.154	IWC-1221(E)		30	70	-	RUST TO VALVE 808.
2D-SI-151	CS-520			A312	2.00	0.154	IWC-1221(A)		30	70	-	10-SI-151 TO VALVE 873C.
2E-SI-301	CS-100			A312	2.00	0.154	IWC-1221(F)		205	70	N	6G-SI-301 TO 6H-SI-301.
2F-SI-301	CS-100			A312	2.00	0.154	IWC-1221(A)		205	70	N	2E-SI-301 TO VALVE 875A.
2G-SI-301	CS-100			A312	2.00	0.154	IWC-1221(A)		205	70	N	2E-SI-301 TO VALVE 875B.
2H-SI-301	CS-100			A312	2.00	0.154	IWC-1221(A)		205	70	N	2E-SI-301 TO VALVE 876B.
2I-SI-301	CS-100			A312	2.00	0.154	IWC-1221(A)		205	70	N	2E-SI-301 TO VALVE 876A.
2J-SI-151	CS-510		B-19	A312	2.00	0.154	IWC-1221(A)		205	70	-	8B-SI-151 TO CS EDUCTOR
2K-SI-301	CS-510			A312	2.00	0.154	IWC-1221(A)		205	70	-	6C-SI-301 TO VALVE 859A.
2KK-SI-301	CS-510			A312	2.00	0.154	IWC-1221(A)		205	70	-	CS EDUCTORA TO 2K-SI-301
2L-SI-151	CS-510			A312	2.00	0.154	IWC-1221(A)		205	70	-	CS EDUCTOR 1 TO CS EDUCTOR 2
2M-SI-301	CS-510		B-19	A312	2.00	0.154	IWC-1221(A)		205	70	-	8A-SI-151 TO CS EDUCTOR.
2N-SI-301	CS-510			A312	2.00	0.154	IWC-1221(A)		205	70	-	CS EDUCTOR 2 TO 2N-SI-301.
2P-SI-151	CS-510,CS-520			A312	2.00	0.154	IWC-1221(A)				-	2L-SI-151 TO VALVE 873B.
2Q-SI-301	CS-510			A312	2.00	0.154	IWC-1221(A)				-	6J-SI-301 TO VALVE 859B.
2QA-SI-151	CS-520			A312	2.00	0.154	IWC-1221(A)				-	2P-SI-151 BY VALVE 873B TO VALVE 873D.
2R-SI-151	CS-520			A312	2.00	0.154	IWC-1221(A)				-	2" TEE ON P-SI-151 PAST VALVE.
2S-SI-151	CS-520			A312	2.00	0.154	IWC-1221(A)				-	2P-SI-151 PAST VALVE 836B TO 1ST 2" TEE.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CONTAINMENT SPRAY
P&ID No: 33013-1261

RG&E Line No.	GILBERT Line No.	SURI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?.	Line Description/Remarks
10-SI-151	RHR-450	10-SI-2004	B-19	A312	10.00	0.165	IWC-1221(E)		30	70	-	RWST TO 1ST 10X8 & 10X6 RDCR.
108-AC-601					10.00	0.000					-	THIS LINE ACCOUNTED FOR ON 33013-1247.
8-SI-151	RHR-450		B-19	A312	8.00	0.148	IWC-1221(E)		30	70	-	ACCOUNTED IN 10-SI-151; ACTUALLY A TEE.
8A-SI-151	RHR-450		B-19	A312	8.00	0.148	IWC-1221(E)		30	70	-	6A-SI-151 TO CS PUMP.
8B-SI-151	RHR-450	10-SI-151R	B-19	A312	8.00	0.148	IWC-1221(E)		30	70	-	6B-SI-151 TO CS PUMP
8K-SI-151					8.00	0.000					-	THIS LINE ACCOUNTED FOR ON 33013-1247.
6A-SI-151	RHR-450	6-SI-151R	B-19	A312	6.00	0.148	IWC-1221(E)		30	70	-	10-SI-151 TO RDCR PAST VALVE 858B.
6B-SI-151	RHR-450	10-SI-151R	B-19	A312	6.00	0.148	IWC-1221(E)		30	70	-	10-SI-151 TO VALVE 858A
6C-SI-301	CS-500		B-46	A403	6.00	0.280		SUR/VOL	205	70	-	4A-SI-151 TO 1ST 6X6X6 TEE
6D-SI-301	CS-500		B-46	A403	6.00	0.280		SUR/VOL	205	70	-	6C-SI-301 TO VALVE 860A.
6DD-SI-301	CS-500		B-46	A403	6.00	0.280	IWC-1221(F)		205	70	-	VALVE 860A TO 1ST 6" TEE BEFORE V.862A.
6E-SI-301	CS-500		B-46	A403	6.00	0.280		SUR/VOL	205	70	-	1ST TEE ON LINE 6C-SI-301 TO VALVE 860B.
6EE-SI-301	CS-500		B-46	A403	6.00	0.280	IWC-1221(F)		205	70	-	VALVE 860B TO VALVE 862A.
6F-SI-301	CS-500		B-46	A403	6.00	0.280	IWC-1221(F)		205	70	-	TEE BEFORE VALVE 862A TO PEN 105.
6G-SI-301	CS-100,CS-150		B-48	A312	6.00	0.281	IWC-1221(F)		205	70	N	PEN 105 TO CS SPRAY RING.
6H-SI-301	CS-150			A312	6.00	0.281	IWC-1221(F)		205	70	N	6G-SI-301 TO 4B-SI-301.
6I-SI-301	CS-150			A312	6.00	0.281	IWC-1221(F)		205	70	N	4C-SI-301 TO 6G-SI-301.
6J-SI-301	CS-800		B-47	A403	6.00	0.280		SUR/VOL	205	70	-	4D-SI-301 TO 1ST 6X6X6 TEE
6K-SI-301	CS-800		B-47	A403	6.00	0.280		SUR/VOL	205	70	-	6X6X6 TEE ON 6J-SI-301 TO VALVE 860D.
6KK-SI-301	CS-800		B-47	A403	6.00	0.280	IWC-1221(F)		205	70	-	VALVE 860D TO 6" TEE BEFORE VALVE 862B.
6L-SI-301	CS-800		B-47	A403	6.00	0.280		SUR/VOL	205	70	-	6X6X6 TEE ON 6J-SI-301 TO VALVE 860C.
6LL-SI-301	CS-800		B-47	A403	6.00	0.280	IWC-1221(F)		205	70	-	VALVE 860C TO 6" TEE BEFORE VALVE 862B.
6M-SI-301	CS-800		B-47	A403	6.00	0.280	IWC-1221(F)		205	70	-	6X6X6 TEE BEFORE VALVE 862B TO PEN 109.
6N-SI-301	CS-200,CS-250			A312	6.00	0.280	IWC-1221(F)		205	70	N	PEN 109 TO 6" TEE ON LOWER CS RING.
6P-SI-301	CS-250			A312	6.00	0.281	IWC-1221(F)		205	70	N	4F-SI-301 TO 6N-SI-301.
6Q-SI-301	CS-250			A312	6.00	0.281	IWC-1221(F)		205	70	N	6N-SI-301 TO 4E-SI-301.
4-CH-151	CVC-1200			A312	4.00	0.120			30	70	-	THIS LINE ACCOUNTED FOR ON 33013-1265.
4A-SI-301	CS-500		B-46	A312	4.00	0.000	IWC-1221(A)		205	70	-	CS PUMP TO 6C-SI-301.
4B-SI-301	CS-150			A312	4.00	0.237	IWC-1221(F)		205	70	N	6H-SI-301 TO 3A-SI-301.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CONTAINMENT SPRAY
P&ID No: 33013-1261

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2I-SI-151	CS-520			A312	2.00	0.154	IWC-1221(A)				-	VALVE 881B TO SPRAY ADDITIVE TANK.
1B-SI-301	CS-800			A312	1.00	0.133	IWC-1221(A)		205	70	-	6" TEE ON 6H-SI-301 BY V 862B TO V 861.
1H-SI-1501					1.00	0.000					-	THIS LINE ACCOUNTED FOR ON 33013-1262.
.75A-SI-301	CS-500			A312	0.75	0.113	IWC-1221(A)		205	70	-	6F-SI-301 TO VALVE 864A.
.75B-SI-301	CS-500			A312	0.75	0.113	IWC-1221(A)		205	70	-	6F-SI-301 TO VALVE 869A.
.75C-SI-301				A312	0.75	0.113	IWC-1221(A)				-	3C-SI-301 TO VALVE 1802.
.75D-SI-151				A312	0.75	0.113	IWC-1221(A)		30	70	-	6A-SI-151 BY VALVE 858B TO VALVE 861.

42-52D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS
P&ID No: 33013-1266

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
8A-SI-301B	SI-400	8-SI-2001	B-15	A312	8.00	0.322	IWC-1221(E)		5	210	Y	BORIC ACID TANK B TO 8X8X8 TEE.
8B-SI-301A	SI-400	8-SI-2001	B-15	A312	8.00	0.322	IWC-1221(E)		5	210	Y	BORIC ACID TANK A TO 8X8X8 TEE.
3A-CH-151A	SI-400				3.00	0.000	IWC-1221(A)		5	210	Y	BORIC ACID TANK A TO LOOP SEAL.
3B-CH-151B	SI-400			A312	3.00	0.120	IWC-1221(A)		5	210	Y	BORIC ACID TANK B TO LOOP SEAL.
2A-SI-151B	SI-400			A312	2.00	0.154	IWC-1221(A)		5	210	Y	8X8X2 TEE ON 8A-SI-301B TO VALVE 345.
2B-CH-151B	SI-400			A312	2.00	0.154	IWC-1221(A)		5	210	Y	BORIC ACID TANK B TO HCV 105.
2C-SI-151A	SI-400			A312	2.00	0.154	IWC-1221(A)		5	210	Y	8X8X2 TEE ON 8B-SI-301A TO VALVE 331.
2D-CH-151					2.00	0.000	IWC-1221(A)		80	190	-	THIS LINE WILL CONT. ON 33013-1265.
2D-CH-151A	SI-400			A312	2.00	0.154	IWC-1221(A)		5	210	Y	BORIC ACID TANK A TO HCV 104.
2E-CH-151					2.00	0.000	IWC-1221(A)		80	200	-	THIS LINE ACCOUNTED FOR ON 33013-1265.
2F-CH-151					2.00	0.000	IWC-1221(A)		80	200	-	THIS LINE ACCOUNTED FOR ON 33013-1265.
1A-CH-151B	SI-400			A312	1.00	0.000	IWC-1221(A)		5	210	Y	BORIC ACID TANK TO VALVE 343.
1B-CH-151A	SI-400				1.00	0.000	IWC-1221(A)		5	210	Y	BORIC ACID TANK A TO VALVE 328.
1C-CH-151					1.00	0.000	IWC-1221(A)		80	190	-	THIS LINE ACCOUNTED FOR ON 33013-1265.
1D-CH-151					1.00	0.000	IWC-1221(A)		80	190	-	THIS LINE ACCOUNTED FOR ON 33013-1265.
.75A-CH-151B	SI-400				0.75	0.113	IWC-1221(A)		5	210	Y	2B-CH-151B TO VALVE 344.
.75B-CH-151A					0.75	0.083	IWC-1221(A)		5	210	Y	2D-CH-151A TO VALVE 2242.
.75C-CH-151A					0.75	0.083	IWC-1221(A)		5	210	Y	2D-CH-151A TO VALVE 329.
.75D-CH-151					0.75	0.000	IWC-1221(A)		80	190	-	2D-CH-151 CONTINUE ON 33013-1265.
.75E-SI-151B	SI-400			A312	0.75	0.113	IWC-1221(A)		5	210	Y	2A-SI-151B TO VALVE 346B.
.75F-SI-151A	SI-400			A312	0.75	0.113	IWC-1221(A)		5	210	Y	2C-SI-151A TO VALVE 346A.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS CHARGING
P&ID No: 33013-1265

RG&E Line No.	GILBERT Line No.	SURI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
8A-CH-2502	CVC-851	PULSE DAMPENER	B-6	A312	8.00	0.906		SUR/VOL	2450	100	Y	3" TEE ON 3K-CH-2502 TO PULSE DAMPENER.
8B-CH-2502	CVC-852	PULSE DAMPENER	B-6	A312	8.00	0.906		SUR/VOL	2450	100	Y	3" TEE ON 3L-CH-2502 TO PULSE DAMPENER.
8C-CH-2502	CVC-853	PULSE DAMPENER	B-6	A312	8.00	0.906		SUR/VOL	2450	100	Y	3" TEE ON 3M-CH-2502 TO PULSE DAMPENER.
4-CH-151	CVC-1200			A312	4.00	0.120	IWC-1222(A)		30	100	-	4B-CH-151 PAST VALVE 357 TO RWST.
4A-CH-151	CVC-1200			A312	4.00	0.120	IWC-1222(A)		30	100	-	VCT TO 1ST 4X4X3 TEE AFTER VALVE 266.
4B-CH-151	CVC-1200			A312	4.00	0.120	IWC-1222(A)		30	100	-	4" TEE ON 4A-CH-151 TO VALVE 268.
4C-CH-151	CVC-1200			A312	4.00	0.120	IWC-1222(A)		30	100	-	4" TEE ON 4B-CH-151 TO VALVE 399.
3A-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		30	100	Y	VCT TO 1ST 3X3X3 TEE BEFORE VALVE 394.
3AN-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		40	140	-	3" TEE ON 3E-CH-151 PAST VALVE 315C.
3B-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		30	100	Y	3" TEE ON 3A-CH-151 TO 1ST 3" TEE.
3C-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		30	100	Y	3" TEE ON 3A-CH-151 PAST VALVE 321..
3D-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		40	140	-	SEAL WATER HE PAST VALVE 265 TO 1ST TEE.
3E-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		40	140	-	3" TEE ON 3D-CH-151 TO VALVE 315B.
3F-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		40	140	-	3" TEE ON 3E-CH-151 TO SEAL WATER FILTER
3G-CH-151	CVC-1000			A312	3.00	0.120	IWC-1222(A)		40	140	-	SEAL WATER RETURN FILTER TO PEN 108.
3H-CH-151	CVC-1200			A312	3.00	0.120	IWC-1222(A)		40	140	Y	4" TEE ON 4A-CH-151 TO CHARGING PUMP 1.
3I-CH-151	CVC-1200			A312	3.00	0.120	IWC-1222(A)		2450	100	-	4" TEE ON 4B-CH-151 TO CHARGING PUMP 2.
3J-CH-151	CVC-1200			A312	3.00	0.120	IWC-1222(A)		30	100	Y	4" TEE ON 4C-CH-151 TO CHARGING PUMP 3.
3K-CH-2502	CVC-851			A312	3.00	0.437	IWC-1222(A)		2450	100	-	CHG PUMP 1 DISCHG PAST VALVE 287 TO TEE.
3L-CH-2502	CVC-852			A312	3.00	0.437	IWC-1222(A)		2450	100	-	CHG PUMP 2 DISCHG PAST VALVE 288 TO TEE.
3M-CH-2502	CVC-853			A312	3.00	0.437	IWC-1222(A)		2450	100	-	CHG PUMP 3 DISCHG PAST VALVE 291 TO TEE.
3M-CH-601					3.00	0.000	IWC-1222(A)		30	100	-	THIS LINE ACCOUNTED FOR ON 33013-1264.
3N-CH-2502	CVC-1000			A312	3.00	0.438	IWC-1222(A)		2450	100	Y	PULSE DAMPENER PAST VALVE 289 TO 1ST TEE
3P-CH-2502				A312	3.00	0.438	IWC-1222(A)		2450	100	Y	PULSE DAMPENER PAST VALVE 290 PUL DAMP.
3Q-CH-2502	CVC-1100			A312	3.00	0.438	IWC-1222(A)		2450	100	Y	2" TEE ON 3N-CH-2502 PAST VALVE 275.
3R-CH-151	CVC-200			A312	3.00	0.120	IWC-1222(A)		140	40	Y	PEN 108 TO 1ST 3X3X2 TEE.
3R-CH-2502	CVC-1100			A312	3.00	0.438	IWC-1222(A)		2450	100	Y	2" TEE ON 3Q-CH-2502 TO 1ST 2" RDCR TEE.
2A-CH-151				A312	2.00	0.109	IWC-1222(A)		30	100	-	VALVE 257 TO VOLUME CONTROL TANK.
2AA-CH-2502	CVC-600,CVC-601			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	PEN 106 PAST VALVES 304A & 304C PUMP 1A

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS CHARGING
P&ID No: 33013-1265

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE-Method	Po	Temp	In?	Line Description/Remarks
2AB-CH-2502	CVC-500,CVC-501		B-33	A312	2.00	0.344	IWC-1222(A)		2450	100	Y	PEN 110 PAST VALVES 304B & 304D PUMP 1B
2AV-CH-151	CVC-200			A312	2.00	0.109	IWC-1222(A)		140	40	Y	3R-CH-151 PAST VALVES 362A & 270A PMP 1A
2D-CH-151				A312	2.00	0.000	IWC-1222(A)		30	100	-	4A-CH-151 VALVE 368 TO VALVE FCV110B.
2E-CH-151	CVC-1200			A312	2.00	0.109	IWC-1222(A)		30	100	-	4C-CH-151 TO VALVE 350.
2F-CH-2502	CVC-800			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	PULSE DAMP PAST V 384A & 384B TO PEN 100
2G-CH-2502	CVC-800			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2F-CH-2502 PAST VALVE 384C BYPASS.
2H-CH-2502	CVC-1100			A312	2.00	0.343	IWC-1222(A)		2450	100	Y	3Q-CH-2502 PAST VALVE 323 TO PEN 102.
2I-CH-2502	CVC-1100			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2" RDCR TEE ON 3R-CH-2502 TO PEN 110.
2J-CH-2502	CVC-1100			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2" RDCR TEE 3R-CH-2502 PAST VALVE 300A.
2K-CH-2502	CVC-1100			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2" TEE ON 3N-CH-2502 PAST VALVE 303D.
2L-CH-2502	CVC-1100			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2K-CH-2502 PAST VALVE 303B TO SW FIL 2.
2M-CH-2502	CVC-1100			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2" TEE ON 3Q-CH-2502 PAST VALVE 303C.
2N-CH-2502	CVC-1100			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	2M-CH-2502 PAST VALVE 303A TO SW FIL 2.
2P-CH-2502	CVC-300			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	PEN 100 TO REGEN HEAT EXCHANGER.
2Q-CH-2501	CVC-700			A376	2.00	0.344	IWC-1222(A)		2300	525	Y	REGENERATIVE H EX TO VALVE 9314..
2R-CH-2501	CVC-700			A376	2.00	0.344	IWC-1222(A)		2300	525	Y	2Q-CH-2501 TO VALVE 9313.
2S-CH-2501	CVC-700			A376	2.00	0.344	IWC-1222(A)		2300	525	Y	2Q-CH-2501 TO VALVE 9315.
2T-CH-151	CVC-200			A312	2.00	0.109	IWC-1222(A)		140	40	Y	3R-CH-151 TO VALVE 314.
2U-CH-151	CVC-1110		B-34	A312	2.00	0.109	IWC-1222(A)		30	100	-	THIS LINE ACCOUNTED FOR ON 33013-1264.
2V-CH-151	CVC-200			A312	2.00	0.109	IWC-1222(A)		140	40	Y	2U-CH-151 TO 2X.75X.75 TEE.
2W-CH-151	CVC-200			A312	2.00	0.109	IWC-1222(A)		140	40	Y	3" TEE ON 3R-CH-151 TO VALVE 362B.
2X-CH-2501	CVC-700			A376	2.00	0.344	IWC-1222(A)		140	40	Y	VALVE 362B PAST VALVE 270B TO RC PMP 1B
2Z-CH-2502	CVC-400,CVC-402			A312	2.00	0.344	IWC-1222(A)		2450	100	Y	PEN 102 TO VALVE 392B.
1A-CH-151				A312	1.00	0.000	IWC-1222(A)		30	100	-	2A-CH-151 TO TEE BEFORE VAL 205A & 975.
1B-CH-151				A312	1.00	0.000	IWC-1222(A)		30	100	-	4A-CH-151 TO VALVE 264A.
1C-CH-151				A312	1.00	0.000	IWC-1222(A)		30	100	-	1B-CH-151 TO VALVE 264B.
1D-CH-151				A312	1.00	0.000	IWC-1222(A)		30	100	-	4A-CH-151 PAST VALVE 271 TO VALVE 272.
1E-CH-151				A312	1.00	0.000	IWC-1222(A)		30	100	-	4A-CH-151 TO VALVE 356.
1F-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	3J-CH-151 PAST VALVES 274 & 279C.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS CHARGING
P&ID No: 33013-1265

RGIE Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
1G-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	1F-CH-151 TO VALVE 283.
1H-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	3J-CH-151 TO VALVE 280C.
1I-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	1H-CH-151 TO VALVE 2249.
1J-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		2450	100	-	CHARGING PUMP 3 TO VALVE 280F.
1K-CH-151				A312	1.00	0.000	IWC-1222(A)		2450	100	-	DRAIN OF STUFF BOX LEAKOFF OF CHG PMP 3
1L-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	3I-CH-151 PAST VALVES 276 & 279B.
1M-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	1L-CH-151 TO VALVE 284.
1N-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	3I-CH-151 TO VALVE 280B.
1P-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	1N-CH-151 TO VALVE 2248.
1Q-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		2450	100	-	CHARGING PUMP 2 TO VALVE 280E.
1R-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		2450	100	-	DRAIN STUFFING BOX LEAKOFF TO VALVE 2245
1S-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	3H-CH-151 PAST VAL 278 & 278A TO 1ST TEE
1T-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	1S-CH-151 TO VALVE 285.
1U-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	3H-CH-151 TO VALVE 280A.
1V-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		30	100	-	1U-CH-151 TO VALVE 2247.
1W-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		2450	100	-	CHARGING PUMP 1 TO VALVE 280D.
1X-CH-151	CVC-1200			A312	1.00	0.000	IWC-1222(A)		2450	100	-	DRAIN STUFFING BOX LEAKOFF TO VALVE 2244
1Y-CH-151	CVC-200			A312	1.00	0.000	IWC-1222(A)		140	40	Y	.75X1 RDCR ON .75Z-CH-151 TO NEXT RDCR.
.75A-CH-151					0.75	0.000	IWC-1222(A)		30	100	-	2A-CH-151 TO VALVE 1275D.
.75AA-CH-2501	CVC-700			A312	0.75	0.218	IWC-1222(A)		140	40	Y	1X.75 RDCR ON 1Y-CH-151 TO #2 SEAL BYPAS
.75AB-CH-2501	CVC-700			A312	0.75	0.218	IWC-1222(A)		140	40	Y	.75AA-CH-2501 PAST VALVE 320A.
.75B-CH-151					0.75	0.000	IWC-1222(A)		30	100	-	TEE ON 1A-CH-151 TO VALVE PCV141
.75C-CH-151					0.75	0.000	IWC-1222(A)		30	100	-	SEAL WATER HEAT EXCH TO VALVE 282A.
.75D-CH-151					0.75	0.000	IWC-1222(A)		30	100	-	SEAL WATER HEAT EXCH TO VALVE 282B.
.75E-CH-151					0.75	0.000	IWC-1222(A)		40	140	-	SEAL WATER HEAT EXCH TO VALVE 2719.
.75F-CH-151					0.75	0.000	IWC-1222(A)		40	140	-	SEAL WATER HEAT EXCH TO VALVE 282C.
.75G-CH-151					0.75	0.000	IWC-1222(A)		40	140	-	SEAL WATER HEAT EXCH TO VALVE 2717.
.75H-CH-151	CVC-1000				0.75	0.000	IWC-1222(A)		40	140	-	SEAL WATER RETURN FILTER TO VALVE 319B.
.75I-CH-151	CVC-1000				0.75	0.000	IWC-1222(A)		40	140	-	SEAL WATER RETURN FILTER TO VALVE 319A.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS CHARGING
P&ID No: 33013-1265

ASME Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75J-CH-151					0.75	0.000	IWC-1222(A)		30	100	-	3H-CH-601 TO 1ST TEE BEFORE VALVE 279A.
.75K-CH-2502	CVC-851			A312	0.75	0.000	IWC-1222(A)		2450	100	-	3K-CH-2502 TO VALVE 292C.
.75L-CH-2502	CVC-851			A312	0.75	0.000	IWC-1222(A)		2450	100	-	3K-CH-2502 TO VALVE 285 ON 1T-CH-151.
.75M-CH-2502	CVC-852			A312	0.75	0.000	IWC-1222(A)		2450	100	-	3L-CH-2502 TO VALVE 292D.
.75N-CH-2502	CVC-852			A312	0.75	0.000	IWC-1222(A)		2450	100	-	3L-CH-2502 TO VALVE 284 ON 1H-CH-151.
.75P-CH-2502	CVC-853			A312	0.75	0.154	IWC-1222(A)		2450	100	-	3M-CH-2502 TO VALVE 292E.
.75Q-CH-2502	CVC-853			A312	0.75	0.154	IWC-1222(A)		2450	100	-	3M-CH-2502 TO VALVE 283 ON 1G-CH-151.
.75R-CH-2502	CVC-851		B-6	A312	0.75	0.154	IWC-1222(A)		2450	100	Y	8A-CH-2502 TO VALVE 281C.
.75S-CH-2502	CVC-852		B-6	A312	0.75	0.154	IWC-1222(A)		2450	100	Y	8B-CH-2502 TO VALVE 281G.
.75T-CH-2502	CVC-853		B-6	A312	0.75	0.154	IWC-1222(A)		2450	100	Y	8C-CH-2502 TO VALVE 281E.
.75U-CH-2502	CVC-1100			A312	0.75	0.154	IWC-1222(A)		2450	100	Y	2I-CH-2502 TO VALVE 301A.
.75V-CH-2502	CVC-1100			A312	0.75	0.154	IWC-1222(A)		2450	100	Y	2I-CH-2502 TO VALVE 301B.
.75W-CH-2502	CVC-1100			A312	0.75	0.154	IWC-1222(A)		2450	100	Y	2J-CH-2502 TO VALVE 298A.
.75X-CH-2502	CVC-1100			A312	0.75	0.154	IWC-1222(A)		2450	100	Y	2J-CH-2502 TO VALVE 298B.
.75Y-CH-2501	CVC-700			A376	0.75	0.219	IWC-1222(A)		2300	525	Y	2Q-CH-2501 TO VALVE 311F.
.75Z-CH-151	CVC-200			A312	0.75	0.093	IWC-1222(A)		140	40	Y	3R-CH-151 TO .75X1 RDCR BEFORE VALVE 386
.375A-CH-151					0.37	0.000	IWC-1222(A)		30	100	-	3H-CH-601 PAST VALVE 969A TO 5A-SL-151.
.375B-CH-151					0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 3 TO DRAIN FH STUFF BOX LEAKOFF
.375C-CH-151					0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 3 TO DRAIN FH STUFF BOX LEAKOFF
.375D-CH-151					0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 3 TO DRAIN FH STUFF BOX LEAKOFF
.375E-CH-151	CVC-1200			A312	0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 2 TO DRAIN FH STUFF BOX LEAKOFF
.375F-CH-151	CVC-1200			A312	0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 2 TO DRAIN FH STUFF BOX LEAKOFF
.375F-SL-2505					0.37	0.000	IWC-1222(A)		30	100	-	TEE ON 1A-CH-151 TO VALVE 977.
.375G-CH-151	CVC-1200			A312	0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 2 TO DRAIN FH STUFF BOX LEAKOFF
.375H-CH-151	CVC-1200			A312	0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 1 TO DRAIN FH STUFF BOX LEAKOFF
.375I-CH-151	CVC-1200			A312	0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 1 TO DRAIN FH STUFF BOX LEAKOFF
.375J-CH-151	CVC-1200			A312	0.37	0.000	IWC-1222(A)		2450	100	-	CHG PUMP 1 TO DRAIN FH STUFF BOX LEAKOFF
.25A-CH-151					0.25	0.000	IWC-1222(A)		2450	100	-	1K-CH-151 TO VALVE 2273.
.25B-CH-151				A312	0.25	0.000	IWC-1222(A)		2450	100	-	1R-CH-151 TO VALVE 2273 SAMPLE.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS CHARGING
P&ID No: 33013-1265

RGIE Line No.	GILBERT Line No.	SURI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.25C-CH-151	CVC-1200			A312	0.25	0.000	IWC-1222(A)		2450	100	-	1X-CH-151 TO VALVE 2271 SAMPLE.
.75RR-CH-2501	CVC-700				0.00	0.000	IWC-1222(A)		140	40	-	INFO CONCERNING LINE ON 33013-1264.

QUALITY ASSURANCE MANUAL
GINNA STATION

TITLE:
ATTACHMENT A
APPENDIX B
GINNA NUCLEAR POWER STATION
INSERVICE INSPECTION PROGRAM
FOR THE 1990-1999 INTERVAL

REV. 0

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS LETDOWN
P&ID No: 33013-1264

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3H-CH-601	CVC-1000		B-36	A312	3.00	0.120	IWC-1222(A)		250	100	Y	VCT TO 2K-CH-601 BY VALVE 204E.
2-CH-151	CVC-1110			A312	2.00	0.109	IWC-1222(A)		20	100	-	2T-CH-151 TO 10A-AC-601 (33013-1247).
2A-CH-601	CVC-105			A312	2.00	0.154	IWC-1222(A)		250	350	Y	VALVE 200A TO 1ST 2X2X2 TEE.
2B-CH-601	CVC-105			A312	2.00	0.154	IWC-1222(A)		250	350	Y	VALVE 200B TO 1ST 2" TEE ON 2A-CH-601.
2C-CH-601	CVC-105			A312	2.00	0.154	IWC-1222(A)		250	350	Y	2" TEE ON 2A-CH-601 TO 702 & 703.
2D-CH-601	CVC-105			A312	2.00	0.154	IWC-1222(A)		250	350	Y	VALVE 202 TO LINE 2C-CH-601.
2E-CH-601	CVC-105		B-32	A312	2.00	0.154	IWC-1222(A)		250	350	Y	2C-CH-601 TO VALVE 203.
2F-CH-601	CVC-100			A312	2.00	0.154	IWC-1222(A)		250	350	Y	10-AC-601 TO 2" TEE PAST VALVE HCV-133.
2G-CH-601	CVC-100		B-32	A312	2.00	0.154	IWC-1222(A)		250	350	Y	2" TEE ON 2C-CH-601 TO PEN 112.
2H-CH-601	CVC-900		B-35	A312	2.00	0.154	IWC-1222(A)		250	350	Y	PEN 112 TO NON-REGEN. HEAT EXCHG.
2I-CH-601	CVC-900		B-35	A312	2.00	0.154	IWC-1222(A)		250	350	Y	2H-CH-601 TO VALVE 821.
2J-CH-601	CVC-900		B-35	A312	2.00	0.154	IWC-1222(A)		250	350	Y	2H-CH-601 TO VALVE 820.
2K-CH-601	CVC-1000		B-36	A312	2.00	0.145	IWC-1222(A)		250	100	Y	DISCHARGE ON NONREGEN HE TO VALVE TVC145
2L-CH-601	CVC-1000		B-36	A312	2.00	0.145	IWC-1222(A)		250	100	Y	2K-CH-601 TO 2" TEE ON 2H-CH-601.
2N-CH-151	CVC-1000		B-36	A312	2.00	0.109	IWC-1222(A)		70	100	Y	VALVE TCV145 TO 2" TEE NEAR VALVE 2237.
2P-CH-151	CVC-1000		B-36	A312	2.00	0.109	IWC-1222(A)		70	100	Y	2" TEE ON 2N-CH-151 TO 1ST 2" TEE.
2Q-CH-151	CVC-1000		B-36	A312	2.00	0.109	IWC-1222(A)		40	100	Y	1ST 2" TEE ON 2P-CH-151 TO VALVE 2275
2R-CH-151	CVC-1110		B-36	A312	2.00	0.109	IWC-1222(A)		40	100	Y	1ST 2" TEE ON 2P-CH-151 TO RC FILTER.
2S-CH-151	CVC-1110			A312	2.00	0.109	IWC-1222(A)		40	100	-	2" TEE ON 2R-CH-151 TO 2T-CH-151.
2T-CH-151	CVC-1110			A312	2.00	0.109	IWC-1222(A)		20	100	-	RC FILTER TO VALVE LCV112A.
2U-CH-151	CVC-1110			A312	2.00	0.109	IWC-1222(A)		20	100	-	VALVE LCV112A PAST VALVE 256 TO VCT.
2V-CH-151	CVC-1110			A312	2.00	0.109	IWC-1222(A)		20	100	-	2U-CH-151 TO VALVE FCV110C.
2W-CH-151	CVC-1110			A312	2.00	0.109	IWC-1222(A)		40	100	-	2" TEE ON 2P-CH-151 TO VALVE 2398.
.75A-CH-601	CVC-105			A312	0.75	0.113	IWC-1222(A)		250	350	Y	2A-CH-601 TO VALVE 2200.
.75B-CH-601	CVC-105			A312	0.75	0.113	IWC-1222(A)		250	350	Y	2B-CH-601 TO VALVE 2201.
.75C-CH-601	CVC-105			A312	0.75	0.113	IWC-1222(A)		250	350	Y	2D-CH-601 TO VALVE 2202.
.75D-CH-601	CVC-900		B-35	A312	0.75	0.113	IWC-1222(A)		250	350	Y	2H-CH-601 TO VALVE 2231.
.75E-CH-601	CVC-900		B-35	A312	0.75	0.113	IWC-1222(A)		250	350	Y	2H-CH-601 TO VALVE 369.
.75F-CH-601	CVC-900		B-35	A312	0.75	0.113	IWC-1222(A)		250	350	Y	BYPASS 2H-CH-601 PAST V 2232 TO V 2233.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: CVCS LETDOWN
P&ID No: 33013-1264

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75G-CH-601	CVC-1000		B-36	A312	0.75	0.113	IWC-1222(A)		250	100	Y	.75F-CH-601 TO V 2233 COMPLETES BYPASS.
.75H-CH-601	CVC-1000		B-36	A312	0.75	0.113	IWC-1222(A)		250	100	Y	2K-CH-601 TO VALVE 207A.
.75I-CH-601	CVC-1000		B-36	A312	0.75	0.113	IWC-1222(A)		250	100	Y	2K-CH-601 TO VALVE 2234.
.75J-CH-151	CVC-1000			A312	0.75	0.083	IWC-1222(A)		70	100	-	2" TEE ON 2N-CH-151 TO VALVE 2237.
.75K-CH-151	CVC-1110			A312	0.75	0.083	IWC-1222(A)		40	100	-	2R-CH-151 TO VALVE 248A.
.75L-CH-151	CVC-1110			A312	0.75	0.083	IWC-1222(A)		40	100	-	REACTOR COOLANT FILTER TO VALVE 251.
.75M-CH-151	CVC-1110			A312	0.75	0.083	IWC-1222(A)		40	100	-	REACTOR COOLANT FILTER TO VALVE 389.
.75P-CH-151	CVC-1110			A312	0.75	0.083	IWC-1222(A)		20	100	-	2T-CH-151 TO VALVE 248B.
.75Q-CH-2501				A312	0.75	0.219	IWC-1222(A)		35		-	VALVE HCV123 TO VALVE 312.
.75R-CH-2501					0.75	0.000	IWC-1222(A)		35		-	VALVE 312 TO TEE ON 2V-CH-150.
.75S-CH-601	CVC-1000		B-36		0.75	0.113	IWC-1222(A)		250	100	-	2K-CH-601 BYPASS ON VALVES 207C & 207B.
.75T-CH-601					0.75	0.113	IWC-1222(A)		250	100	-	2F-CH-601 PAST VALVES 703 & 702.
.375D-SL-2505					0.37	0.000	IWC-1222(A)		40	100	-	2R-CH-151 TO VALVE 989.
.375E-SL-2505				A312	0.37	0.083	IWC-1222(A)		250	100	-	2K-CH-601 TO VALVE 988.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: FEEDWATER (SG-1A)
P&ID No: 33013-1236

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
14A-FW-900-1A	FW-100	FW-1001	B-11	A106	14.00	0.938		SUR/VOL	715	505	Y	VALVE 3993 TO S/G-1A.
3A-FW-900-1A	AFW-500	FW-1001	B-11	A106	3.00	0.300	IWC-1222(A)		715	505	Y	14" FW PAST VALVE 4011 TO VALVE 4000C
3B-FW-900-1A	AFW-500	FW-1001	B-11	A106	3.00	0.300	IWC-1222(A)		715	505	Y	14" FW PAST VALVE 4005 TO VALVE 4003.
3C-FW-900-1A	AFW-500	FW-1001	B-12	A106	3.00	0.300	IWC-1222(A)		715	505	Y	14" FW TO VALVE 9704A.
1A-FW-900-1A	FW-301	FW-1001	B-11	A106	1.00	0.179	IWC-1222(A)		715	505	Y	BYPASS LINE ON VALVE 3995A.
1B-FW-900-1A	FW-301	FW-1001	B-11	A106	1.00	0.179	IWC-1222(A)		715	505	Y	14" FW PAST VALVE 4099E.
1C-FW-900-1A	AFW-500	FW-1001	B-12	A106	1.00	0.179	IWC-1222(A)		715	505	Y	14-FW-900-1A PAST VALVE 3414J TO END CAP
.75A-FW-900-1A	FW-301	FW-1001	B-11	A106	0.75	0.154	IWC-1222(A)		715	505	Y	14" FW TO VALVE 8651.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: FEEDWATER (SG-1B)
P&ID No: 33013-1236

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
14B-FW-900-1B	FW-200	FW-1005	B-13	A106	14.00	0.938		SUR/VOL	715	505	Y	VALVE 3992 TO S/G-1B.
3A-FW-900-1B	AFW-400	FW-1005	B-14	A106	3.00	0.300	IWC-1222(A)		715	505	Y	14" FW PAST VALVE 4012 TO VALVE 4000D
3B-FW-900-1B	AFW-400	FW-1005	B-14	A106	3.00	0.300	IWC-1222(A)		715	505	Y	14" FW PAST VALVE 4006 TO VALVE 4004.
3C-FW-900-1B	AFW-400	FW-1005	B-13	A106	3.00	0.300	IWC-1222(A)		715	505	Y	14" FW TO VALVE 9704B.
1A-FW-900-1B	FW-300	FW-1005	B-14	A106	1.00	0.179	IWC-1222(A)		715	505	Y	14" FW TO VALVE 8650.
1B-FW-900-1B	FW-300	FW-1005	B-14	A106	1.00	0.179	IWC-1222(A)		715	505	Y	BYPASS ON VALVE 3994A.
1C-FW-900-1B	FW-300	FW-1005	B-14	A106	1.00	0.179	IWC-1222(A)		715	505	Y	14" FW TO VALVE 3994E/4099F.
1D-FW-900-1B	AFW-400	FW-1005	B-13	A106	1.00	0.179	IWC-1222(A)		715	505	Y	14-FW-900-1B PAST VALVE 3415J TO END CAP

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: FIRE PROTECTION IN CONTAINMENT
P&ID No: 33013-1241

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
4A-FS-152					4.00	0.000	IWC-1222(A)		135	80	-	VALVE 9227 TO VALVE 9229 IN CONTAINMENT.
1A-FS-152					1.00	0.000	IWC-1222(A)		135	80	-	4A-FS-152 IN CONTAINMENT TO VALVE 9228.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: HVAC
P&ID No: 33013-1865

RG&E Line No.	GILBERT Line No.	SURI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
6A-HPS-2000					6.00	0.000	IWC-1222(C)		125	80	-	V 7445 OUT CONT PAST PEN 309 TO V 7478.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: HVAC
P&ID No: 33013-1870

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
8A-CFA					8.00	0.000	IWC-1222(C)		.5	110	-	FILTER INLET IN CONT TO 8X6 RDCR.
6A-CFA					6.00	0.000	IWC-1222(C)		.5	110	-	8X6 RDCR 8A-CFA PAST PEN 132 TO V AAD04

42-52D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: MAIN STEAM STEAM GENERATOR 1A
P&ID No: 33013-1231

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
30A-MS-600-1A	MS-100	MS-1000	B-8	A115	30.00	1.250		SUR/VOL	715	505	Y	S/G 1A TO VALVE 3517.
6A-MS-600-1A	MS-100	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3505A.
6B-MS-600-1A	MS-300	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3411.
6C-MS-600-1A	MS-300	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3509.
6D-MS-600-1A	MS-300	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3511.
6E-MS-600-1A	MS-300	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3513.
6F-MS-600-1A	MS-300	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3515.
6G-MS-600-1A	MS-300	MS-1000	B-9	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3521.
3A-MS-600-1A	MS-300	MS-1000	B-9	A106	3.00	0.300	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3615.
1.5A-MS-600-1A	MS-300	MS-1000	B-9	A106	1.50	0.145	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3455.
1.5B-MS-600-1A	MS-300	MS-1000	B-9	A106	1.50	0.147	IWC-1222(A)		715	505	Y	6A-MS-600-1A TO VALVE 3505C.
1.5C-MS-600-1A	MS-300	MS-1000	B-9	A106	1.50	0.147	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3413A.
1.5D-MS-600-1A	MS-100	MS-1000	B-8	A106	1.50	0.145	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3503.
1.5E-MS-600-1A	MS-300	MS-1000	B-9	A106	1.50	0.145	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3453B.
1A-MS-600-1A	MS-300	MS-1000	B-9	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3669.
1B-MS-600-1A	MS-300	MS-1000	B-9	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3445B.
1C-MS-600-1A	MS-300	MS-1000	B-9	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3409B.
1D-MS-600-1A	MS-300	MS-1000	B-9	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3447B.
.75A-MS-600-1A	MS-300	MS-1000	B-9	A106	0.75	0.154	IWC-1222(A)		715	505	Y	BYPASS LINE ON VALVE 3411.
.75B-MS-600-1A	MS-100	MS-1000	B-8	A106	0.75	0.154	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3405.
.75C-MS-600-1A	MS-100	MS-1000	B-8	A106	0.75	0.154	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3407.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: MAIN STEAM STEAM GENERATOR 1B
P&ID No: 33013-1231

RGEE Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
30B-MS-600-1B	MS-200	SHS-1001	B-8	A115	30.00	1.250		SUR/VOL	715	505	Y	S/G 1B TO VALVE 3516.
6A-MS-600-1B	MS-300	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3504A.
6B-MS-600-1B	MS-120	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS-PIPE TO VALVE 3410.
6C-MS-600-1B	MS-300	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3508.
6D-MS-600-1B	MS-300	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3510.
6E-MS-600-1B	MS-300	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3512.
6F-MS-600-1B	MS-300	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3514.
6G-MS-600-1B	MS-300	SHS-1001	B-10	A106	6.00	0.432		SUR/VOL	715	505	Y	30" MS PIPE TO VALVE 3520.
3A-MS-600-1B	MS-300	SHS-1001	B-10	A106	3.00	0.300	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3614.
1.5A-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.50	0.145	IWC-1222(A)		715	505	Y	6A-MS-600-1B TO VALVE 3504C.
1.5B-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.50	0.147	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3412A.
1.5C-MS-600-1B	MS-200	SHS-1001	B-8	A106	1.50	0.147	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3500.
1.5D-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.50	0.147	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3456.
1A-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3668.
1B-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3448B.
1C-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3446B.
1D-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3408B.
1E-MS-600-1B	MS-300	SHS-1001	B-10	A106	1.00	0.179	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3454B.
.75A-MS-600-1B	MS-120	SHS-1001	B-10	A106	0.75	0.154	IWC-1222(A)		715	505	Y	6B-MS-600-1B BYPASS TO VALVE 3506.
.75B-MS-600-1B	MS-200	SHS-1001	B-8	A106	0.75	0.154	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3406.
.75C-MS-600-1B	MS-200	SHS-1001	B-8	A106	0.75	0.154	IWC-1222(A)		715	505	Y	30" MS PIPE TO VALVE 3404.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: NUCLEAR SAMPLING
P&ID No: 33013-1278

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Flg.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.5A-SL-151					0.50	0.000	IWC-1222(A)		30	80	-	VALVE 999B TO 1ST .5X.5X.5 TEE.
.5B-SL-151					0.50	0.000	IWC-1222(A)		30	80	-	VALVE 969 TO 1ST .5X.5X.5 TEE.
.375A-CH-151					0.37	0.000			30	80	-	THIS LINE IS ACCOUNTED FOR ON 33013-1265
.375A-SL-2505					0.37	0.000	IWC-1222(A)		2235	602	-	VALVE 966C OUT CONT TO PEN 205.
.375B-SL-2505					0.37	0.000	IWC-1222(A)		2235	650	-	V 966B OUT CONT PAST PEN 206 TO V 953.
.375C-SL-2505					0.37	0.000	IWC-1222(A)		2235	650	-	V 966A OUT CONT PAST PEN 207 TO V 951.
.375D-SL-2505					0.37	0.000			40	110	-	THIS LINE IS ACCOUNTED FOR ON 33013-1264
.375E-SL-2505					0.37	0.000			250	110	-	THIS LINE IS ACCOUNTED FOR ON 33013-1264
.375F-SL-2505					0.37	0.000			30	100	-	THIS LINE IS ACCOUNTED FOR ON 33013-1265
.375G-SL-2505					0.37	0.000	IWC-1222(A)		2235	602	-	VALVE 959 TO VALVE 958.
.25A-SL-2505					0.25	0.000	IWC-1222(A)		2235	602	-	PEN 205 PAST DELAY COIL TO VALVE 955.
.25B-SL-2505					0.25	0.000	IWC-1222(A)		2235	602	-	.25A-SL-2505 TO VALVE 998.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: POST ACCIDENT SAMPLING
P&ID No: 33013-1279

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.5A-PAS-153					0.50	0.000	IWC-1222(A)		30	80	-	TEE ON .5A-SL-151 & .5B TO VALVE 4268.
.5B-PAS-2505					0.50	0.000	IWC-1222(A)		30	80	-	.5A-PAS-153 TO VALVE 426.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: PUMPS & VALVES
P&ID No: 33013-1247

RG&E Line No.	GILBERT Line No.	SURI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
RHR PUMP A	1247		B-28		0.00	0.000		SUR	410	350	-	MFG. PACIFIC
RHR PUMP B	1247		B-28		0.00	0.000		SUR	410	350	-	MFG. PACIFIC

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: PUMPS & VALVES
P&ID No: 33013-1261

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
CS PUMP 1	1261				0.00	0.000		VT	205	70	-	MFG. INGERSOL RAND
CS PUMP 2	1261				0.00	0.000		VT	205	70	-	MFG. INGERSOL RAND

QUALITY ASSURANCE MANUAL
GINNA STATION

TITLE:
ATTACHMENT A
APPENDIX B
GINNA NUCLEAR POWER STATION
INSERVICE INSPECTION PROGRAM
FOR THE 1990-1999 INTERVAL

REV. 0

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: PUMPS & VALVES
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
SI PUMP A	1262				0.00	0.000		VT	1550	200	-	MFG. WORTHINGTON
SI PUMP B	1262				0.00	0.000		VT	1550	200	-	MFG. WORTHINGTON
SI PUMP C	1262				0.00	0.000		VT	1550	200	-	MFG. WORTHINGTON

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: PUMPS & VALVES
P&ID No: 33013-1265

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
CHARGING PUMP 1	1265				0.00	0.000	IWC-1222(A)		2450	100	-	MFG. AJAX
CHARGING PUMP 2	1265				0.00	0.000	IWC-1222(A)		2450	100	-	MFG. AJAX
CHARGING PUMP 3	1265				0.00	0.000	IWC-1222(A)		2450	100	-	MFG. AJAX

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: RCS OVERPRESS. PROT. N2 ACCUM.
P&ID No: 33013-1263

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
1A-RCSOP-A					1.00	0.000	IWC-1222(A)				?	PCV-430 TO VALVE 8606A.
1B-RCSOP-B					1.00	0.000	IWC-1222(A)				?	PCV-431C TO VALVE 8606B.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: REACTOR COOLANT
P&ID No: 33013-1260

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
6-AC-601	RHR-100	6-RH-2015	B-17	A312	6.00	0.280					Y	THIS LINE ACCOUNTED FOR ON 33013-1247.
6A-AC-601	RHR-100	6-RH-2016	B-18	A312	6.00	0.280					Y	THIS LINE ACCOUNTED FOR ON 33013-1247.
.75A-RC-2501					0.75	0.000	IWC-1222(A)		2235	80	-	VALVE 597 TO 1ST .75X.375 RDCR.
.75B-RC-2501					0.75	0.000	IWC-1222(A)		2235	80	-	FLANGE BEFORE VALVE 500A TO .75X.375 RDCR
.75C-RC-2501					0.75	0.000	IWC-1222(A)		0	0	-	SEAL CHAMBER VENT PAST VALVES 598 & 599.
.75D-RC-2501					0.75	0.000	IWC-1222(A)		0	100	-	FLANGE PAST VALVE SV592 TO VALVE SV590.
.75E-RC-2501					0.75	0.000	IWC-1222(A)		0	100	-	FLANGE PAST VALVE SV593 TO VALVE SV591.
.375A-RC-2501					0.37	0.000	IWC-1222(A)		2235	80	-	.75X.375 RDCR PAST VALVE 595.
.375B-RC-2501					0.37	0.000	IWC-1222(A)		2235	80	-	.75X.375 RDCR TO VALVE 594.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: REACTOR COOLANT PRESSURIZER
P&ID No: 33013-1258

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-RC-151					2.00	0.000	IWC-1222(A)		60	70	-	VALVE 529 BY PEN 121 TO VALVE 508.
.75A-RC-151					0.75	0.000	IWC-1222(A)		125	70	-	VALVE 528 BY PEN 121 TO VALVE 441.
.75E-RC-151					0.75	0.000	IWC-1222(A)		1.5	70	-	THIS LINE ACCOUNTED FOR ON 33013-1272.
.375A-RC-2505					0.37	0.000	IWC-1222(A)		5	90	-	PEN 120 IN CONT TO VALVE 492 OUT CONT.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: RESIDUAL HEAT REMOVAL
P&ID No: 33013-1247

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
10-AC-601	RHR-100	10-RH-2013	B17	A312	10.00	0.365		SUR/VOL	350	350	Y	V 720 TO REDUCER 10X6 BEFORE VALVE 717.
10A-AC-601	RHR-300	10-RH-2001	B-22	A312	10.00	0.365		SUR/VOL	410	350	Y	VAL 701 TO 1ST TEE BEFORE VAL 704A & B.
10B-AC-601	RHR-300	10-RH-2001	B-20	A312	10.00	0.365		SUR/VOL	410	350	Y	10" TEE BY PEN 140 TO REFUEL WATER S TK.
10C-AC-601	RHR-300	10-RH-2004	B-20	A312	10.00	0.365		SUR/VOL	350	350	Y	TEE ON 10D-AC-601 TO RHR PUMP A.
10D-AC-601	RHR-300	10-RH-2004	B-20	A312	10.00	0.365		SUR/VOL	350	350	Y	TEE ON 10C-AC-601 TO RHR PUMP B.
10E-AC-601	RHR-300	10-RH-2003	B-20	A312	10.00	0.365		SUR/VOL	350	350	Y	TEE ON 10D-AC-601 TO VALVE 850B.
10EE-AC-601	RHR-300	10-RH-2003	B-20	A312	10.00	0.365	IWC-1221(F)		350	350	Y	VALVE 850B TO 1ST REDUCING ELBOW.
10G-AC-601	RHR-300	10-SI-2010	B-20	A312	10.00	0.365		SUR/VOL	350	350	Y	VALVE 850A PAST TEE TO 10C-AC-601.
10GG-AC-601	RHR-300	10-SI-2010	B-20	A312	10.00	0.365	IWC-1221(F)		350	350	Y	VALVE 850A TO 1ST REDUCING ELBOW.
10H-AC-601	RHR-350,RHR-400	10-RH-2007	B-24	A312	10.00	0.365		SUR/VOL	410	350	N	CONT. OF 8B-AC-601 RED ELBOW TO TEE.
8-AC-601	RHR-400	8-RH-2010	B-26	A312	8.00	0.322		SUR/VOL	350	350	Y	10-AC-601 TEE-RED TO TEE PAST HVC-624.
8A-AC-601	RHR-400	8-RH-2010	B-26	A312	8.00	0.322		SUR/VOL	350	350	Y	10X8 REDUCER TO RESIDUAL HEAT EXCH A
8B-AC-601	RHR-350		B-23	A312	8.00	0.322		SUR/VOL	410	350	N	DISCHG RHR PMP A TO 1ST REDUCING ELBOW.
8C-AC-601	RHR-350	8-RH-2008	B-23	A312	8.00	0.322		SUR/VOL	410	350	N	10" TEE ON 10H-AC-601 TO RH EX B.
8D-AC-601	RHR-350	8-RH-2008	B-23	A312	8.00	0.322		SUR/VOL	410	350	N	DISCHG RHR PMP B TO 1ST TEE AFTER V 710B
8E-AC-601	RHR-400	8-RH-2017	B-24	A312	8.00	0.322		SUR/VOL	410	350	Y	REDUCER BEFORE VALVE 714 TO RH HT EX A.
8F-AC-601	RHR-300	8-SI-2012	B-20	A312	8.00	0.500	IWC-1221(F)		350	350	Y	VALVE 851A TO RED ELBOW BY VALVE 850A.
8FF-AC-601	RHR-300	8-SI-2009	B-20	A312	8.00	0.500	IWC-1221(F)		350	350	Y	VALVE 851B TO RED ELBOW BY VALVE 850B.
8K-SI-151	SI-151		B-19	A312	8.00	0.148		SUR/VOL			Y	8X6 REDUCER ON 6C-AC-601 TO 10X8 REDUCER
8X-AC-601	RHR-400	8-RH-2018	B-26	A312	8.00	0.322		SUR/VOL	410	350	-	RES HEAT EX B TO 8X8X6 REDUCER TEE.
6-AC-601	RHR-100	6-RH-2015	B-17	A312	6.00	0.280		SUR/VOL	350	350	Y	VALVE 852A TO 10-AC-601 IN CONT.
6A-AC-601	RHR-100	6-RH-2016	B-18	A312	6.00	0.280		SUR/VOL	350	350	Y	VALVE 852B TO 10-AC-601 IN CONT.
6B-AC-601	RHR-400	6-RH-2012	B-24	A312	6.00	0.280		SUR/VOL	350	350	Y	10-AC-601 TO 10A-AC-601.
6C-AC-601	RHR-400,RHR-450	6-RH-2014	B-27	A304	6.00	0.280		SUR/VOL	410	350	Y	8A-AC-601 TO 8X6 REDUCER AFTER VAL 857C.
6D-AC-601	RHR-450	6-RH-2011	B-26	A312	6.00	0.280		SUR/VOL	410	350	Y	TEE AFTER V 857A TO 8" TEE BY RES H EX B
6E-AC-151	RHR-300	6-SI-2011	B-20	A312	6.00	0.134	IWC-1221(F)		350	350	N	10" TEE ON 10EE-AC-601 TO VALVE 1813B.
6K-AC-151	RHR-300	6-SI-2014	B-20	A312	6.00	0.134	IWC-1221(F)		350	350	N	10" TEE BETWEEN V 850A & PEN 141.
4C-SI-151					4.00	0.000			410	350	-	THIS LINE ACCOUNTED FOR ON 33013-1262.
2-CH-151				A312	2.00	0.109			410	350	-	THIS LINE ACCOUNTED FOR ON 33013-1264.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: RESIDUAL HEAT REMOVAL
P&ID No: 33013-1247

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-AC-601	RHR-300,RHR-400		B-25	A312	2.00	0.154	IWC-1221(A)		350	350	3	10-AC-601 TO 10B-AC-601.
2B-AC-601	RHR-350		B-24	A312	2.00	0.154	IWC-1221(A)		410	350	N	8C-AC-601 TO VALVE 1812A.
2C-AC-601	RHR-400		B-24	A312	2.00	0.154	IWC-1221(A)		410	350	N	8E-AC-601 TO VALVE 1812B.
2F-AC-601	CVC-100		B-17	A312	2.00	0.154	IWC-1221(A)		350	350	Y	THIS LINE ACCOUNTED FOR ON 33013-1264.
RHEA-2A-AC-601				A312	2.00	0.154	IWC-1221(A)		410	350	-	RESIDUAL HEAT EXCH. A TO VALVE 807C.
RHEA-2B-AC-601				A312	2.00	0.154	IWC-1221(A)		410	350	-	RESIDUAL HEAT EXCH. A TO VALVE 807D.
RHEA-2C-AC-601				A312	2.00	0.154	IWC-1221(A)		410	350	-	RESIDUAL HEAT EXCH. A TO VALVE 807E.
RHEB-2A-AC-601				A312	2.00	0.154	IWC-1221(A)		410	350	-	RESIDUAL HEAT EXCH. B TO VALVE 807B.
RHEB-2B-AC-601				A312	2.00	0.154	IWC-1221(A)		410	350	-	RESIDUAL HEAT EXCH. B TO VALVE 807F.
RHEB-2C-AC-601				A312	2.00	0.154	IWC-1221(A)		410	350	-	RESIDUAL HEAT EXCH. B TO VALVE 807G.
.75-AC-601	RHR-400		B-25	A312	0.75	0.113	IWC-1221(A)		350	350	Y	VALVE 2780 TO 10-AC-601.
.75A-AC-601			B-25	A312	0.75	0.133	IWC-1221(A)		350	350	Y	2A-AC-601 TO VALVE 719A.
.75B-AC-601			B-25	A312	0.75	0.133	IWC-1221(A)		350	350	Y	2A-AC-601 TO VALVE 719B.
.75BA-AC-601	RHR-100	10-RH-2013	B-17	A312	0.75	0.133	IWC-1221(A)		350	350	Y	10-AC-601 TO CAP.
.75BB-AC-601	RHR-100	6-RH-2015	B-17	A312	0.75	0.133	IWC-1221(A)		350	350	Y	6-AC-601 PAST VALVE 2848 TO CAP.
.75BC-AC-601	RHR-100	6-RH-2015	B-17	A312	0.75	0.133	IWC-1221(A)		350	350	Y	6-AC-601 PAST VALVE 2847 TO CAP.
.75BD-AC-601	RHR-100	6-RH-2016	B-18	A312	0.75	0.133	IWC-1221(A)		350	350	Y	6A-AC-601 PAST VALVE 2840 TO CAP.
.75BE-AC-601	RHR-100	6-RH-2016	B-18	A312	0.75	0.133	IWC-1221(A)		350	350	Y	6A-AC-601 PAST VALVE 2853 TO CAP.
.75BF-AC-601	RHR-400		B-24	A312	0.75	0.133	IWC-1221(A)		350	350	Y	2-AC-601 PAST VALVE 2785 TO CAP.
.75BG-AC-601	RHR-400		B-24	A312	0.75	0.133	IWC-1221(A)		350	350	Y	10H-AC-601 PAST VALVE 807A TO CAP.
.75BH-AC-601	RHR-350		B-24	A312	0.75	0.133	IWC-1221(A)		410	350	Y	2B-AC-601 PAST VALVE 2784 TO CAP.
.75BI-AC-601	RHR-350	8-RH-2008	B-23	A312	0.75	0.133	IWC-1221(A)		350	350	Y	8C-AC-601 PAST VALVE 2782 TO CAP.
.75C-AC-601	RHR-400		B-25	A312	0.75	0.133	IWC-1221(A)		350	350	Y	VALVE 2779 TO 10-AC-601.
.75D-AC-601	RHR-400		B-25	A312	0.75	0.133	IWC-1221(A)		350	350	Y	VALVE 958 TO 10-AC-601.
.75E-AC-601	RHR-400		B-25	A312	0.75	0.133	IWC-1221(A)		350	350	Y	VALVE 718B TO 10-AC-601.
.75F-AC-601	RHR-400		B-25	A312	0.75	0.133	IWC-1221(A)		350	350	Y	VALVE 718A TO 10-AC-601.
.75G-AC-601	RHR-450		B-26	A312	0.75	0.133	IWC-1221(A)		410	350	N	FLANGE BY VALVE 2788 TO 6D-AC-601.
.75H-AC-601	RHR-450		B-26	A312	0.75	0.133	IWC-1221(A)		410	350	Y	VALVE 1829C TO 6D-AC-601.
.75I-AC-601	RHR-450		B-26	A312	0.75	0.133	IWC-1221(A)		410	350	Y	VALVE 1829D TO 6D-AC-601.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: RESIDUAL HEAT REMOVAL
P&ID No: 33013-1247

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75J-AC-601	RHR-450		B-27	A312	0.75	0.113	IWC-1221(A)		410	350	N	FLANGE BY VALVE 2788 TO 6C-AC-601.
.75JA-AC-601	RHR-300		B-20	A312	0.75	0.133	IWC-1221(A)		410	350	N	DRAIN VAL 850B TO FLANGE AFTER VAL 711E.
.75K-AC-601	RHR-450		B-27	A312	0.75	0.113	IWC-1221(A)		410	350	N	VALVE 1829A TO 6C-AC-601.
.75KA-AC-601	RHR-300		B-20	A312	0.75	0.133	IWC-1221(A)		410	350	N	10F-AC-601 BEFORE RHR PMP A TO FLANGE.
.75L-AC-601	RHR-450		B-27	A312	0.75	0.113	IWC-1221(A)		410	350	N	VALVE 1829B TO 6C-AC-601.
.75LA-AC-601	RHR-300		B-20	A312	0.75	0.133	IWC-1221(A)		410	350	N	DRAIN FROM VALVE 850A TO .75K-AC-601.
.75LB-AC-601	RHR-300		B-28	A312	0.75	0.133	IWC-1221(A)		410	350	N	RHR PUMP A TO VALVE 706A.
.75M-AC-601	RHR-300		B-28	A312	0.75	0.133	IWC-1221(A)		350	350	Y	DRAIN FM RHR PUMP A TO VALVE 705A.
.75N-AC-601	RHR-350		B-23	A312	0.75	0.113	IWC-1221(A)		410	350	N	8B-AC-601 TO VALVE 711C.
.75O-AC-601	RHR-350		B-23	A312	0.75	0.113	IWC-1221(A)		410	350	N	8D-AC-601 TO VALVE 711B.
.75P-AC-601	RHR-350		B-28	A312	0.75	0.113	IWC-1221(A)		410	350	N	RHR PUMP B DRAIN TO VALVE 705B.
.75Q-AC-601	RHR-350		B-28	A312	0.75	0.113	IWC-1221(A)		350	350	N	RHR PUMP B TO VALVE 706B.
.75R-AC-601	RHR-2500		B-22	A312	0.75	0.113	IWC-1221(A)		410	350	-	10A-AC-601 TO FLANGE PAST VALVE 2763.
.75S-AC-601	RHR-2500		B-22	A312	0.75	0.113	IWC-1221(A)		410	350	-	10A-AC-601 TO FLANGE PAST VALVE 2786.
.75T-AC-601	RHR-400		B-24	A312	0.75	0.113	IWC-1221(A)		410	350	-	10H-AC-601 TO VALVE 711A.
.75U-AC-601	RHR-400		B-24	A312	0.75	0.113	IWC-1221(A)		410	350	-	10H-AC-601 TO FLANGE PAST VALVE 2781.
18A-AC-601	RHR-400	8-RH-2010	B-26	A312	0.75	0.133	IWC-1221(A)		410	350	Y	8-AC-601 TO VALVE 2778.
CONT SUMP B	RHR-300				0.00	0.000			ATM.	<200	-	CONCRETE SUMP EXEMPT FROM EXAMINATION.
RHR PUMP A	RHR-300,RHR-350		B-28		0.00	0.000		SUR	410	350	-	
RHR PUMP B	RHR-300,RHR-350		B-28		0.00	0.000		SUR	410	350	-	

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SAFETY INJECTION SYSTEM
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
10A-SI-902	SIS-200			A376	10.00	1.000	IWC-1221(E)		740	80	-	ACCUMULATOR 1 TO VALVE 842A.
10B-SI-902	SIS-200			A376	10.00	1.000	IWC-1221(E)		740	80	-	10A-SI-902 TO CAP ON TEE.
10C-SI-902	SIS-200			A376	10.00	1.000	IWC-1221(E)		740	80	-	10A-SI-902 TO CAP ON TEE.
10D-SI-902	SI-100			A376	10.00	1.000	IWC-1221(E)		740	80	Y	ACCUMULATOR 2 TO VALVE 842B.
10E-SI-902	SI-100			A376	10.00	1.000	IWC-1221(E)		740	80	-	10D-SI-902 TO TEE CAP.
10F-SI-902	SI-100			A376	10.00	1.000	IWC-1221(E)		740	80	-	10D-SI-902 TO TEE CAP.
8A-SI-301B	SI-400,8-SI-301	8-SI-2001	B-15	A312	8.00	0.322			5	210	Y	THIS LINE ACCOUNTED FOR ON 33013-1266.
8B-SI-301A	SI-400,8-SI-301	8-SI-2001	B-15	A312	8.00	0.322			5	210	Y	THIS LINE ACCOUNTED FOR ON 33013-1266.
8C-SI-301	SI-400,8-SI-301	8-SI-2001	B-15	A312	8.00	0.322	IWC-1221(E)		200	200	Y	8A/8B-SI-301 TO 8E/8D-SI-301.
8D-SI-301	SI-400,SI-301	8-SI-2003	B-16	A312	8.00	0.322	IWC-1221(E)		200	200	Y	8C-SI-301 TO TEE PAST VALVE 826B.
8E-SI-301	SI-400,8-SI-301	8-SI-2002	B-16	A312	8.00	0.322	IWC-1221(E)		200	200	Y	8C-SI-301 TEE TO 1ST TEE PAST VALVE 826D
8F-SI-301	SI-400,RHR-450	8-SI-2003	B-16A	A312	8.00	0.322	IWC-1221(E)		200	200	Y	8D/8E-SI-301 TO 1ST 8X4 RDCR.
8G-SI-151	RHR-450		B-19	A312	8.00	0.322	IWC-1221(E)		200	200	Y	8F-SI-301 TO 8H-SI-151.
8H-SI-151	RHR-450		B-16B	A312	8.00	0.148	IWC-1221(E)		200	200	-	8F-SI-301 TO 10-SI-151
4-SI-301	RHR-450		B-16B	A312	4.00	0.237	IWC-1221(E)		200	200	Y	8F-SI-301 TO SI PUMP SUCTION #1.
4A-SI-301	RHR-450		B-16B	A312	4.00	0.237	IWC-1221(A)		200	200	Y	8H-SI-151 TO SI PUMP SUCTION #3.
4B-SI-301	RHR-450		B-16B	A312	4.00	0.237	IWC-1221(E)		200	200	Y	8G-SI-151 TO SI PUMP SUCTION #2.
4C-SI-151	RHR-450		B-16B	A312	4.00	0.237	IWC-1221(E)		200	200	Y	4B-SI-301 PAST V 1816A & B TO 6C-AC-601.
4D-SI-1501	SI-300,SI-210		B-37	A312	4.00	0.337		SUR/VOL	1550	200	N	3A-SI-1501 TO VALVES 878B & 878A.
4E-SI-1501	SI-300,SI-110		B-42	A312	4.00	0.337		SUR/VOL	1550	200	-	3E-SI-1501 TO TEE BEFORE V 878C & 878D
3A-SI-1501	SI-300		B-37	A312	3.00	0.300		SUR/VOL	1550	200	-	SI PUMP A TO RDCR AFTER VALVE 888A.
3B-SI-1501	SI-300		B-40	A312	3.00	0.300		SUR/VOL	1550	200	-	SI PUMP C PAST FLANGE TO 3" TEE.
3C-SI-1501	SI-300		B-41	A312	3.00	0.300		SUR/VOL	1550	200	-	3B-SI-1501 PAST V 871A & 870A 4D-SI-1501
3D-SI-1501	SI-300		B-41	A312	3.00	0.300		SUR/VOL	1550	200	-	3B-SI-1501 PAST V 871B & 870B 3E-SI-1501
3E-SI-1501	SI-300		B-42	A312	3.00	0.300		SUR/VOL	1550	200	-	SI PUMP B PAST V 888B TO RDCR PEN 101.
2A-SI-1501	SI-210		B-39	A312	2.00	0.218		SUR	1550	200	N	4D-SI-1501 PAST VAL 878A TO VAL 878F.
2B-SI-1501	SI-210		B-39	A376	2.00	0.218		SUR	1550	200	N	4D-SI-1501 PAST VAL 878B TO VAL 878G.
2C-SI-1501	SI-110		B-45	A376	2.00	0.343		SUR	1550	200	Y	4" TEE ON 4E-SI-1501 TO VALVE 878J.
2D-SI-1501	SI-110		B-44	A312	2.00	0.218		SUR	1550	200	Y	4" TEE ON 4E-SI-1501 TO VALVE 878H.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SAFETY INJECTION SYSTEM
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2G-SI-902	SIS-200			A312	2.00	0.343	IWC-1221(E)		740	80	-	10A-SI-902 TO VALVE 844A.
2H-SI-902	SI-101			A312	2.00	0.154	IWC-1221(E)		740	80	-	100-SI-902 TO VALVE 844B.
2I-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	ACCUMULATOR 1 PAST V 833A TO LT 939.
2J-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	LT 939 PAST V 833B TO ACCUMULATOR 1.
2K-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	ACCUMULATOR 1 PAST V 832A TO LT 938.
2L-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	LT 938 PAST V 832B TO ACCUMULATOR 1.
2M-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	ACCUMULATOR 2 PAST V 837A TO LT 934.
2N-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	LT 934 PAST V 837B TO ACCUMULATOR 2.
2P-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	ACCUMULATOR 2 PAST V 838A TO LT 395.
2Q-SI-902				A312	2.00	0.154	IWC-1221(E)		740	80	-	LT 395 PAST V 838B TO ACCUMULATOR 2.
1A-SIP1				A312	1.00	0.133	IWC-1221(A)		200	200	-	SIAPSI-1 TO VALVE 2803.
1B-SIP1				A312	1.00	0.133	IWC-1221(A)		200	200	-	SIAPSI-1 TO VALVE 2802.
1C-SIP1				A312	1.00	0.133	IWC-1221(B)		200	200	-	SIAPSI-1 TO VALVE 2801.
1D-SIP1				A312	1.00	0.133	IWC-1221(B)		200	200	-	SIAPSI-1 TO VALVE 2804.
1E-SIP2				A312	1.00	0.133	IWC-1221(A)		200	200	-	SIAPSI-2 TO VALVE 2814.
1F-SIP2				A312	1.00	0.133	IWC-1221(A)		200	200	-	SIAPSI-2 TO VALVE 2811.
1G-SIP2				A312	1.00	0.133	IWC-1221(B)		200	200	-	SIAPSI-2 TO VALVE 2812.
1H-SIP2				A312	1.00	0.133	IWC-1221(B)		200	200	-	SIAPSI-2 TO VALVE 2813.
1I-SIP3				A312	1.00	0.133	IWC-1221(A)		200	200	-	SIAPSI-3 TO VALVE 2808.
1J-SIP3				A312	1.00	0.133	IWC-1221(A)		200	200	-	SIAPSI-3 TO VALVE 2806.
1K-SIP3				A312	1.00	0.133	IWC-1221(B)		200	200	-	SIAPSI-3 TO VALVE 2809.
1M-SI-1501				A312	1.00	0.133	IWC-1221(B)		200	200	-	RDCR TOWARDS SIAPSI-1 TO VALVE 897.
1P-SI-1501	SI-110			A312	1.00	0.133	IWC-1221(B)		1550	200	-	75J-SI-1501 TO VALVE 887.
1Q-SI-1501	SI-110			A312	1.00	0.133	IWC-1221(B)		1550	200	-	4E-SI-1501 PAST V 835B TO ACCUMULATOR 2.
1R-SI-1501	SI-110			A312	1.00	0.133	IWC-1221(B)		1550	200	-	1Q-SI-1501 PAST V 835A TO ACCUMULATOR 1.
1R-SI-902				A312	1.00	0.133	IWC-1221(E)		740	80	-	ACCUMULATOR 1 TO VALVE 834A.
1S-SI-902				A312	1.00	0.133	IWC-1221(E)		740	80	-	ACCUMULATOR 1 TO VALVE 830A.
1T-SI-902				A312	1.00	0.133	IWC-1221(E)		740	80	-	1S-SI-902 TO VALVE 886A.
1U-SI-902	SI-100			A312	1.00	0.133	IWC-1221(E)		740	80	-	ACCUMULATOR 2 TO VALVE 834B.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SAFETY INJECTION SYSTEM
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
1V-SI-902	SI-100			A312	1.00	0.133	IWC-1221(E)		740	80	-	ACCUMULATOR 2 TO VALVE 830B.
1W-SI-902	SI-100			A312	1.00	0.133	IWC-1221(E)		740	80	-	1V-SI-902 TO VALVE 886B.
1X-SI-903					1.00	0.000	IWC-1221(A)		740	80	-	V 846 OUT CONT PAST PEN 120 TO V 8623.
.75A-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	-	1H-SI-1501 TO 3A-SI-1501
.75AA-SI-301	SI-400		B-16		0.75	0.000	IWC-1221(A)		200	200	-	8E-SI-301 TO FLANGE PAST VALVE 1822.
.75B-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	-	1H-SI-1501 TO VALVE 884.
.75BB-SI-301	SI-400		B-16		0.75	0.000	IWC-1221(A)		200	200	-	8D-SI-301 TO FLANGE PAST VALVE 1821.
.75BC-SI-151	RHR-450		B-19		0.75	0.000	IWC-1221(A)		200	200	-	8H-SI-151 TO FLANGE PAST VALVE 2817.
.75BD-SI-151	RHR-450		B-19		0.75	0.000	IWC-1221(A)		200	200	-	8G-SI-151 BYPASS.
.75BE-SI-1501	SI-300		B-37		0.75	0.000	IWC-1221(B)		1550	200	-	4D-SI-1501 PAST VALVE 885A.
.75BF-SI-1501	SI-300		B-37		0.75	0.000	IWC-1221(B)		1550	200	-	4D-SI-1501 PAST VALVE 828A.
.75BG-SI-1501	SI-300		B-37		0.75	0.000	IWC-1221(B)		1550	200	-	4D-SI-1501 PAST VALVE 828B.
.75BH-SI-1501	SI-210		B-39		0.75	0.000	IWC-1221(B)		1550	200	-	2B-SI-1501 TO FLANGE PAST VALVE 2842.
.75BI-SI-1501	SI-210		B-39		0.75	0.000	IWC-1221(B)		1550	200	-	2A-SI-1501 TO FLANGE PAST VALVE 2841.
.75C-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	-	3B-SI-1501 TO 1H-SI-1501.
.75D-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	-	3E-SI-1501 TO 1H-SI-1501.
.75E-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	-	3E-SI-1501 TO VALVE 2810.
.75F-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	-	3E-SI-1501 TO VALVE 885B.
.75G-SI-1501	SI-300			A312	0.75	0.154	IWC-1221(B)		1550	200	N	4D-SI-1501 PAST V 872A PEN 110 TO V 879.
.75H-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	N	4E-SI-1501 TO TEE PAST VALVE 880B.
.75I-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	-	.75G-SI-1501 TO .75H-SI-1501.
.75J-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	-	.75I-SI-1501 TO VALVE 2854.
.75M-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	-	1Q-SI-1501 TO VALVE 3836.
.75N-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	N	4E-SI-1501 TO VALVE 829A.
.75P-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	N	4E-SI-1501 TO VALVE 829B.
.75Q-SI-1501	SI-110			A376	0.75	0.218	IWC-1221(B)		1550	200	Y	4E-SI-1501 TO FLANGE PAST VALVE 2833.
.75R-SI-1501	SI-110				0.75	0.000	IWC-1221(B)		1550	200	-	2D-SI-1501 PAST VALVE 2832 TO FLANGE.
.75S-SI-1501	SI-110			A312	0.75	0.154	IWC-1221(B)		1550	200	-	1R-SI-1501 TO FLANGE PAST VALVE 2846.
.75T-SI-902					0.75	0.000	IWC-1221(E)		740	80	-	ACCUMULATOR 1 TO VALVE 843A.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SAFETY INJECTION SYSTEM
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75U-SI-902	SI-100				0.75	0.000	IWC-1221(E)		740	80	-	ACCUMULATOR 2 TO VALVE 843B.
.75V-SI-2501				A376	0.75	0.218	IWC-1221(E)		740	80	-	100-SI-902 TO .75H-SI-1501.
.75W-SI-2501				A376	0.75	0.218	IWC-1221(E)		740	80	-	100-SI-902 TO VALVE 840B.

QUALITY ASSURANCE MANUAL GINNA STATION

TITLE: ATTACHMENT A APPENDIX B GINNA NUCLEAR POWER STATION INSERVICE INSPECTION PROGRAM FOR THE 1990-1999 INTERVAL

REV. 0

42-92D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SERVICE WATER
P&ID No: 33013-1250

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
8-SW-125	SW-1400			A53	8.00	0.322	IWC-1222(C)		60	80	Y	VALVE 4644 TO 8" TEE BY PEN 323 OUT CONT
8A-SW-125	SW-1400			A53	8.00	0.322	IWC-1222(C)		60	80	Y	PEN 323 TO COOLING COILS & FAN MOTORS.
8B-SW-125	SW-700			A53	8.00	0.322	IWC-1222(C)		60	80	Y	CVCC & FAN MOTOR AAGRO2D TO PEN 312.
8C-SW-125	SW-1500			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW AFTER PEN 312 TO VALVE 4642.
8D-SW-125	SW-1400			A53	8.00	0.322	IWC-1222(C)		60	80	Y	VALVE 4643 TO 8" TEE BEFORE PEN 315.
8E-SW-125	SW-100			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW PAST PEN 315 TO AAGRO2C.
8F-SW-125	SW-200			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW PAST PEN 320 TO AAGRO2C.
8G-SW-125	SW-1500			A53	8.00	0.322	IWC-1222(C)		60	80	Y	VALVE 4641 TO RED ELBOW PAST PEN 320.
8H-SW-125	SW-1400			A53	8.00	0.322	IWC-1222(C)		60	80	Y	VALVE 4630 TO 1ST 8" TEE BY PEN 311.
8I-SW-125	SW-150			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW BY PEN 311 TO AAGRO2B.
8J-SW-125	SW-600			A53	8.00	0.322	IWC-1222(C)		60	80	Y	COOLING COILS & FAN MOTORS TO PEN 316.
8K-SW-125	SW-1500			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW AFTER PEN 316 TO VALVE 4628.
8L-SW-125	SW-1400			A53	8.00	0.322	IWC-1222(C)		60	80	Y	VALVE 4629 TO 8" TEE BY PEN 308.
8M-SW-125	SW-200			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW BY PEN 308 TO AAGRO2A.
8N-SW-125	SW-500			A53	8.00	0.322	IWC-1222(C)		60	80	Y	AAGRO2A TO RED ELBOW BEFORE PEN 319.
8O-SW-125	SW-1500			A53	8.00	0.322	IWC-1222(C)		60	80	Y	RED ELBOW BY PEN 319 TO VALVE 4627.
6-SW-125	SW-1400			A53	6.00	0.280	IWC-1222(C)		60	80	Y	8" TEE BY PEN 323 TO 1ST RED ELBOW.
6A-SW-125	SW-700,SW-1500			A53	6.00	0.280	IWC-1222(C)		60	80	Y	RED ELBOW BEFORE & AFTER PEN 312.
6B-SW-125	SW-1400			A53	6.00	0.280	IWC-1222(C)		60	80	Y	8" TEE ON 8D-SW-125 TO PEN 315.
6C-SW-125	SW-200,SW-1500			A53	6.00	0.280	IWC-1222(C)		60	80	Y	RED ELBOW IN CONT & OUT CONT AT PEN 320.
6D-SW-125	SW-1400,SW-150			A53	6.00	0.280	IWC-1222(C)		60	80	Y	8" TEE BY PEN 311 TO RED ELBOW PEN 311.
6E-SW-125	SW-600,SW-1500			A53	6.00	0.280	IWC-1222(C)		60	80	Y	RED ELBOW BEFORE & AFTER PEN 316.
6F-SW-125	SW-1400,SW-200			A53	6.00	0.280	IWC-1222(C)		60	80	Y	8" TEE BY PEN 308 TO 1ST RED ELBOW 308.
6G-SW-125	SW-500,SW-1500			A53	6.00	0.280	IWC-1222(C)		60	80	Y	RED ELBOW PEN 319 IN CONT TO 319 OUT CON
3G-SW-125					3.00	0.000	IWC-1222(A)		60	80	-	BYPASS CONNECT AAGRO2A & AAGRO2B TO RDCR
2.5A-SW-125	SW-1410			A53	2.50	0.203	IWC-1222(A)		60	80	Y	VALVE 4636 TO 2.5X2.5X2 TEE PAST PEN 201
2.5B-SW-125	SW-800			A53	2.50	0.203	IWC-1222(A)		60	80	Y	1ST ELBOW PAST PEN 201 TO AAGRO1B.
2.5C-SW-125	SW-450			A53	2.50	0.203	IWC-1222(A)		60	80	Y	REACTOR COOLER AAGRO1B TO PEN 209.
2.5D-SW-125	SW-1550			A53	2.50	0.203	IWC-1222(A)		60	80	Y	2.5X2.5X2 TEE BY PEN 209 TO VALVE 4635.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SERVICE WATER
P&ID No: 33013-1250

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2.5E-SW-125	SW-1410			A53	2.50	0.203	IWC-1222(A)		60	80	Y	VALVE 4758 TO 2.5X2.5X2 TEE BY.PEN 209.
2.5F-SW-125	SW-300			A53	2.50	0.203	IWC-1222(A)		60	80	Y	1ST RED ELBOW BY PEN IN CONT TO AAGRO1A.
2.5G-SW-125	SW-800			A53	2.50	0.203	IWC-1222(A)		60	80	Y	REACTOR COOLER AAGRO1A TO PEN 201.
2.5H-SW-125	SW-1550			A53	2.50	0.203	IWC-1222(A)		60	80	Y	2.5X2.5X2 TEE BY PEN 201 TO VALVE 4757.
2.5I-SW-125					2.50	0.000	IWC-1222(A)		60	80	-	BYPASS ON AAGRO2A.
2.5J-SW-125					2.50	0.000	IWC-1222(A)		60	80	-	BYPASS ON AAGRO2B.
2.5K-SW-125					2.50	0.000	IWC-1222(A)		60	80	-	BYPASS ON AAGRO2C.
2.5L-SW-125					2.50	0.000	IWC-1222(A)		60	80	-	BYPASS ON AAGRO2D.
2A-SW-125	SW-1410,SW-800			A53	2.00	0.154	IWC-1222(A)		60	80	Y	2.5X2.5X2 TEE AT PEN 201 1ST RED ELBOW.
2B-SW-125	SW-450,SW-1550			A53	2.00	0.154	IWC-1222(A)		60	80	Y	RED ELBOW BY PEN 209 TO 1ST 2" TEE.
2C-SW-125	SW-1410,SW-300			A53	2.00	0.154	IWC-1222(A)		60	80	Y	2.5X2.5X2 TEE BY PEN 209 1ST RED ELBOW.
2D-SW-125	SW-800,SW-1550			A53	2.00	0.154	IWC-1222(A)		60	80	Y	1ST RED ELBOW BY PEN 201 TO 2" TEE.
2E-SW-125					2.00	0.000	IWC-1222(A)		60	80	-	2.5I-SW-125 PAST VALVE 4794F.
2F-SW-125					2.00	0.000	IWC-1222(A)		60	80	-	2.5J-SW-125 PAST VALVE 4794K.
2G-SW-125					2.00	0.000	IWC-1222(A)		60	80	-	3X2 RDCR ON 3A-SW-125 TO AAGRO2C.
2H-SW-125					2.00	0.000	IWC-1222(A)		60	80	-	BYPASS FM AAGRO1B TO AAGRO2D.
.75A-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2A TO VALVE 4794D.
.75B-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2A TO VALVE 4794C.
.75C-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2A TO VALVE 4794E.
.75D-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2A TO VALVE 4794B.
.75E-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2B TO VALVE 4794H.
.75F-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2B TO VALVE 4794J.
.75G-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2B TO VALVE 4794G.
.75H-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2B TO VALVE 4794V.
.75I-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2C TO VALVE 4794M.
.75J-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2C TO VALVE 4794N.
.75K-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2C TO VALVE 4794L.
.75L-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2C TO VALVE 4794W.
.75M-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2D TO VALVE 4794R.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: SERVICE WATER
P&ID No: 33013-1250

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75N-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2D TO VALVE 4794S.
.75P-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2D TO VALVE 4794Q.
.75Q-SW-125					0.75	0.000	IWC-1222(A)		60	80	-	AAGRO2D TO VALVE 4794T.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: STANDBY AUXILIARY FEEDWATER
P&ID No: 33013-1238

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3C-FW-900-1A	AFW-500	FW-1001	B-12	A106	3.00	0.300	IWC-1222(A)		715	505	Y	THIS LINE IS ACCOUNTED FOR ON 33013-1236
3C-FW-900-1B	AFW-400	FW-1005	B-13	A106	3.00	0.300	IWC-1222(A)		715	505	Y	THIS LINE IS ACCOUNTED FOR ON 30013-1236
.75D-FW-900-1A	AFW-500	FW-1001			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1A TO VALVE 9719A.
.75D-FW-900-1B	AFW-400	FW-1005			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1B TO VALVE 9719B.
.75E-FW-900-1A	AFW-500	FW-1001			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1A PAST VALVE 9727 TO FLANGE.
.75E-FW-900-1B	AFW-400	FW-1005			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1B PAST VALVE 9725 TO CAP.
.75F-FW-900-1A	AFW-500	FW-1001			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1A PAST VALVE 9726 TO FLANGE.
.75F-FW-900-1B	AFW-400	FW-1005			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1B PAST VALVE 9724 TO CAP.
.75G-FW-900-1A	AFW-500	FW-1001			0.75	0.000	IWC-1222(A)		715	505	Y	3C-FW-900-1A PAST VALVE 9723 TO CAP.

42-62D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: STEAM GENERATOR BLOWDOWN 1A
P&ID No: 33013-1277

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-MS-600-1A	SGB-100			A106	2.00	0.218	IWC-1222(A)		1005	547	Y	N. NOZZLE TO FIRST TEE GIRTH WELD.
2B-MS-600-1A	SGB-100			A106	2.00	0.218	IWC-1222(A)		1005	547	Y	S. NOZZLE TO FIRST TEE GIRTH WELD.
2C-MS-600-1A	SGB-100			A106	2.00	0.218	IWC-1222(A)		1005	547	Y	1ST GIRTH OUT FROM TEE 2A-MS-600-1A & 2B
2D-MS-600-1A	SGB-100			A106	2.00	0.218	IWC-1222(A)		1005	547	Y	2C-MS-600-1A TO 1ST 2X2X2 TEE.
2E-MS-600-1A	SGB-100,SGB-300			A106	2.00	0.218	IWC-1222(A)		1005	547	Y	2" TEE ON 2D-MS-600-1A TO VALVE 5738.
1A-MS-600-1A	SGB-100			A106	1.00	0.179	IWC-1222(A)		1005	547	Y	TEE ON 2C-MS-600-1A TO VALVE 5705.
1B-MS-600-1A	SGB-100			A106	1.00	0.179	IWC-1222(A)		1005	547	Y	S/G-1A NOZZLE TO TEE BEFORE V 5748A,5748
1E-MS-600-1A	SGB-100			A106	1.00	0.179	IWC-1222(A)		1005	547	Y	1B-MS-600-1A TO TEE PAST VALVE 5748A.
.75A-MS-600-1A	SGB-100			A106	0.75	0.154	IWC-1222(A)		1005	547	Y	1A-MS-600-1A TO VALVE 5705A.
.375A-MS-600-1A	SGB-100				0.37	0.000	IWC-1222(A)		1005	547	Y	TEE BEFORE VALVE 5731 TO VALVE 5735
.375B-MS-600-1A	SGB-100				0.37	0.000	IWC-1222(A)		1005	547	Y	BYPASS .375A-MS-600-1A TO 2E-MS-600-1A.
.375C-MS-600-1A	SGB-100				0.37	0.000	IWC-1222(A)		1005	547	Y	.375B-MS-600-1A TO VALVE 5157A.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: STEAM GENERATOR BLOWDOWN 1B
P&ID No: 33013-1277

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-MS-600-1B	SGB-200		B-31	A106	2.00	0.218	IWC-1222(A)		1005	547	Y	N. NOZZLE TO FIRST TEE GIRTH WELD.
2B-MS-600-1B	SGB-200		B-31	A106	2.00	0.218	IWC-1222(A)		1005	547	Y	S. NOZZLE TO FIRST TEE GIRTH WELD.
2C-MS-600-1B	SGB-200,SGB-400		B-31	A106	2.00	0.218	IWC-1222(A)		1005	547	Y	2A-MS-600-1B & 2B TO VALVE 5737.
1A-MS-600-1B	SGB-200		B-31	A106	1.00	0.179	IWC-1222(A)		1005	574	Y	NOZZLE ON S/B-1B TO TEE BEFORE V 5704A.
1B-MS-600-1B	SGB-200		B-31	A106	1.00	0.179	IWC-1222(A)		1005	547	Y	2C-MS-600-1B TO VALVE 5706.
1C-MS-600-1B	SGB-200			A106	1.00	0.179	IWC-1222(A)		1005	547	Y	1A-MS-600-1B TO 2C-MS-600-1B.
.375A-MS-600-1B	SGB-200,SGB-400				0.37	0.000	IWC-1222(A)		1005	547	Y	1A-MS-600-1B TEE TO VALVE 5736.
.375B-MS-600-1B	SGB-400				0.37	0.000	IWC-1222(A)		1005	547	Y	.375A-MS-600-1B TO 2C-MS-600-1B
X-MS-600-1B	SGB-400				0.00	0.000	IWC-1222(A)		1005	547	-	.375B-MS-600-1B TO VALVE 5753.
Y-MS-600-1B	SGB-400				0.00	0.000	IWC-1222(A)		1005	547	-	.375B-MS-600-1B TO VALVE 5756A.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1231

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
STEAM GEN 1A			B-1		0.00	0.000		VOL	715	505	-	
STEAM GEN 1B			B-1		0.00	0.000		VOL	715	505	-	

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1247

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
R H EX A	1247		B-109		0.00	0.000		VOL	410	350	-	
R H EX B	1247		B-109		0.00	0.000		VOL	410	350	-	

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1261

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
CS EDUCTOR 1	1261				0.00	0.000	IWC-1221(A)				-	
CS EDUCTOR 2	1261				0.00	0.000	IWC-1221(A)				-	
R W STOR TANK	1261				0.00	0.000	IWC-1221(E)		30	70	-	
SPRAY ADD TANK	1261				0.00	0.000	IWC-1221(E)				-	

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1262

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
ACCUMULATOR 1	1262				0.00	0.000	IWC-1221(E)		740	80	-	
ACCUMULATOR 2	1262				0.00	0.000	IWC-1221(E)		740	80	-	

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1264

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
NON REG H EX	1264		B-2		0.00	0.000		VOL/SUR	250	350	-	
R C FILTER	1264		B-3		0.00	0.000		VOL/SUR	40	100	-	

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1265

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
24" CHG PMP P F	1265		B-6		0.00	0.000		VOL/SUR	2450	100	-	PULSE DAMPENER
REG HEAT EX	1265		A-8		0.00	0.000			2300	525	-	CLASS 1 COMPLETE BY DEFINITION.
S W I FILTER 1	1265		B-7		0.00	0.000		VOL/SUR	2450	100	-	
S W I FILTER 2	1265		B-7		0.00	0.000		VOL/SUR	2450	1007	-	
SEAL WATER H EX	1265		B-4		0.00	0.000		VOL/SUR	40	140	-	
SEAL WATER R F	1265		B-3		0.00	0.000		VOL/SUR	40	140	-	
VOL CONTROL TK	1265		B-5		0.00	0.000	IWC-1222(C)		30	100	-	

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: VESSELS
P&ID No: 33013-1266

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
B A TANK A	1266				0.00	0.000	IWC-1221(E)		5	210	-	
B A TANK B	1266				0.00	0.000	IWC-1221(E)		5	210	-	

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: WASTE DISPOSAL
P&ID No: 33013-1272

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
4A-WD-151	RHR-300			A312	4.00	0.120	IWC-1222(A)		2	80	-	PEN 143 TO 1ST 4X3 RDCR VALVE 1721.
4B-WD-151	RHR-300			A312	4.00	0.120	IWC-1222(A)		2	80	-	3X4 RDCR ON 3A-WD-151 TO VALVE 1003A.
4C-WD-151	RHR-300			A312	4.00	0.120	IWC-1222(A)		2	80	-	4X4X4 TEE ON 4B-WD-151 TO 4X3X3 TEE.
3A-WD-151	RHR-300			A312	3.00	0.120	IWC-1222(A)		2	80	-	4X3 RDCR ON 4A-WD-151 TO VALVE 1721.
3B-WD-151	RHR-300			A312	3.00	0.120	IWC-1222(A)		2	80	-	4C-WD-151 TO 1ST 4X3X3 TEE.
3C-WD-151	RHR-300			A312	3.00	0.120	IWC-1222(A)		2	80	-	4X3X3 TEE ON 4C-WD-151 TO VALVE 1003B.
3D-WD-151				A312	3.00	0.120	IWC-1222(A)		2	80	-	PEN 107 TO VALVE 1723.
2A-WD-152					2.00	0.000	IWC-1222(A)		3	80	-	PEN 129 OUTSIDE CONT TO 1ST TEE W/RDCR'S
1A-WD-152					1.00	0.000	IWC-1222(A)		3	80	-	2A-WD-152 TO VALVE 1785.
.75A-WD-152					0.75	0.000	IWC-1222(A)		3	80	-	2A-WD-152 TO VALVE 1676A TC CONN
.75B-WD-152					0.75	0.000	IWC-1222(A)		3	80	-	TEE W/RDCR'S ON 2A-WD-152 TO VALVE 1713.
.75C-WD-152					0.75	0.000	IWC-1222(A)		3	80	-	.375X.75 RDCR ON .375A-WD-152 TO V 1789.
.75D-WD-151	RHR-300			A312	0.75	0.120	IWC-1222(A)		2	80	-	3C-WD-151 TO VALVE 1709G.
.75E-RC-151				A312	0.75	0.120	IWC-1222(A)		3	80	-	VALVE 527 TO 2A-WD-152.
.375A-WD-152					0.37	0.000	IWC-1222(A)		3	80	-	PEN 123 TO .375X.75 RDCR BEFORE V 1789.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 2
System: WASTE DISPOSAL
P&ID No: 33013-1275

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-WDGHR					2.00	0.000	IWC-1222(A)		<275	<200	-	PEN 202 IN CONT TO VALVE 1084B OUT CONT
1A-WDGHR					1.00	0.000	IWC-1222(A)		<275	<200	-	PEN 210 IN CONT TO VALVE 1080A OUT CONT
1B-WDGHR					1.00	0.000	IWC-1222(A)		<275	<200	-	PEN 304 IN CONT TO VALVE 1076A OUT CONT
1C-WDGHR					1.00	0.000	IWC-1222(A)		<275	<200	-	PEN 304 IN CONT TO VALVE 1084A OUT CONT
1D-WDGHR					1.00	0.000	IWC-1222(A)		<275	<200	-	PEN 202 IN CONT TO VALVE 1076B OUT CONT
X-WDGHR					0.00	0.000	IWC-1222(A)		<275	<200	-	SAMPLE AIR/N2 TUBING INTO CONT WALL.
Y-WDGHR					0.00	0.000	IWC-1222(A)		<275	<200	-	SAMPLE S TUBING INTO CONTAINMENT WALL.

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY COOLANT
P&ID No: 33013-1245

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
14A-ACS-152	CC-100	N/A	C-2	A53	14.00	0.375		VT-3	<150	<200	?	CC PUMPS TO 10" REDUCERS
14B-ACS-152	CC-200	N/A	C-4	A53	14.00	0.365		VT-3	<150	<200	?	734B TO REDUCER AT RHRHE
14C-ACS-152	CC-300	N/A	C-6	A53	14.00	0.375		VT-3	<150	<200	?	REDUCER AT RHR HES TO CC PUMPS
10A-ACS-152	CC-100	N/A	C-2	A53	10.00	0.365		VT-3	<150	<200	?	VALVE 723A TO 14" REDUCER
10B-ACS-152	CC-100	N/A	C-2	A53	10.00	0.365		VT-3	<150	<200	?	VALVE 723B TO 14" LINE
10C-ACS-152	CC-100	N/A	C-2	A53	10.00	0.365		VT-3	<150	<200	?	14" REDUCER TO CCHE 1.
10D-ACS-152	CC-100	N/A	C-2	A53	10.00	0.365		VT-3	<150	<200	?	14" REDUCER TO CCHE 2.
10E-ACS-152	CC-200	N/A	C-4	A53	10.00	0.365		VT-3	<150	<200	?	CCE 1 TO VALVE 734A & REDUCER
10F-ACS-152	CC-200	N/A	C-4	A53	10.00	0.365		VT-3	<150	<200	?	14B TO RHR HE 2
10G-ACS-152	CC-200	N/A	C-4	A53	10.00	0.365		VT-3	<150	<200	?	14B TO RHR HE 1
10H-ACS-152	CC-300	N/A	C-6	A53	10.00	0.365		VT-3	<150	<200	?	RHR HE 2 TO 14" REDUCER
10I-ACS-152	CC-300	N/A	C-6	A53	10.00	0.365		VT-3	<150	<200	?	RHR HE 1 TO 14" REDUCER
10J-ACS-152	CC-300	N/A	C-6	A53	10.00	0.365		VT-3	<150	<200	?	14C TO CC PUMP 1 SUCTION
10K-ACS-152	CC-300	N/A	C-6	A53	10.00	0.365		VT-3	<150	<200	?	14C TO CC PUMP 2 SUCTION
10L-ACS-152	CC-200	N/A	C-4	A53	10.00	0.365		VT-3	<150	<200	?	CCHE TO 734B
8A-ACS-152	CC-100	N/A	C-2	A53	8.00	0.365		VT-3	<150	<200	?	CC PUMP 1 TO VALVE 723A
8B-ACS-152	CC-100	N/A	C-2	A53	8.00	0.365		VT-3	<150	<200	?	CC PUMP 2 TO VALVE 723B
4A-ACS-152	CC-200	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	10C TO 6" REDUCER
4B-ACS-152	CC-200	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	CCHE1 TO ACAPCP
4C-ACS-152	CC-300	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	COMPONENT COOLING SURGE TANK TO CC PUMP
3A-ACS-152	CC-340	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	DRAIN FROM 14B TO SAMPLE HES
3B-ACS-152	CC-350	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	HES TO 772D
3C-ACS-152	CC-350	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	772D TO CC PUMPS
3D-ACS-152		N/A			3.00	0.216	IWD-1220.1		<150	<200	?	CC SURGE TANK TO 732
2A-ACS-152	CC-240,120	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	10F TO RHR PUMP 1.
2B-ACS-152	CC-240,120	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	2A TO RHR PUMP 2.
2C-ACS-152	CC-140,180	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	RHR PUMP 2 TO 2D
2D-ACS-152	CC-170,180	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	RHR PUMP 1 TO VALVE 741A
2E-ACS-152		N/A			2.00	0.154	IWD-1220.1		<150	<200	?	CC SURGE TANK TO 699

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY COOLANT
P&ID No: 33013-1245

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2F-ACS-152		N/A			2.00	0.154	IWD-1220.1		<150	<200	?	CC SURGE TANK TO 731A
2G-ACS-152		N/A			2.00	0.154	IWD-1220.1		<150	<200	?	CC SURGE TANK TO 713
2H-ACS-152	CC-170	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	823 TO 4C
1A-ACS-152	CC-230	N/A		A53	1.00	0.145	IWD-1220.1		<150	<200	?	14A TO CC SURGE TANK
1B-ACS-152	CC-340	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3A TO BLOWDOWN SAMPLE HE
1C-ACS-152	CC-340	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3A TO BLOWDOWN SAMPLE HE
1D-ACS-152	CC-340	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3A TO SAMPLE HE A
1E-ACS-152	CC-340	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3A TO SAMPLE HE B
1F-ACS-152	CC-340	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3A TO SAMPLE HE C
1G-ACS-152	CC-350	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	SAMPLE HE C TO CC PUMPS
1H-ACS-152	CC-350	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	SAMPLE HE B TO CC PUMPS
1I-ACS-152	CC-350	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	SAMPLE HE A TO CC PUMPS
1J-ACS-152	CC-350	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	BLOWDOWN SAMPLE HE 4 TO CC PUMPS
1K-ACS-152	CC-350	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	BLOWDOWN SAMPLE HE 5 TO CC PUMPS
1L-ACS-152	CC-350	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	FAIL FUEL RAD MONITOR HE
1M-ACS-152	CC-340	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3A TO FAIL FUEL RAD MON HE
1N-ACS-152		N/A			1.00	0.133	IWD-1220.1		<150	<200	?	CC SURGE TANK TO RCV017
1P-ACS-152	CC-240	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	VENT FROM 2A
.75A-ACS-152	CC-100	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 10D TO 2702
.75B-ACS-152	CC-200	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 14B TO VALVE 737A, 2742
.75C-ACS-152	CC-200	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 14B TO VALVE 735A
.75D-ACS-152	CC-200	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 14B TO VALVE 735B
.75E-ACS-152	CC-180	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2D TO 727P
.75F-ACS-152	CC-180	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2D TO 727Q
.75G-ACS-152	CC-300	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	14C TO 779A
.75H-ACS-152	CC-300	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 4C TO VALVE 727A

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY COOLANT
P&ID No: 33013-1246

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
8A-AC6-152	CC-200	N/A	C-5	A53	8.00	0.322		VT-3	<150	<200	?	COMP. CLG. HEX TO VALVE 817
8B-AC6-152	CC-220	N/A	C-5	A53	8.00	0.322		VT-3	<150	<200	?	4F TO 816
6A-AC6-152	CC-200	N/A	C-5	A53	6.00	0.280		VT-3	<150	<200	?	8A TO 813; PEN 131
6B-AC6-152	CC-310	N/A	C-3	A53	6.00	0.280		VT-3	<150	<200	?	COMP. CLG. HE TO 4A; 773
6C-AC6-152	CC-230	N/A	C-7	A53	6.00	0.280		VT-3	<150	<200	?	COMP CLG HE TO 3A & 760A & B ACID EVAP
6D-AC6-152	CC-220	N/A	C-5	A53	6.00	0.280		VT-3	<150	<200	?	PEN 130 TO 8B
6E-AC6-152	CC-160	N/A		A53	6.00	0.280		VT-3	<150	<200	?	4A TO COMP. COOL. PUMPS
6F-AC6-152	CC-500,-525	N/A	C-9	A53	6.00	0.280		VT-3	<150	<200	?	PEN 131 TO 4B
6G-AC6-152	CC-190, CC-300	N/A	C-7	A53	6.00	0.280		VT-3	<150	<200	?	BORIC ACID EVAP TO COMP COOLING PUMPS
6H-AC6-152	CC-575	N/A	C-9	A53	6.00	0.280		VT-3	<150	<200	?	4" COOLERS TO PEN 130
4A-AC6-152	CC-330	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	8A TO REDUCER BEFORE 749A
4B-AC6-152	CC-625, CC-600	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	750A TO RCP 1
4C-AC6-152	CC-160	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	6B TO 773; TO NON REG. HE TO 778
4D-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	6F TO REACTOR SUPPORT COOLER "B"
4E-AC6-152	CC-700	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	3E TO 3F
4F-AC6-152	CC-700,-725	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	3F TO PEN 126
4G-AC6-152	CC-450	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	750B AT PEN 128 TO RCP 2
4H-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	6F TO REACTOR SUPPORT COOLER "A"
4I-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	4H TO REACTOR SUPPORT COOLER "A"
4J-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	4H TO REACTOR SUPPORT COOLER "A"
4K-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	REACTOR SUPPORT COOLER "A" TO 4L
4L-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	REACTOR SUPPORT COOLER "A" TO 6H
4M-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	4D TO REACTOR SUPPORT COOLER "B"
4N-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	REACTOR SUPPORT COOLER B TO 4O
4O-AC6-152	CC-525	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	REACTOR SUPPORT COOLER B TO 6H
4P-AC6-152	CC-400	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	757B TO P125
4Q-AC6-152	CC-220	N/A		A53	4.00	0.237	IWD-1220.1		<150	<200	?	CONNECTS 8B & 2B
3A-AC6-152	CC-330	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	4A TO 749A
3B-AC6-152	CC-330	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	DRAIN FROM 4A TO 749B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
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Class: 3
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P&ID No: 33013-1246

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3C-AC6-152	CC-600	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	4B TO 751A & RCP 1
3D-AC6-152	CC-600	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	3C TO 752A TO RCP 1
3E-AC6-152	CC-700	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	RCP 1 TO REDUCER AT 4E
3F-AC6-152	CC-700	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	4F TO 757A & 4E
3G-AC6-152	CC-220	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	759A TO 8B
3H-AC6-152	CC-220	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	759B TO 4Q
3I-AC6-152	CC-400	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	RCP 2 TO 4P
3J-AC6-152	CC-525	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	4H TO REACTOR SUPPORT COOLER "A"
3K-AC6-152	CC-525	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	4D TO REACTOR SUPPORT COOLER "B"
3M-AC6-152	CC-525	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	2771 TO 4L
3N-AC6-152	CC-525	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	REACTOR SUP. COOLER B TO 2752 & 4M
3O-AC6-152	CC-450	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	4G TO 451B & RCP 2
3P-AC6-152	CC-230	N/A		A53	3.00	0.216	IWD-1220.1		<150	<200	?	6C TO WASTE EVAPORATOR
2A-AC6-152	CC-330	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	DRAIN FROM 4A TO 742A, 742C, ELHE
2B-AC6-152	CC-250	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	6B TO 777C
2C-AC6-152	CC-260	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	CS PUMPS A & B TO 6E
2D-AC6-152	CC-300	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	DISTILLATE COOLER TO 6G
2E-AC6-152	CC-230	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	6C TO DISTILLATE COOLER
2F-AC6-152	CC-525	N/A		A53	2.00	0.109	IWD-1220.1		<150	<200	?	4H TO REACTOR SUPPORT COOLER "A"
2G-AC6-152	CC-270	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	6B TO SI PUMPS
2H-AC6-152	CC-230	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	1.5C TO 6G
2I-AC6-152	CC-220	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	EXCESS LD HE TO PEN 124 & TO 4Q
2J-AC6-152	CC-525	N/A		A53	2.00	0.109	IWD-1220.1		<150	<200	?	4D TO REACTOR SUPPORT COOLER "B"
2K-AC6-152	CC-320	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	SI PUMPS TO 6E
2L-AC6-152	CC-230	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	1.5D TO 2H
2M-AC6-152	CC-700	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	4F TO 758A
2N-AC6-152	CC-310	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	6B TO 763, SEAL WATER HE
2O-AC6-152	CC-230	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	WASTE EVAP TO 764B TO 6G
2P-AC6-152	CC-400	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	4P TO 758B

R. E. GINNA NUCLEAR POWER PLANT
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Class: 3
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RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2Q-AC6-152	CC-525	N/A		A53	2.00	0.109	IWD-1220.1		<150	<200	?	REACTOR SUP. COOLER B TO 2753 & 4O
2R-AC6-152	CC-160	N/A		A53	2.00	0.154	IWD-1220.1		<150	<200	?	SEAL WATER HX TO 6B
1.5A-AC6-152	CC-700	N/A		A53	1.50	0.145	IWD-1220.1		<150	<200	?	RCP 1 TO .75AE & 4P
1.5B-AC6-152	CC-230	N/A		A53	1.50	0.145	IWD-1220.1		<150	<200	?	3A TO .75H
1.5C-AC6-152	CC-400	N/A		A53	1.50	0.145	IWD-1220.1		<150	<200	?	RCP 2 TO 755B
1.5D-AC6-152	CC-230	N/A		A53	1.50	0.145	IWD-1220.1		<150	<200	?	764A TO GAS COMPRESSOR HEX
1.5E-AC6-152	CC-230	N/A		A53	1.50	0.145	IWD-1220.1		<150	<200	?	GAS COMPRESSOR HEX TO 2L
1.5F-AC6-152	CC-230	N/A		A53	1.50	0.145	IWD-1220.1		<150	<200	?	3P TO GAS COMPRESSOR HEX
1A-AC6-152	CC-700	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	RCP 1 to 756A & 4O
1B-AC6-152	CC-160	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	766 TO 6E
1C-AC6-152	CC-230	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	2E TO STEAM AIR COOLER
1D-AC6-152	CC-450	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	3O TO 752B TO RCP 2
1E-AC6-152	CC-310	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	776 TO 4A
1F-AC6-152	CC-300	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	774B TO 6C
1G-AC6-152	CC-400	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	RCP 2 TO 4P
1H-AC6-152	CC-230	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	1.5D TO 774A TO 2D
1I-AC6-152	CC-230	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	774D TO 1.5C
1J-AC6-152	CC-300	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	STEAM AIR COOLER TO 748D TO 2D
1K-AC6-152	CC-300	N/A		A53	1.00	0.133	IWD-1220.1		<150	<200	?	774C TO 6G
.75A-AC6-152	CC-600	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4A TO 750C
.75AA-AC6-152	CC-270	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	777L TO SI PUMP 1
.75AB-AC6-152	CC-230	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	1.5C TO 727H & J
.75AC-AC6-152	CC-270	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	777F TO SI PUMP 1
.75AD-AC6-152	CC-230	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	1.5C TO 774D
.75AE-AC6-152	CC-450	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4E TO 750O
.75AF-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	SI PUMP 1 TO 777H
.75AG-AC6-152	CC-230	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 1.5D TO 2712
.75AH-AC6-152	CC-450	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3O TO 2737
.75AI-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	SI PUMP 1 TO 777E

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY COOLANT
P&ID No: 33013-1246

RG&E-Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75AJ-AC6-152	CC-230	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	1.5E TO 727K & L
.75AK-AC6-152	CC-400	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3I TO 2745
.75AL-AC6-152	CC-270	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	777N TO SI PUMP 2
.75AM-AC6-152	CC-230	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	TO GAS COMPRESSOR HEX
.75AN-AC6-152	CC-400	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3I TO 2739
.75AO-AC6-152	CC-270	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	777M TO SI PUMP 2
.75AP-AC6-152	CC-450	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3O TO 2736
.75AQ-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	SI PUMP 2 TO 777P
.75AR-AC6-152	CC-400	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 1.5B TO 2744
.75AS-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	SI PUMP 2 TO 777G
.75AT-AC6-152	CC-450	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 4E TO 2743
.75AU-AC6-152	CC-270	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	777S TO SI PUMP 3
.75AV-AC6-152	CC-450	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4G TO 2735
.75AW-AC6-152	CC-270	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	777J TO SI PUMP 3
.75AX-AC6-152	CC-400	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	1.5B TO 765A & B
.75AY-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	SI PUMP 3 TO 777R
.75AZ-AC6-152	CC-330	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2A TO 2776
.75B-AC6-152	CC-250	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2B TO CS PUMP 1
.75BA-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	SI PUMP 3 TO 777K
.75BB-AC6-152	CC-220	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2I TO 744
.75BC-AC6-152	CC-320	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2K TO 727F & G
.75BD-AC6-152	CC-500	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 6A TO 2733
.75BE-AC6-152	CC-310	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 2E TO 2716
.75BF-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4G TO 2769
.75BG-AC6-152	CC-310	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 2E TO 2720
.75BH-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4I TO 2766
.75BI-AC6-152	CC-310	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2E TO 766
.75BJ-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4I TO 2767
.75BK-AC6-152	CC-310	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4A TO 776

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY COOLANT
P&ID No: 33013-1246

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75BL-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4J TO 2768
.75BM-AC6-152	CC-160	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4A TO 2718
.75BN-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4H TO 2751
.75BO-AC6-152	CC-160	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4A TO 727R & S
.75BP-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2754 TO 4H
.75BQ-AC6-152	CC-160	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	6B TO 2722
.75BR-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	6B TO 818
.75BS-AC6-152	CC-310	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 4A TO 2721
.75BT-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	6B TO 2734
.75BU-AC6-152	CC-260	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	CS PUMP 1 TO 2C
.75BV-AC6-152	CC-220	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	6D TO 2725
.75BW-AC6-152	CC-260	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	CS PUMP 2 TO 2C
.75C-AC6-152	CC-300	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	6G TO 774B
.75D-AC6-152	CC-220	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	6B TO 736A & B
.75E-AC6-152	CC-220	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2I TO 2727
.75F-AC6-152	CC-220	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3G TO 2729
.75G-AC6-152	CC-700	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 4D
.75H-AC6-152	CC-700	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	1.5A TO 755A
.75I-AC6-152	CC-700	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 1.5A TO 2773
.75J-AC6-152	CC-700	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 1A TO 2759
.75K-AC6-152	CC-600	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3D TO 2774
.75L-AC6-152	CC-600	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4B TO 2757
.75M-AC6-152	CC-600	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4B TO 2772
.75N-AC6-152	CC-600	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 3C TO 2755
.75O-AC6-152	CC-400	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	1.5B TO 755B
.75P-AC6-152	CC-220	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	3H TO 2731
.75Q-AC6-152	CC-525	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	4L TO 2749
.75R-AC6-152	CC-700	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 4E TO 2775
.75S-AC6-152	CC-700	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	VENT FROM 4C TO 2758

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY COOLANT
P&ID No: 33013-1246

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75T-AC6-152	CC-200	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 817 TO 2709
.75U-AC6-152	CC-250	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2B TO CS PUMP 2
.75V-AC6-152	CC-300	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 2D TO 2710
.75W-AC6-152	CC-200	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 6A TO 2723
.75X-AC6-152	CC-260	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2B TO 727D & E
.75Y-AC6-152	CC-300	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	2D TO 774C
.75Z-AC6-152	CC-330	N/A		A53	0.75	0.113	IWD-1220.1		<150	<200	?	DRAIN FROM 3A TO 2730

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY FEEDWATER
P&ID No: 33013-1237

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
5A-FW-900-1	AFW-200		C-1	A106	5.00	0.375		VT-3	1250	100	?	AUX FW PUMP DISCHARGE TO 3998
5B-FW-900-1	AFW-200		C-1A	A106	5.00	0.375		VT-3	1250	100	?	3F TO REDUCER AT 4003
5C-FW-900-1	AFW-400,200			A106	5.00	0.375		VT-3	1250	100	?	3H TO 3J
4A-CD-150-1					4.00	0.237	IWD-1220.1				?	SERVICE WATER TO AUX FW PUMP
4A-CD-150-1A	SW-1520		C-33	A53	4.00	0.237	IWD-1220.1		12	80	?	4017 TO AUX FW PUMP A SUCTION
4A-CD-150-1B	SW-1520		C-33	A53	4.00	0.237	IWD-1220.1	VT-3	12	80	?	4016 TO AUX FW PUMP B SUCTION
4A-SW-125-1A			C-33	A53	4.00	0.237	IWD-1220.1		75	80	?	STATION SERVICE WATER TO AFW PUMP A
4A-SW-125-1B					4.00	0.237	IWD-1220.1				?	SER WATER TO 4013, 4098 TO 4A-CD-150-1
4C-CD-150-1	SW-1520		C-33	A53	4.00	0.237	IWD-1220.1		12	80	?	4A TO 4027 AND 4A-SW-125-1A
4D-SW-125-1A			C-33	A53	4.00	0.237	IWD-1220.1		75	80	?	4028 TO 4344
4E-CD-150-1	SW-1520		C-33	A53	4.00	0.237	IWD-1220.1		12	80	?	4014 TO 4024
3A-FW-900-1A	AFW-100,500		C-1A	A106	3.00	0.300		VT-3	1250	100	?	AUX FW PUMP A DISCHARGE TO 4000C
3A-FW-900-1B	AFW-100,400		C-1F	A106	3.00	0.300		VT-3	1250	100	?	AUX FW PUMP B DISCHARGE TO 4000D
3C-FW-900-1	AFW-100		C-1D	A106	3.00	0.300		VT-3	1250	100	?	3A TO 3A-FW-900-1B, 4356,7, 4000A&B
3D-FW-900-1	AFW-100				3.00	0.438		VT-3	1250	100	?	BYPASS BETWEEN 4357 AND 4356
3E-FW-900-1	AFW-100		C-1D	A106	3.00	0.438		VT-3	1250	100	?	3C TO 5A-FW-900-1
3F-FW-900-1	AFW-200		C-1	A106	3.00	0.300		VT-3	1250	100	?	5A TO 4001
3G-FW-900-1	AFW-200,500		C-1A	A106	3.00	0.300		VT-3	1250	100	?	5B TO 4003
3H-FW-900-1	AFW-200			A106	3.00	0.300		VT-3	1250	100	?	5A TO 5C, 4000, 4002
3J-FW-900-1	AFW-200			A106	3.00	0.438		VT-3	1250	100	?	5C TO 4004
2A-FW-900-1			C-1	A106	2.00	0.145		VT-3	1250	100	?	5A TO 4291 AND 4290A
2A-FW-900-1A	AFW-100		C-1G	A106	2.00	0.145		VT-3	1250	100	?	3A TO 4304
2B-FW-900-1A	AFW-100		C-1G	A106	2.00	0.145		VT-3	1250	100	?	2A TO 4482, 4480, 4483
2B-FW-900-1B			C-1H		2.00	0.145		VT-3	1250	100	?	2A TO 4484, 4481, 4485
2A-FW-900-1B	AFW-100		C-1H	A106	1.50	0.200		VT-3	1250	100	?	3A TO 4310
1.25A-SW-125-1A			C-33	A53	1.25	0.133	IWD-1220.1		75	80	?	4A TO 4091
1.25B-SW-125-1A			C-33	A53	1.25	0.133	IWD-1220.1		75	80	?	4A TO .5B
1A-CD-150-1A	SW-1520		C-33	A53	1.00	0.133	IWD-1220.1		12	80	?	DRAIN FROM 4C TO 4343
1A-SW-125-1A			C-33	A53	1.00	0.133	IWD-1220.1		75	80	?	.5A TO LUBE OIL CLEANER

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: AUXILIARY FEEDWATER
P&ID No: 33013-1237

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
1A-SW-125-1B					1.00	0.133	IWD-1220.1				?	4A BYPASS TO 4292 & 4293
1B-CD-150-1A	SW-1520		C-33	A53	1.00	0.133	IWD-1220.1		12	80	?	4083 TO 4294, .5A
1B-SW-125-1A			C-33	A53	1.00	0.133	IWD-1220.1		75	80	?	.5B TO LUBE OIL CLEANER
.75A-CD-150-1A	SW-1520		C-33	A53	0.75	0.113	IWD-1220.1		12	80	?	4A TO 4026 AND 4A-CD-150-1B
.75A-CD-150-1B	SW-1520		C-33	A53	0.75	0.113	IWD-1220.1	VT-3	12	80	?	4A TO 4022
.75A-FW-900-1A	AFW-100		C-1A	A106	0.75	0.113		VT-3	1250	100	?	3A TO 4351A
.75A-FW-900-1B	AFW-100		C-1H	A106	0.75	0.113		VT-3	1250	100	?	3A TO 4350A
.75A-SW-125-1B					0.75	0.113	IWD-1220.1				?	4A TO 4020
.75B-CD-150-1A	SW-1520		C-33	A53	0.75	0.113	IWD-1220.1		12	80	?	4A TO 4021
.75B-CD-150-1B	SW-1520		C-33	A53	0.75	0.113	IWD-1220.1	VT-3	12	80	?	4A TO 4305
.75B-FW-900-1A	AFW-100		C-1A	A106	0.75	0.113		VT-3	1250	100	?	3A TO 4353A
.75B-FW-900-1B	AFW-100		C-1H	A106	0.75	0.113		VT-3	1250	100	?	3A TO 4352A
.75B-SW-125-1B					0.75	0.113	IWD-1220.1				?	.5A TO 4324
.75C-FW-900-1A	AFW-100		C-1A	A106	0.75	0.113		VT-3	1250	100	?	DRAIN FROM 2B TO 4493
.75C-FW-900-1B					0.75	0.113		VT-3	1250	100	?	DRAIN FROM 2B TO 4487
.75D-FW-900-1	AFW-200			A106	0.75	0.113		VT-3	1250	100	?	5C TO 4348
.75E-FW-900-1	AFW-200			A106	0.75	0.113		VT-3	1250	100	?	5C TO 4346
.5A-CD-150-1	SW-1520		C-33	A53	0.50	0.100	IWD-1220.1		12	80	?	.75A TO 4024
.5A-SW-125-1A			C-33	A53	0.50	0.100	IWD-1220.1		75	80	?	1.25A TO AUX FW PUMP A
.5A-SW-125-1B					0.50	0.100	IWD-1220.1				?	4A TO 4089 & AUX PUMP
.5B-SW-125-1A			C-33	A53	0.50	0.100	IWD-1220.1		75	80	?	1.25B TO AUX FW PUMP B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: CHEMICAL & VOLUME CONTROL
P&ID No: 33013-1264

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3A-CH4-151	CVC-1000			A312	3.00	0.438	IWD-1220.1				?	DEBOR DEMIN TANK 2 TO 240
3B-CH4-151	CVC-1000			A312	3.00	0.438	IWD-1220.1				?	DEBOR DEMIN TANK 1 TO 241
3C-CH4-151	CVC-1000			A312	3.00	0.438	IWD-1220.1				?	CATION BED DEMIN TO 220
3D-CH4-151	CVC-1000			A312	3.00	0.438	IWD-1220.1				?	MIXED BED DEMIN 2 TO 213
3E-CH4-151	CVC-1000			A312	3.00	0.438	IWD-1220.1				?	MIXED BED DEMIN 1 TO 224
2A-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	TCV145 TO MIXED BED DEMIN 1 & 2
2B-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	2A TO DEBOR DEMIN 1 & 2
2C-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	MIXED BED DEMIN 1 TO 2275
2D-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	MIXED BED DEMIN 2 TO 2C
2E-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	2C TO CATION BED DEMIN
2F-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	CATION BED DEMIN TO 2C
2G-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	DEBOR DEMIN 1 TO 396
2H-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	DEBOR DEMIN 2 TO 395
2I-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	229 TO 2B
2J-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	DEBOR DEMIN 2 TO 237B, 2K
2K-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	DEBOR DEMIN 1 TO 239B
2L-CH4-151	CVC-1000			A312	2.00	0.145	IWD-1220.1				?	2K TO 236
1A-CH4-151	CVC-1000			A312	1.00	0.133	IWD-1220.1				?	381 TO 2F
1B-CH4-151	CVC-1000			A312	1.00	0.133	IWD-1220.1				?	374 TO 2D
1C-CH4-151	CVC-1000			A312	1.00	0.133	IWD-1220.1				?	376 TO 2C
1D-CH4-151	CVC-1000			A312	1.00	0.133	IWD-1220.1				?	CATION BED DEMIN TO 378
1E-CH4-151	CVC-1000			A312	1.00	0.133	IWD-1220.1				?	MIXED BED DEMIN 2 TO 214
1F-CH4-151	CVC-1000			A312	1.00	0.133	IWD-1220.1				?	MIXED BED DEMIN 1 TO 222

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: CHEMICAL & VOLUME CONTROL
P&ID No: 33013-1266

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-CH6-151				A312	2.00	0.109	IWD-1220.1				?	375C, 331 TO 345
2B-CH6-151				A312	2.00	0.109	IWD-1220.1				?	2A TO BORIC ACID TRANSFER PUMP 1 SUCTION
2C-CH6-151				A312	2.00	0.109	IWD-1220.1				?	2A TO BORIC ACID TRANSFER PUMP 2 SUCTION
2D-CH6-151				A312	2.00	0.109	IWD-1220.1				?	217 TO 2B & 2C
2E-CH6-151				A312	2.00	0.109	IWD-1220.1				?	PUMP 1 DISHCARGE TO 2G
2F-CH6-151				A312	2.00	0.109	IWD-1220.1				?	PUMP 2 DISHCARGE TO 2G
2G-CH6-151				A312	2.00	0.109	IWD-1220.1				?	2E, 2F TO 348A, 1A
2H-CH6-151				A312	2.00	0.109	IWD-1220.1				?	2E TO HCV104, 2G
2I-CH6-151				A312	2.00	0.109	IWD-1220.1				?	2F TO HCV105, 2H
2J-CH6-151				A312	2.00	0.109	IWD-1220.1				?	BORIC ACID BLENDER TO FCV110B, 365A
2K-CH6-151				A312	2.00	0.109	IWD-1220.1				?	BORIC ACID BLENDER TO FCV111, 223B
1A-CH6-151				A312	1.00	0.109	IWD-1220.1				?	2G TO 356 BORIC ACID BLENDER
.75A-CH6-151				A312	0.75	0.083	IWD-1220.1				?	2F TO 344
.75B-CH6-151				A312	0.75	0.083	IWD-1220.1				?	2E TO 330D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: CHEMICAL & VOLUME CONTROL
P&ID No: 33013-1268

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-CH8-151				A304	2.00	0.109	IWD-1220.1				?	1231, 1230 TO POLISHING DEMIN
1A-CH8-151				A304	1.00	0.109	IWD-1220.1				?	1141B TO HOLDUP TANKS

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: CHEMICAL VOLUME & CONTROL
P&ID No: 33013-1267

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
6A-CH7-151				A53	6.00	0.280		VT-3			?	4B, 4C, 4D TO 4E
4A-CH7-151				A53	4.00	0.237	IWD-1220.1				?	1100A, 1103A TO HOLDUP TANKS A,B,C
4B-CH7-151				A53	4.00	0.237	IWD-1220.1				?	HOLDUP TANK A TO 6A, 1127
4C-CH7-151				A53	4.00	0.237	IWD-1220.1				?	HOLDUP TANK B TO 1126, 6A
4D-CH7-151				A53	4.00	0.237	IWD-1220.1				?	HOLDUP TANK C TO 1113, 6A
4E-CH7-151				A53	4.00	0.237	IWD-1220.1				?	6A TO RECIRC PUMP SUCTION
4F-CH7-151				A53	4.00	0.237	IWD-1220.1				?	RECIRC PUMP DISCHARGE TO 4A
3A-CH7-151				A53	3.00	0.216	IWD-1220.1				?	HOLDUP TANK B TO 1266
3B-CH7-151				A53	3.00	0.216	IWD-1220.1				?	HOLDUP TANK C TO 1265
3C-CH7-151				A53	3.00	0.216	IWD-1220.1				?	4A, 1100B TO 3D
3D-CH7-151				A53	3.00	0.216	IWD-1220.1				?	4A, 1100C TO 1100D
3E-CH7-151				A53	3.00	0.216	IWD-1220.1				?	ION EXCHANGER TO 1164
3F-CH7-151				A53	3.00	0.216	IWD-1220.1				?	ION EXCHANGER TO 1172
3G-CH7-151				A53	3.00	0.216	IWD-1220.1				?	ION EXCHANGER TO 1184
3H-CH7-151				A53	3.00	0.216	IWD-1220.1				?	ION EXCHANGER TO 1189
2A-CH7-151				A53	2.00	0.154	IWD-1220.1				?	HOLDUP TANKS A,B,C TO VENT HEADER
2B-CH7-151				A53	2.00	0.154	IWD-1220.1				?	2A TO 1123C
2C-CH7-151				A53	2.00	0.154	IWD-1220.1				?	2A TO 1123B
2D-CH7-151				A53	2.00	0.154	IWD-1220.1				?	2A TO 1123A
2E-CH7-151				A53	2.00	0.154	IWD-1220.1				?	HOLDUP TANK A TO 1267
2F-CH7-151				A53	2.00	0.154	IWD-1220.1				?	4A TO 361
2G-CH7-151				A53	2.00	0.154	IWD-1220.1				?	HOLDUP TKS A,B,C TO GAS STPR PMPS A & B
2H-CH7-151				A53	2.00	0.154	IWD-1220.1				?	DEMIN TO ION EXCHANGERS
2I-CH7-151				A53	2.00	0.154	IWD-1220.1				?	CATION ION EXCH 2 TO 1163
2J-CH7-151				A53	2.00	0.154	IWD-1220.1				?	CATION ION EXCH 1 TO 1170
2K-CH7-151				A53	2.00	0.154	IWD-1220.1				?	BASE REMOVAL ION EXCH 2 TO 1183
2L-CH7-151				A53	2.00	0.154	IWD-1220.1				?	BASE REMOVAL ION EXCH 1 TO 1186
2M-CH7-151				A53	2.00	0.154	IWD-1220.1				?	HOLDUP TANK A TO 1272
2N-CH7-151				A53	2.00	0.154	IWD-1220.1				?	HOLDUP TANK B TO 1271

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: CHEMICAL VOLUME & CONTROL
P&ID No: 33013-1267

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2D-CH7-151					2.00	0.154	IWD-1220.1				?	HOLDUP TANK C TO 1270
1A-CH7-151					1.00	0.133	IWD-1220.1				?	4A, 1121C THRU 1121A
1B-CH7-151					1.00	0.133	IWD-1220.1				?	4A, 1121B TO 1A
1C-CH7-151					1.00	0.133	IWD-1220.1				?	GAS STPR PMP A & B TO ION EX
1D-CH7-151					1.00	0.133	IWD-1220.1				?	1E, 1181B TO 1B
1E-CH7-151					1.00	0.133	IWD-1220.1				?	ION EX TO GAS STRIPPER PKG
.75A-CH7-151					0.75	0.113	IWD-1220.1				?	1E TO 1159A
.75B-CH7-151					0.75	0.113	IWD-1220.1				?	1E TO 1180
.75C-CH7-151					0.75	0.113	IWD-1220.1				?	GAS STRIPPER PACKAGE TO GAS ANALYZER

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: DIESEL GENERATORS
P&ID No: 33013-1239

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
20A-DG-A			N/A	A53	20.00	0.250		VT-3			?	AIR INTAKE DG 1A (NO SUPPORTS)
20A-DG-B			N/A	A53	20.00	0.250		VT-3			?	AIR INTAKE DG 1B (NO SUPPORTS)
20B-DG-A			N/A	A53	20.00	0.250		VT-3			?	AIR EXHAUST DG 1A (NO SUPPORTS)
20B-DG-B			N/A	A53	20.00	0.250		VT-3			?	AIR EXHAUST DG 1B (NO SUPPORTS)
3A-FO-150-6B				A53	3.00	0.438	IWD-1220.1				?	2C TO OIL TRANSFER PUMP
3A-FO-153-A				A53	3.00	0.216	IWD-1220.1				?	DIESEL OIL STORAGE TANK TO 5955
3B-FO-150-6B				A53	3.00	0.216	IWD-1220.1				?	DIESEL OIL STORAGE TANK TO 2C
3B-FO-153-A				A53	3.00	0.216	IWD-1220.1				?	2D TO OIL TRANSFER PUMP
2A-FO-150-6A				A53	2.00	0.145	IWD-1220.1				?	5960A TO OIL STORAGE TANK
2A-FO-150-6B				A53	2.00	0.145	IWD-1220.1				?	1.5B TO DIESEL OIL STORAGE TANK B
2A-SA-150-4-A				A53	2.00	0.145	IWD-1220.1				?	5973 TO 5971
2A-SA-150-4B				A53	2.00	0.145	IWD-1220.1				?	AIR RECEIVER TO 5946A
2B-FO-150-6A				A53	2.00	0.145	IWD-1220.1				?	OIL PUMP TO 5963; .75A
2B-FO-150-6B				A53	2.00	0.145	IWD-1220.1				?	OIL TRANSFER PUMP TO 5908
2C-FO-150-6A				A53	2.00	0.145	IWD-1220.1				?	2A TO 5967 & 2A-FO-150-6B
2C-FO-150-6B				A53	2.00	0.145	IWD-1220.1				?	3B TO 3A
2D-FO-153-A				A53	2.00	0.145	IWD-1220.1				?	3A TO 3B
1.5A-FO-150-6A				A53	1.50	0.133	IWD-1220.1				?	DAY TANK A TO 5960A
1.5A-FO-150-6B				A53	1.50	0.133	IWD-1220.1				?	DAY TANK TO 5978C
1.5A-SA-150-4-A				A53	1.50	0.133	IWD-1220.1				?	AIR RECEIVER TO 2A
1.5A-SA-150-4B				A53	1.50	0.133	IWD-1220.1				?	AIR RECEIVER TO 2A-SA-150-4A
1.5A-SW-125-1A				A53	1.50	0.133	IWD-1220.1				?	5900A TO COOLING WATER EXPANSION TANK
1.5A-SW-125-1B				A53	1.50	0.133	IWD-1220.1				?	5900B TO COOLING WATER EXPANSION TANK
1.5B-FO-150-6A				A53	1.50	0.133	IWD-1220.1				?	1.25A TO .75A
1.5B-FO-150-6B				A53	1.50	0.133	IWD-1220.1				?	DAY TANK TO 5960B
1.5B-SA-150-4B				A53	1.50	0.133	IWD-1220.1				?	AIR RECEIVER TO 1.5A
1.5C-FO-150-6B				A53	1.50	0.133	IWD-1220.1				?	5908 TO 1.25A
1.5C-FO-153-A				A53	1.50	0.133	IWD-1220.1				?	5913 TO 5909 & 3A
1.5C-SA-150-4-A				A53	1.50	0.133	IWD-1220.1				?	.5A TO DIESEL GENERATOR 1

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: DIESEL GENERATORS
P&ID No: 33013-1239

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exception Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
1.25A-FO-150-6A					1.25	0.133	IWD-1220.1				?	DAY TANK A TO 1.5B
1.25A-FO-150-6B					1.25	0.133	IWD-1220.1				?	1.5C TO DAY TANK
1A-FO-150-6A					1.00	0.133	IWD-1220.1				?	BYPASS FROM 1.5B TO .75A
1A-FO-150-6B					1.00	0.133	IWD-1220.1				?	1.5A TO 2A
1A-SA-150-4B					1.00	0.133	IWD-1220.1				?	5946A TO 5942A, 5942B
1B-FO-150-6A					1.00	0.133	IWD-1220.1				?	DAY TANK TO 2A
1B-FO-150-6B					1.00	0.133	IWD-1220.1				?	1.5C TO 5938
.75A-FO-150-6A					0.75	0.113	IWD-1220.1				?	1.5A TO 5907, 5965 TO 2B
.75A-FO-150-6B					0.75	0.113	IWD-1220.1				?	2C TO 2B
.75A-SA-150-4B					0.75	0.113	IWD-1220.1				?	1A TO 5944A
.75A-SW-125-1A					0.75	0.113	IWD-1220.1				?	SER WATER TO 5939 CLG WATER EXPANSION TK
.75A-SW-125-1B					0.75	0.113	IWD-1220.1				?	SER WATER TO CLG WATER EXPANSION TANK
.75B-FO-153-A					0.75	0.113	IWD-1220.1				?	.75A TO 5907A TO 2C
.75B-SA-150-4B					0.75	0.113	IWD-1220.1				?	1A TO INSTRUMENTATION
.75B-SW-125-1A					0.75	0.113	IWD-1220.1				?	COOLING WATER EXPANSION TANK TO DG 1A
.75B-SW-125-1B					0.75	0.113	IWD-1220.1				?	COOLING WATER EXPANSION TANK TO DG 1B
.75C-SA-150-4B					0.75	0.113	IWD-1220.1				?	AIR RECEIVER TO 5948C
.75D-SA-150-4B					0.75	0.113	IWD-1220.1				?	AIR RECEIVER TO 5970
.5A-SA-150-4-A					0.50	0.113	IWD-1220.1				?	2A TO 1.5C
.375A-FO-153-A					0.37	0.100	IWD-1220.1				?	1B TO INSTRUMENT PANELS
.375A-SW-125-1A					0.37	0.100	IWD-1220.1				?	.75B TO DG 1A
.375A-SW-125-1B					0.37	0.100	IWD-1220.1				?	.75B TO DG 1B

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: HE
System: HIGH ENERGY PIPING: APPEN-B
P&ID No: 33013-1236

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
20-FW-900-1		FW-1001	HE-5		20.00	0.000		SUR/VOL	715	505	Y	TEE BETWEEN V 3985 & 3984 & 3982 & 3983.
14A-FW-900-1A	FW-100	FW-1001	HE-3	A106	14.00	0.938		SUR/VOL	715	505	Y	VALVE 3993 TO S/G-1A.
14B-FW-900-1B	FW-200	FW-1005	HE-4	A106	14.00	0.938		SUR/VOL	715	505	Y	VALVE 3992 TO S/G-1B.
8-FW-900-1		FW-1081	HE-5		8.00	0.000		SUR/VOL	715	505	Y	20" FW BYPASS TO 1B CONDENSER AT V 9507D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: HE
System: HIGH ENERGY PIPING: APPEN-B
P&ID No: 33013-1232

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
24A-HS-600-1A		HS-1002	HE-7		24.00	0.000		SUR/VOL	715	505	Y	36" HS PIPE TO VALVE 3544.
24B-HS-600-1A		HS-1003	HE-7		24.00	0.000		SUR/VOL	715	505	Y	36" HS PIPE TO VALVE 3545.

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: HE
System: HIGH ENERGY PIPING: APPEN-B
P&ID No: 33013-1231

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
36-HS-600-1		HS-1000	HE-1		36.00	0.000		SUR/VOL	715	505	Y	TEE BETWEEN VAL 3519 & 3518 TO END CAP.
36-HS-600-1		SHS-1002	HE-2		36.00	0.000		SUR/VOL	715	505	Y	BETWEEN VAL 3518 & 3519 TO END CAP.
30A-HS-600-1A	HS-100	HS-1000	HE-1	A115	30.00	1.250		SUR/VOL	715	505	Y	S/G 1A TO VALVE 3517.
30B-HS-600-1B	HS-200	SHS-1001	HE-2	A115	30.00	1.250		SUR/VOL	715	505	Y	S/G 1B TO VALVE 3516.
6A-HS-600-1A	HS-100	HS-1000	HE-1	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3505A.
6A-HS-600-1B	HS-300	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3504A.
6B-HS-600-1A	HS-300	HS-1000	HE-1	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3411.
6B-HS-600-1B	HS-200	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3410.
6C-HS-600-1A	HS-300	HS-1000	HE-1	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3509.
6C-HS-600-1B	HS-300	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3508.
6D-HS-600-1A	HS-300	HS-1000	HE-1	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3511.
6D-HS-600-1B	HS-300	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3510.
6E-HS-600-1A	HS-300	HS-1000	HE-1	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3513.
6E-HS-600-1B	HS-300	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3512.
6F-HS-600-1A	HS-300	HS-1000	HE-1	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3515.
6F-HS-600-1B	HS-300	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3514.
6G-HS-600-1B	HS-300	SHS-1001	HE-2	A106	6.00	0.432		SUR/VOL	715	505	Y	30" HS PIPE TO VALVE 3520.

42-52D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: WASTE DISPOSAL LIQUID
P&ID No: 33013-1276

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-WD-151					2.00	0.154	IWD-1220.1				?	VALVE 9133 TO SPENT RESIN TANK.
2B-WD-151					2.00	0.154	IWD-1220.1				?	VALVE 9143 TO 2A-WD-151

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: WASTE DISPOSAL
P&ID No: 33013-1273

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-WD3-1A					2.00	0.145	IWD-1220.1				?	1614 TO GAS COMP PUMP SUCTION A & B
2B-WD3-1A					2.00	0.344	IWD-1220.1				?	1269 TO COMP A & B
2C-WD3-1A					2.00	0.344	IWD-1220.1				?	1632 TO 1679
2D-WD3-1A					2.00	0.145	IWD-1220.1				?	2A TO 1050
2E-WD3-1A					2.00	0.145	IWD-1220.1				?	GAS ANALYZER TO 1051
1.25A-WD3-1A					1.25	0.133	IWD-1220.1				?	GAS COMP PMP DISCHG TO WASTE GAS COMP A
1.25B-WD3-1A					1.25	0.133	IWD-1220.1				?	PUMP B TO WASTE GAS COMP B
1A-WD3-1A					1.00	0.133	IWD-1220.1				?	GAS CMP A TO GAS CMP B & WASTE DECAY TKS
1B-WD3-1A					1.00	0.133	IWD-1220.1				?	2B TO TANK A, 1028
1C-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 1 TO 1036B
1D-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 2 TO 1037B
1E-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 3 TO 1038B
1F-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 4 TO 1039B
1G-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 1 TO 1645A
1H-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 2 TO 1646A
1I-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 3 TO 1647A
1J-WD3-1A					1.00	0.133	IWD-1220.1				?	DECAY TANK 4 TO 1648A
1K-WD3-1A					1.00	0.133	IWD-1220.1				?	1638 TO 1H
1L-WD3-1A					1.00	0.133	IWD-1220.1				?	1637 TO 1G
1M-WD3-1A					1.00	0.133	IWD-1220.1				?	1640 TO 1J
1N-WD3-1A					1.00	0.133	IWD-1220.1				?	1639 TO 1I
.75A-WD3-1A					0.75	0.113	IWD-1220.1				?	1668 TO 1664B
.75B-WD3-1A					0.75	0.113	IWD-1220.1				?	TANK A TO 1030B
.75C-WD3-1A					0.75	0.113	IWD-1220.1				?	TANK A TO 1610C
.75D-WD3-1A					0.75	0.113	IWD-1220.1				?	2E TO 1050
.75E-WD3-1A					0.75	0.113	IWD-1220.1				?	B TANK TO 1032B
.75F-WD3-1A					0.75	0.113	IWD-1220.1				?	B TANK TO 1610D
.75G-WD3-1A					0.75	0.113	IWD-1220.1				?	.75A TO 1665A
.375A-WD3-1A					0.37	0.100	IWD-1220.1				?	2D TO 1051

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: WASTE DISPOSAL
P&ID No: 33013-1272

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3-WD2-151					3.00	0.216	IWD-1220.1				?	FUEL TRANSFER DRAIN TO 1795G, 1722

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: WASTE DISPOSAL
P&ID No: 33013-1271

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-AC1-152-A					2.00	0.145	IWD-1220.1				?	COMPONENT COOLING TO 775D
2A-WD1-1A					2.00	0.145	IWD-1220.1				?	FEED TANK TO WHT
2B-AC1-152-A					2.00	0.145	IWD-1220.1				?	CONCENTRATOR TO 764B COMPONENT COOLING
2C-AC1-152-A					2.00	0.145	IWD-1220.1				?	2A TO DISTILLATE COOLER HE
2D-AC1-152-A					2.00	0.145	IWD-1220.1				?	DISTILLATE COOLER HE TO 2B, 2628
1A-AC1-152-A					1.00	0.133	IWD-1220.1				?	2D TO 2627A
1A-WD1-1A					1.00	0.133	IWD-1220.1				?	FEED TANK TO FEED TANK PUMP SUCTION
1B-WD1-1A					1.00	0.133	IWD-1220.1				?	FEED TANK TO FEED TANK PUMP DISCHARGE
1C-WD1-1A					1.00	0.133	IWD-1220.1				?	CONCENTRATOR LEVEL CNTL TO CONCENTRATOR
1D-WD1-1A					1.00	0.133	IWD-1220.1				?	CONCENTRATOR LEVEL CNTL TO CONCENTRATOR
1E-WD1-1A					1.00	0.133	IWD-1220.1				?	FEED TANK DISCHARGE TO CONCENTRATOR
1F-WD1-1A					1.00	0.133	IWD-1220.1				?	CONCENTRATOR LEVEL CNTL TO CONCENTRATOR
.75A-WD1-1A					0.75	0.113	IWD-1220.1				?	1654B TO FEED TANK PUMP SUCTION
.75B-WD1-1A					0.75	0.113	IWD-1220.1				?	1E TO 2616

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: WASTE DISPOSAL
P&ID No: 33013-1270

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3A-WDO-301B					3.00	0.216	IWD-1220.1				?	1687 TO SPENT RESIN STORAGE TANK 2
2A-WDO-151					2.00	0.145	IWD-1220.1				?	WST HOLDUP TK TO 1792, 1610A & WE FD PMP
2A-WDO-151A					2.00	0.145	IWD-1220.1				?	SPENT RESIN STORAGE TANK 1 TO 1704
2A-WDO-301A					2.00	0.145	IWD-1220.1				?	1684 TO SPENT RESIN STORAGE TANK 1
2A-WDO-301B					2.00	0.145	IWD-1220.1				?	1689 TO SPENT RESIN STORAGE TANK 2
2B-WDO-151					2.00	0.145	IWD-1220.1				?	WASTE EVAP FEED PUMP TO 1769 FEED TANK
2B-WDO-151B					2.00	0.145	IWD-1220.1				?	SPENT RESIN STORAGE TANK 2 TO 1705
2B-WDO-301A					2.00	0.145	IWD-1220.1				?	1688 TO SPENT RESIN STORAGE TANK 1
1A-WDO-151					1.00	0.133	IWD-1220.1				?	2B TO SUMP TANK 1654N
1A-WDO-151A					1.00	0.133	IWD-1220.1				?	SPENT RESIN STORAGE TANK 1 TO 1700
1A-WDO-151B					1.00	0.133	IWD-1220.1				?	SPENT RESIN STORAGE TANK 2 TO 1702
1A-WDO-301A					1.00	0.133	IWD-1220.1				?	1692 TO 2B
1A-WDO-301B					1.00	0.133	IWD-1220.1				?	1693 TO 2A
.75A-WDO-301A					0.75	0.113	IWD-1220.1				?	1696, 1800, 1697 TO SPNT RESIN STOR TK 1
.75A-WDO-301B					0.75	0.113	IWD-1220.1				?	1698, 1699 TO SPENT RESIN STORAGE TANK 2

42-52D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: STANDBY AUXILIARY FEEDWATER
P&ID No: 33013-1238

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.5A-FWS-902-A			N/A	A106	0.50	0.109		VT-3			?	.75D TO 9713A
.5A-FWS-902-B			N/A	A106	0.50	0.109		VT-3			?	.75D TO 9713B
.5B-FWS-902-A			N/A	A106	0.50	0.109		VT-3			?	3A TO 9715A TO 9714A
.5B-FWS-902-B			N/A	A106	0.50	0.109		VT-3			?	.75G TO 9714B, 9715B
.5C-FWS-902-A			N/A	A106	0.50	0.109		VT-3			?	.75H TO 9712B
.5D-FWS-902-A			N/A	A106	0.50	0.109		VT-3			?	.75H TO 9712A
.375A-FWS-152-A			N/A	A106	0.12	0.100		VT-3			?	4A TO 9743

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: STANDBY AUXILIARY FEEDWATER
P&ID No: 33013-1238

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
.75D-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 4A TO 9640D
.75D-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	3A TO .5A
.75D-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	3A TO .5A
.75E-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9630B
.75E-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9717A
.75E-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	3A TO 9717B
.75F-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9716B
.75F-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A TO 9718A
.75F-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	3A TO 9718B
.75G-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 4A TO 9742
.75G-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9731
.75G-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	3A TO .5B
.75H-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	4A TO 9744
.75H-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A TO 9730
.75H-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	3A TO .5B
.75I-CD-150-1			N/A	A106	0.75	0.154		VT-3			?	9720B TO 4B
.75I-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A
.75I-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9729
.75I-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9732
.75J-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A TO 9739
.75J-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A TO 9738
.75K-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9740
.75K-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9737
.75L-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A TO 9722A
.75L-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A TO 9722B
.75M-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	3B TO .5C
.75N-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	3B TO .5D
.5A-FW8-152-A			N/A	A106	0.50	0.109		VT-3			?	4A TO 9711A
.5A-FW8-152-B			N/A	A106	0.50	0.109		VT-3			?	4A TO 9711B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: STANDBY AUXILIARY FEEDWATER
P&ID No: 33013-1238

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
4A-CD-150-1			C-28	A106	4.00	0.300		VT-3			?	9707A TO 4A
4A-FW8-152-A			C-26	A106	4.00	0.300		VT-3			?	9626A TO AUX FW PUMP C SUCTION
4A-FW8-152-B			C-27	A106	4.00	0.300		VT-3			?	9626B TO AUX FW PUMP D SUCTION
4B-CD-150-1			C-28	A106	4.00	0.300		VT-3			?	9707B TO 4A
4B-FW8-152			C-26	A106	4.00	0.300		VT-3			?	9627A TO 9627B
3A-FW8-902-A			C-21	A106	3.00	0.300		VT-3			?	AUX FW PUMP C DISCHARGE TO 9704A
3A-FW8-902-B			C-22	A106	3.00	0.300		VT-3			?	AUX FW PUMP DISCHARGE TO 9704B
3C-FW8-902-A			C-23	A106	3.00	0.300		VT-3			?	3A TO 3A-FW8-902-B
1.5A-CD-150-1			C-28	A106	1.50	0.200		VT-3			?	9720A TO 4A
1.5A-FW8-152-A			C-29	A106	1.50	0.200		VT-3			?	4A TO COOLING UNIT A
1.5A-FW8-152-B			C-29	A106	1.50	0.200		VT-3			?	4A TO COOLING UNIT 1B
1.5A-FW8-902-A			N/A	A106	1.50	0.200		VT-3			?	FO PAST 9710A TO 3A
1.5A-FW8-902-B			N/A	A106	1.50	0.200		VT-3			?	3A TO 9710B
1.5B-CD-150-1			C-28	A106	1.50	0.200		VT-3			?	9720B TO 4B
1A-FW8-152-A			N/A	A106	1.00	0.179		VT-3			?	4A TO 9709A
1A-FW8-152-B			N/A	A106	1.00	0.179		VT-3			?	DRAIN FROM 4A TO 9709B
.75A-FW8-152-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9639A
.75A-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9639B
.75A-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A
.75A-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 3A
.75B-FW8-152-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9630A
.75B-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 4A TO 9640B
.75B-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	3A TO 9735
.75B-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	3A TO 9736
.75C-FW8-152-A			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9716A
.75C-FW8-152-B			N/A	A106	0.75	0.154		VT-3			?	DRAIN FROM 4A TO 9640C
.75C-FW8-902-A			N/A	A106	0.75	0.154		VT-3			?	3A TO 9733
.75C-FW8-902-B			N/A	A106	0.75	0.154		VT-3			?	VENT FROM 3A TO 9734
.75D-FW8-152-A			N/A	A106	0.75	0.154		VT-3			?	4A TO 9741

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: SERVICE WATER
P&ID No: 33013-1250

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
2A-SW0-152				A53	2.00	0.154	IWD-1220.1		75	80	?	14F TO 9633A, 9635
2B-SW0-152				A53	2.00	0.154	IWD-1220.1		75	80	?	1.5A TO 20B
1.5A-SW0-125-9				A53	1.50	0.145	IWD-1220.1		75	80	?	3E TO CHARGE PUMP COOLER 1, 3F
1.5B-SW0-125-9				A53	1.50	0.145	IWD-1220.1		75	80	?	ROOM COOLING UNIT 1B TO 9633B
1.5C-SW0-125-9				A53	1.50	0.145	IWD-1220.1		75	80	?	3E TO CHARGE PUMP COOLER 2, 3F
1.5D-SW0-125-9				A53	1.50	0.145	IWD-1220.1		75	80	?	ROOM COOLING UNIT 1A TO 2A
1.5E-SW0-125-9				A53	1.50	0.145	IWD-1220.1		75	80	?	3E TO SI COOLER 2, 3F
1.5F-SW0-125-9				A53	1.50	0.145	IWD-1220.1		75	80	?	3E TO PEN COOLER 3F
1A-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	4A TO 4669A
1B-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	6H TO 4796B
1C-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	14B TO 4667A
1D-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	3E TO 3C, .75A
1E-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	3D TO RHR COOLER 1, 3F
1F-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	3D TO RHR COOLER 2, 3F
1G-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	3D TO SI COOLER 3, 3F
1H-SW0-125-9				A53	1.00	0.133	IWD-1220.1		75	80	?	3D TO SI COOLER 1, 3F
.75A-SW0-125-9				A53	0.75	0.113	IWD-1220.1		75	80	?	1C TO SI PUMPS 3E
.75B-SW0-125-9				A53	0.75	0.113	IWD-1220.1		75	80	?	16C-SW-125-9A TO 14D

42-52D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: SERVICE WATER
P&ID No: 33013-1250

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
8H-SWO-125-9			C-16B	A53	8.00	0.327		VT-3	75	80	?	4627 TO 14E
8I-SWO-125-9			C-17	A53	8.00	0.327		VT-3	75	80	?	20A TO 4609
6A-SWO-125-9	SW-1850		C-13	A53	6.00	0.280		VT-3	75	80	?	16B TO 4B
6B-SWO-125-9	SW-1410		C-18	A53	6.00	0.280		VT-3	75	80	?	4C, 4D TO 14J
6C-SWO-125-9	SW-1020		C-12	A53	6.00	0.280		VT-3	75	80	?	18A TO SFPHE
6D-SWO-125-9	SW-1120		C-11	A53	6.00	0.280		VT-3	75	80	?	6E TO 20C
6E-SWO-125-9	SW-1120		C-11	A53	6.00	0.280		VT-3	75	80	?	S. F. HX TO 20D (DISCHARGE)
6F-SWO-125-9			C-11	A53	6.00	0.280		VT-3	75	80	?	20D TO 8686
6G-SWO-125-9			C-13	A53	6.00	0.280		VT-3	75	80	?	DG COOLERS TO SW RETURNS
6H-SWO-125-9			C-16A	A53	6.00	0.280		VT-3	75	80	?	4642 TO 4733
4A-SWO-125-9	SW-1850			A53	4.00	0.237	IWD-1220.1		75	80	?	14B TO DIESEL GENERATOR 1B
4B-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	6H TO "A" CHILLER
4C-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	DIE GEN 1 TO 4671 & NON-CLASS BOUNDARY
4D-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	6H TO "B" CHILLER
4E-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	DIE GEN 2 TO 4672 & NON-CLASS BOUNDARY
4F-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	14I TO 4617A
4G-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	14H TO 4618A
4H-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	"A" CHILLER TO 4651B
4I-SWO-125-9				A53	4.00	0.237	IWD-1220.1		75	80	?	"B" CHILLER TO 4652B
3A-SWO-125-9	SW-1850			A53	3.00	0.216	IWD-1220.1		75	80	?	4A TO 4668F
3B-SWO-125-9				A53	3.00	0.216	IWD-1220.1		75	80	?	3F TO 6D
3C-SWO-125-9				A53	3.00	0.216	IWD-1220.1		75	80	?	14K TO 4667F
3D-SWO-125-9				A53	3.00	0.216	IWD-1220.1		75	80	?	20A TO 3739
3E-SWO-125-9				A53	3.00	0.216	IWD-1220.1		75	80	?	20B TO 4754
3F-SWO-125-9				A53	3.00	0.216	IWD-1220.1		75	80	?	4753 TO 4739A & DISCHARGE CANAL
2.5A-SWO-125-9				A53	2.50	0.203	IWD-1220.1		75	80	?	14E TO 4625, 4626, 16D
2.5B-SWO-125-9				A53	2.50	0.203	IWD-1220.1		75	80	?	2.5A TO 4635, 4757
2.5C-SWO-125-9				A53	2.50	0.203	IWD-1220.1		75	80	?	4636 TO 4I
2.5D-SWO-125-9				A53	2.50	0.203	IWD-1220.1		75	80	?	4758 TO 2.5C

42-52B

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: SERVICE WATER
P&ID No: 33013-1250

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
20A-SWO-125-9			C-17	A53	20.00	0.375		VT-3	75	80	?	14A & B TO 4610
20B-SWO-125-9	SW-1500		C-17	A53	20.00	0.375		VT-3	75	80	?	PUMP C & D TO 4779
20C-SWO-125-9	SW-1120		N/A	A53	20.00	0.375		VT-3	75	80	?	CCHE 1 TO DISCHARGE (BURIED)
20D-SWO-125-9	SW-1100		C-11	A53	20.00	0.375		VT-3	75	80	?	14F TO DISCHARGE
18A-SWO-125-9	SW-1000		C-12	A53	18.00	0.000		VT-3	75	80	?	20A TO 14I
16A-SWO-125-9	SW-1850		C-13	A53	16.00	0.375		VT-3	75	80	?	20A TO 4665 & 14B
16B-SWO-125-9	SW-1850		C-13	A53	16.00	0.375		VT-3	75	80	?	6A TO 20B
16C-SWO-125-9	SW-1500		C-16	A53	16.00	0.375		VT-3	75	80	?	20A TO 20B
16D-SWO-125-9	SW-1500		C-16	A53	16.00	0.375		VT-3	75	80	?	20A TO 14E
14A-SWO-125-9			C-17	A53	14.00	0.375		VT-3	75	80	?	SERVICE WATER PUMP A TO 20A
14B-SWO-125-9			C-17	A53	14.00	0.375		VT-3	75	80	?	SERVICE WATER PUMP B TO 20A
14C-SWO-125-9			C-17	A53	14.00	0.375		VT-3	75	80	?	SERVICE WATER PUMP C TO 20B
14D-SWO-125-9			C-17	A53	14.00	0.375		VT-3	75	80	?	SERVICE WATER PUMP D TO 20B
14E-SWO-125-9	SW-1500		C-16B	A53	14.00	0.375		VT-3	75	80	?	4627 TO 6H
14F-SWO-125-9	SW-1100		C-11	A53	14.00	0.375		VT-3	75	80	?	CCHE 1 TO 20C
14G-SWO-125-9	SW-1100		C-11	A53	14.00	0.375		VT-3	75	80	?	CCHE 2 TO 20C
14H-SWO-125-9	SW-1000		C-12	A53	14.00	0.375		VT-3	75	80	?	20A TO CCHE 2
14I-SWO-125-9	SW-1000		C-12	A53	14.00	0.375		VT-3	75	80	?	18A TO CCHE 1
14J-SWO-125-9	SW-1400		C-14	A53	14.00	0.375		VT-3	75	80	?	4629 TO DISCHARGE CANAL
14K-SWO-125-9	SW-1400		C-13	A53	14.00	0.375		VT-3	75	80	?	4665 TO 4A-SWO-125-9
10A-SWO-125-9	SW-1400		C-14	A53	10.00	0.375		VT-3	75	80	?	8D TO 14J
10B-SWO-125-9			C-13	A53	10.00	0.375		VT-3	75	80	?	14K TO 4613
8A-SWO-125-9	SW-1500		C-16B	A53	8.00	0.322		VT-3	75	80	?	14E TO 4642
8B-SWO-125-9	SW-1500		C-16B	A53	8.00	0.322		VT-3	75	80	?	14E TO 4628
8C-SWO-125-9	SW-1500		C-16B	A53	8.00	0.322		VT-3	75	80	?	14E TO 4641
8D-SWO-125-9	SW-1400		C-14	A53	8.00	0.322		VT-3	75	80	?	4644 TO 10A
8E-SWO-125-9			C-14	A53	8.00	0.327		VT-3	75	80	?	4643 TO 10A
8F-SWO-125-9			C-14	A53	8.00	0.327		VT-3	75	80	?	4630 TO 14D
8G-SWO-125-9			C-14	A53	8.00	0.327		VT-3	75	80	?	4629 TO 14D

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: POST ACCIDENT SAMPLING
P&ID No: 33013-1279

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
3A-AC9-152					3.00	0.216	IWD-1220.1				?	747F TO AUX COOLANT RETURN
1.5A-AC9-152					1.00	0.133	IWD-1220.1				?	AUX COOLANT SUPPLY TO 747E
1A-AC9-152					1.00	0.133	IWD-1220.1				?	747G TO 3A
1B-AC9-152					1.00	0.133	IWD-1220.1				?	747D TO 1.5A

R. E. GINNA NUCLEAR POWER PLANT
Inservice Examination Boundary Line List
Third Inspection Interval

Class: 3
System: MAIN STEAM
P&ID No: 33013-1231

RG&E Line No.	GILBERT Line No.	SWRI Line No.	ISI Fig.	Matrl.	Size (in.)	Thkns (in.)	Exemption Basis	NDE Method	Po	Temp	In?	Line Description/Remarks
6A-MS-600-1	MS-120		C-32	A106	6.00	0.432		VT-3	1085	508	Y	3505A TO 3505B
6B-MS-600-1	MS-120		C-32	A106	6.00	0.432		VT-3	1085	508	Y	3504A TO 3504B
6C-MS-600-1	MS-120		C-32	A106	6.00	0.432		VT-3	1085	508	Y	6A & 6B TO FW PUMP TURBINE
3A-MS-600-1	MS-120		C-32	A106	3.00	0.216	IWD-1220.1		1085	508	Y	6B TO AUX FW PUMP TURBINE
1.5A-MS-600-1	MS-120			A106	1.50	0.145	IWD-1220.1		1085	508	Y	3505C TO 6A
1.5B-MS-600-1	MS-120			A106	1.50	0.145	IWD-1220.1		1085	508	Y	3504C TO 6B

QUALITY ASSURANCE MANUAL GINNA STATION ROCHESTER GAS & ELECTRIC CORPORATION		REV. 0	PAGE 1 OF 1B
	EFFECTIVE DATE: January 1, 1990		
TITLE: ATTACHMENT B APPENDIX B GINNA NUCLEAR POWER STATION INSERVICE INSPECTION PROGRAM FOR THE 1990-1999 INTERVAL		SIGNATURE	DATE
	PREPARED BY:	Michael J Saporito	7-20-89
	QUALITY ASSURANCE REVIEW	C R Anderson	7-20-89
	APPROVED BY:	John F. Smith	7/20/89

ATTACHMENT B

INTRODUCTION

The drawings included in this section are divided into four(4) groups. The four groups are classified as follows:

1) Class 1 Non-exempted Systems
2) Class 2 Non-exempted Systems
3) Class 3 Non-exempted Systems
4) Exempted System Drawings

Each drawing is a "Class Boundary" drawing that have been used to identify lines in the preparation of "Line Lists" as shown in Attachment A. The first three groups identify Class 1, 2 and 3 Systems that have lines and/or systems requiring examination as indicated on the line list. These drawings identify examined lines by incorporating a color code for Class 1, 2 and 3 Non-Exempted Systems.

The color code for examined line for Class 1 is blue; Class 2 is red and Class 3 is green. Exempted lines and systems shown in the fourth group are not colored for easy reference.

ATTACHMENT B

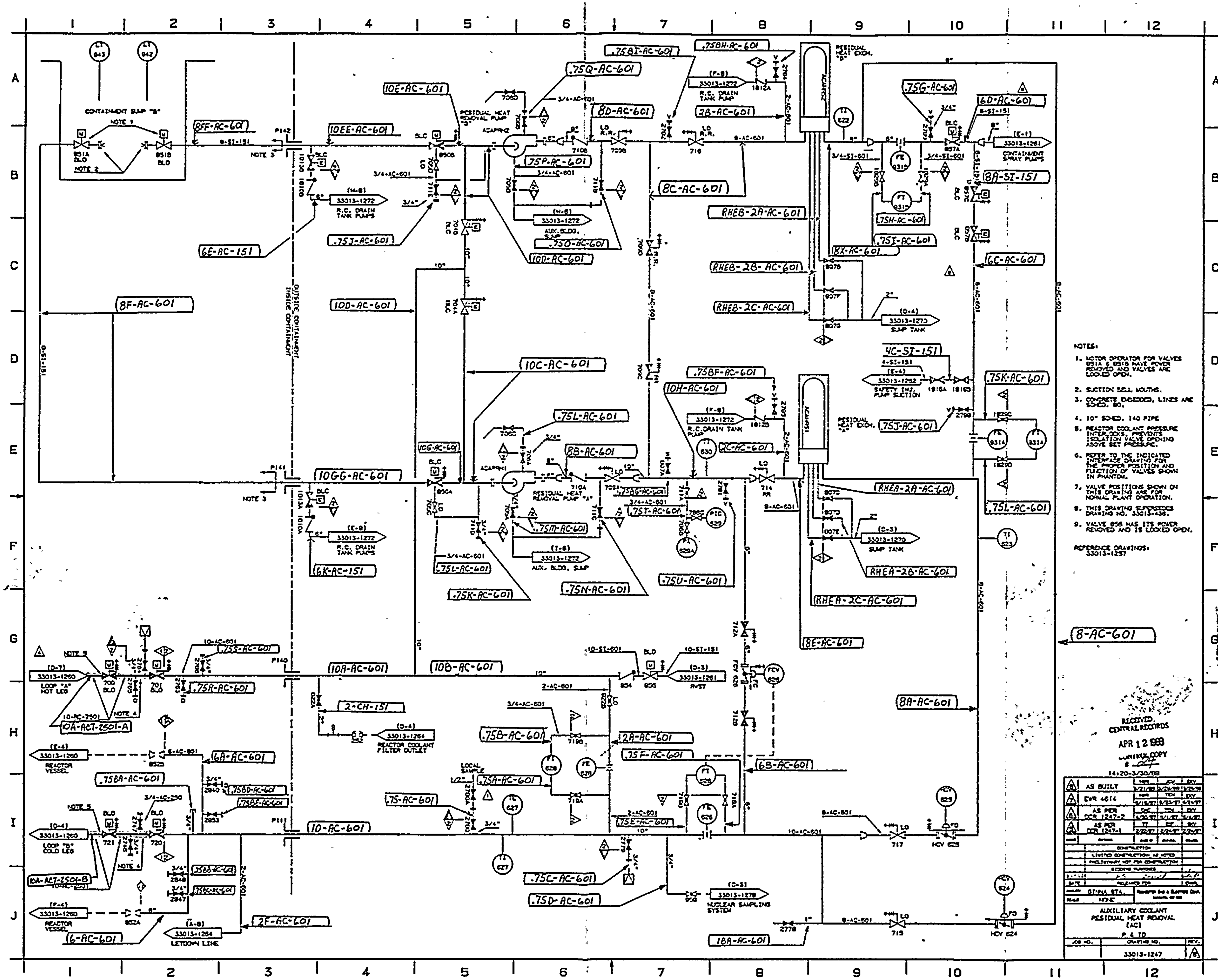
Class 1 Non-Exempted System Drawings

Introduction

All Systems/Lines that require examination by Code Requirements or Rochester Gas and Electric are indicated in blue overlay.

The following Class 1 Non-Exempted P&ID Drawings are included after this listing:

33013-1247	Rev.8
33013-1258	Rev.3
33013-1260	Rev.6
33013-1262	Rev.9
33013-1264	Rev.7
33013-1265	Rev.7



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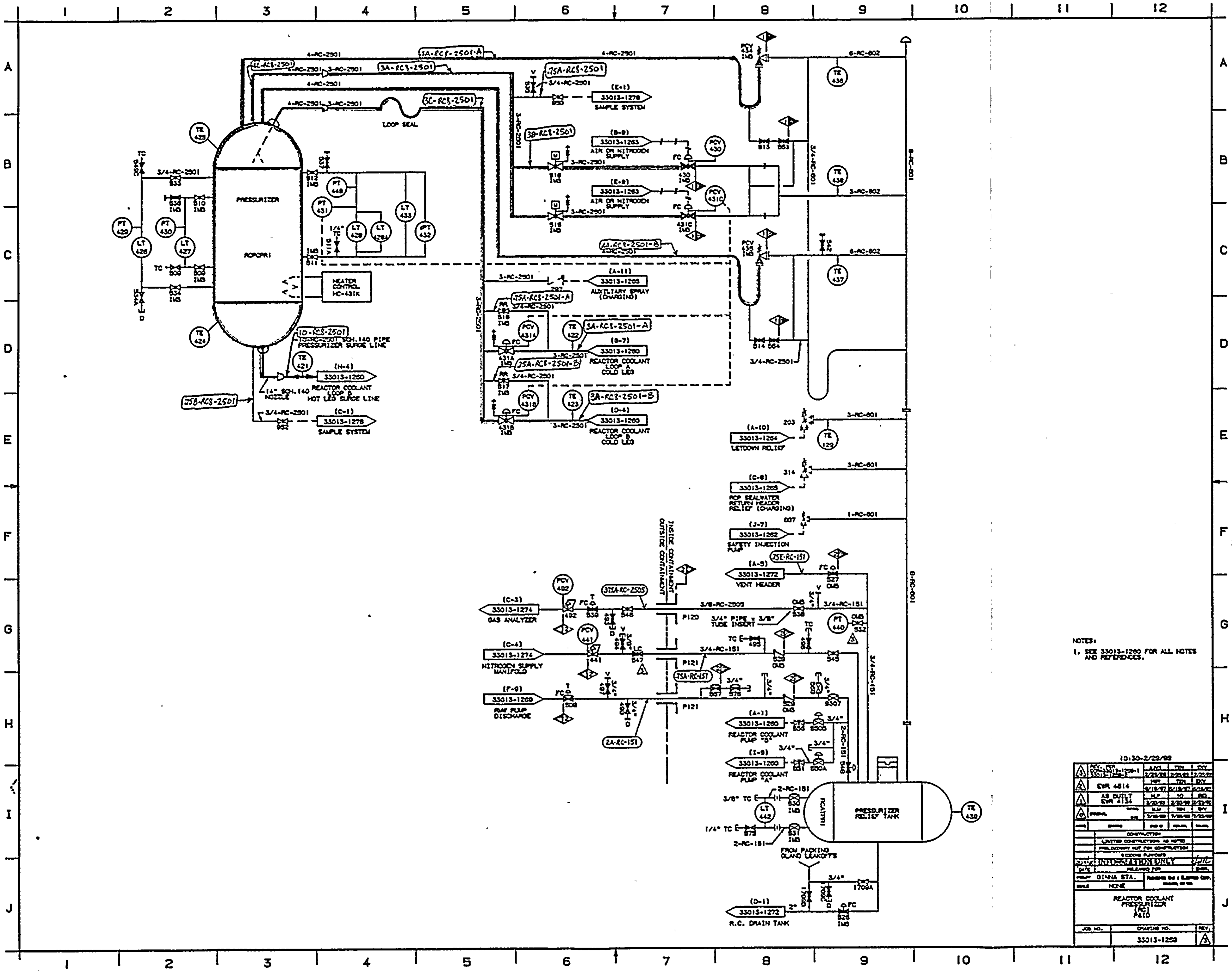

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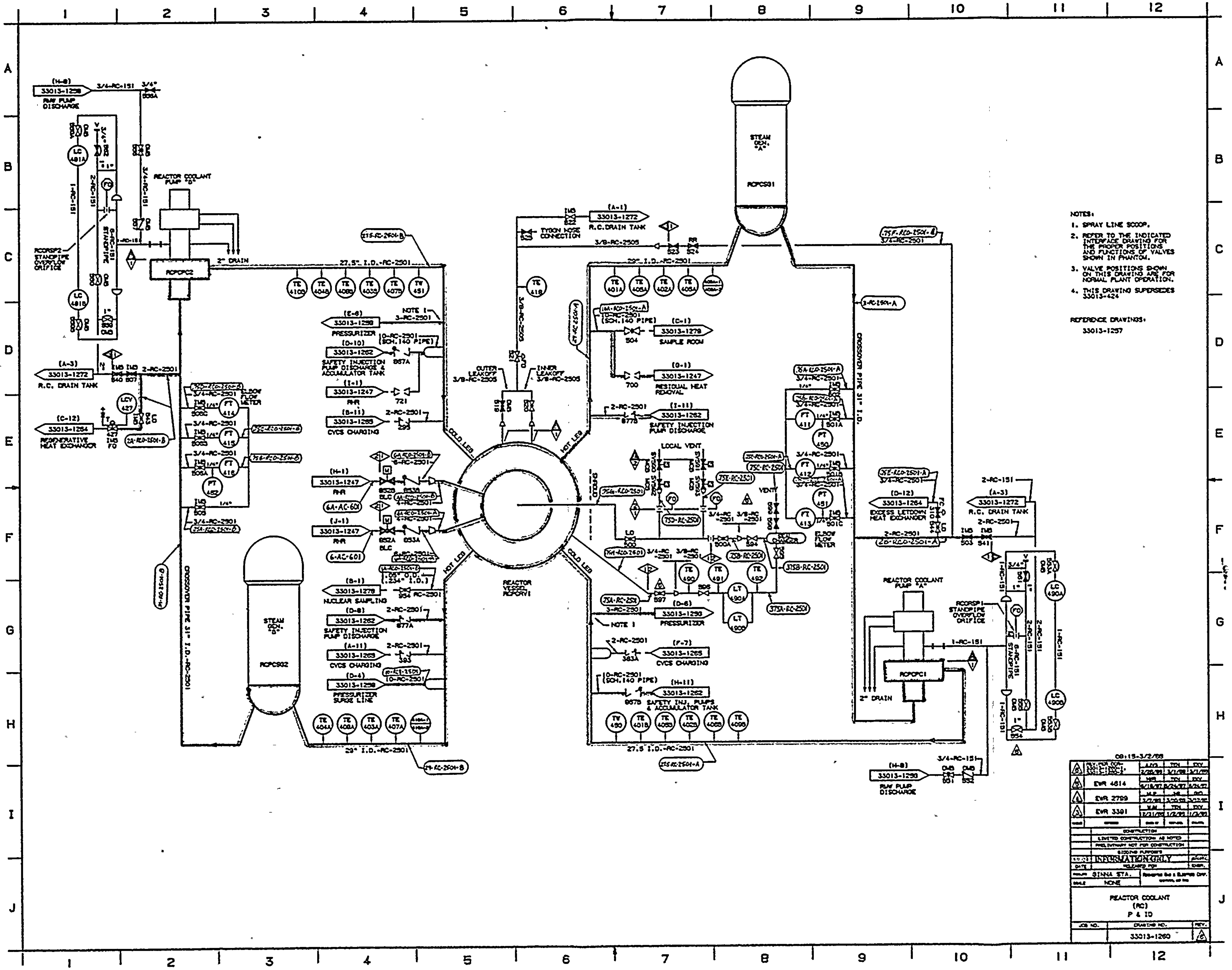

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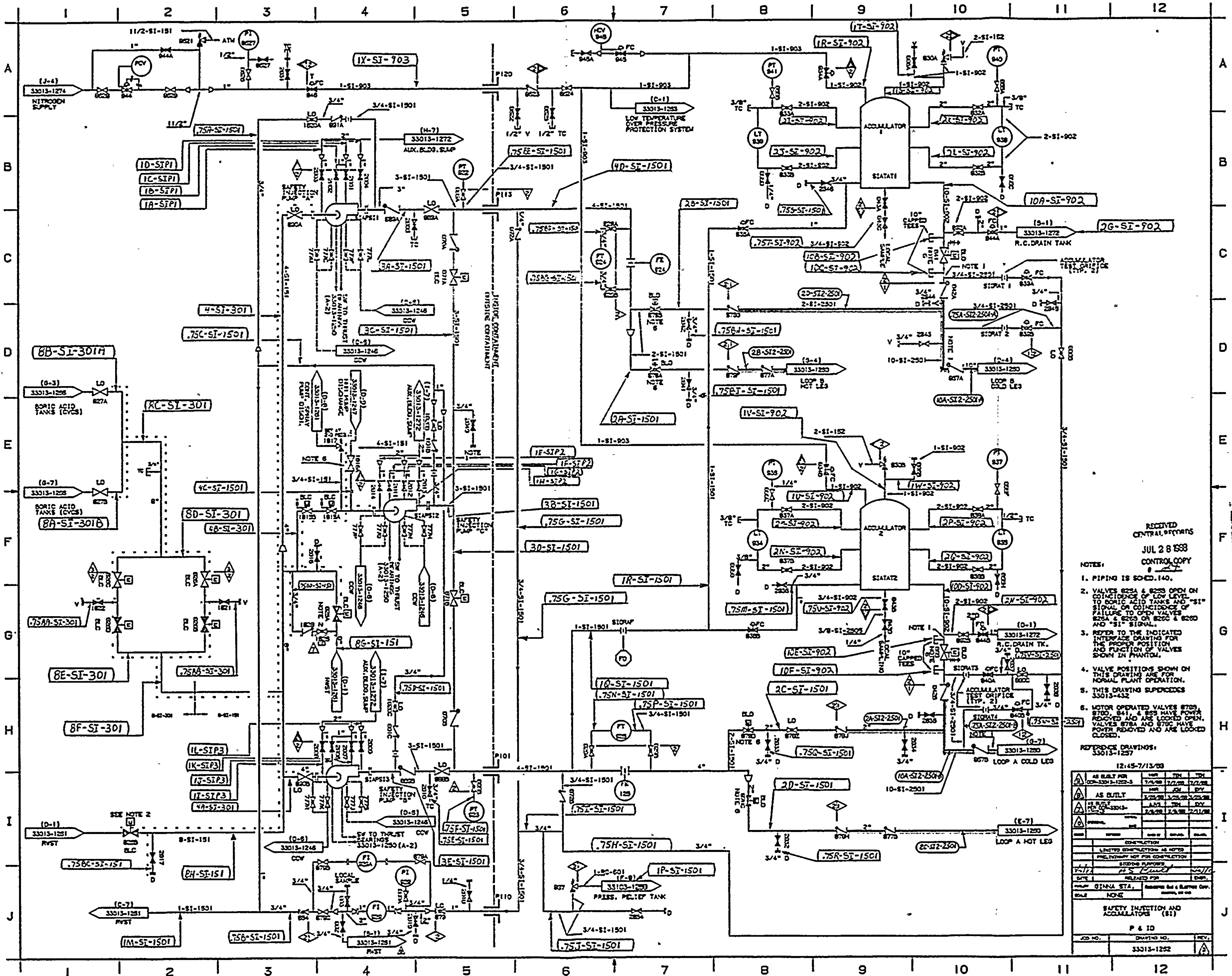

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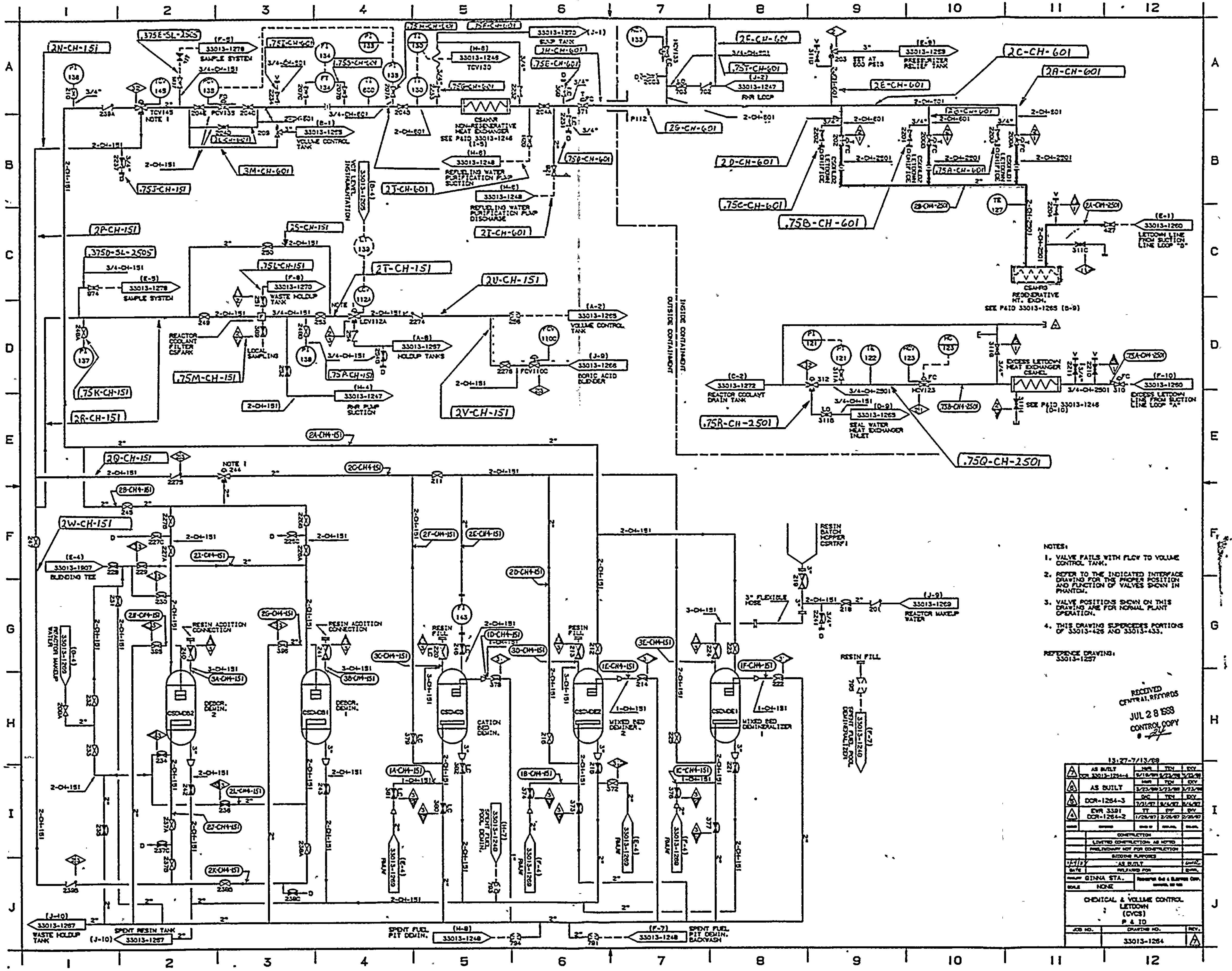

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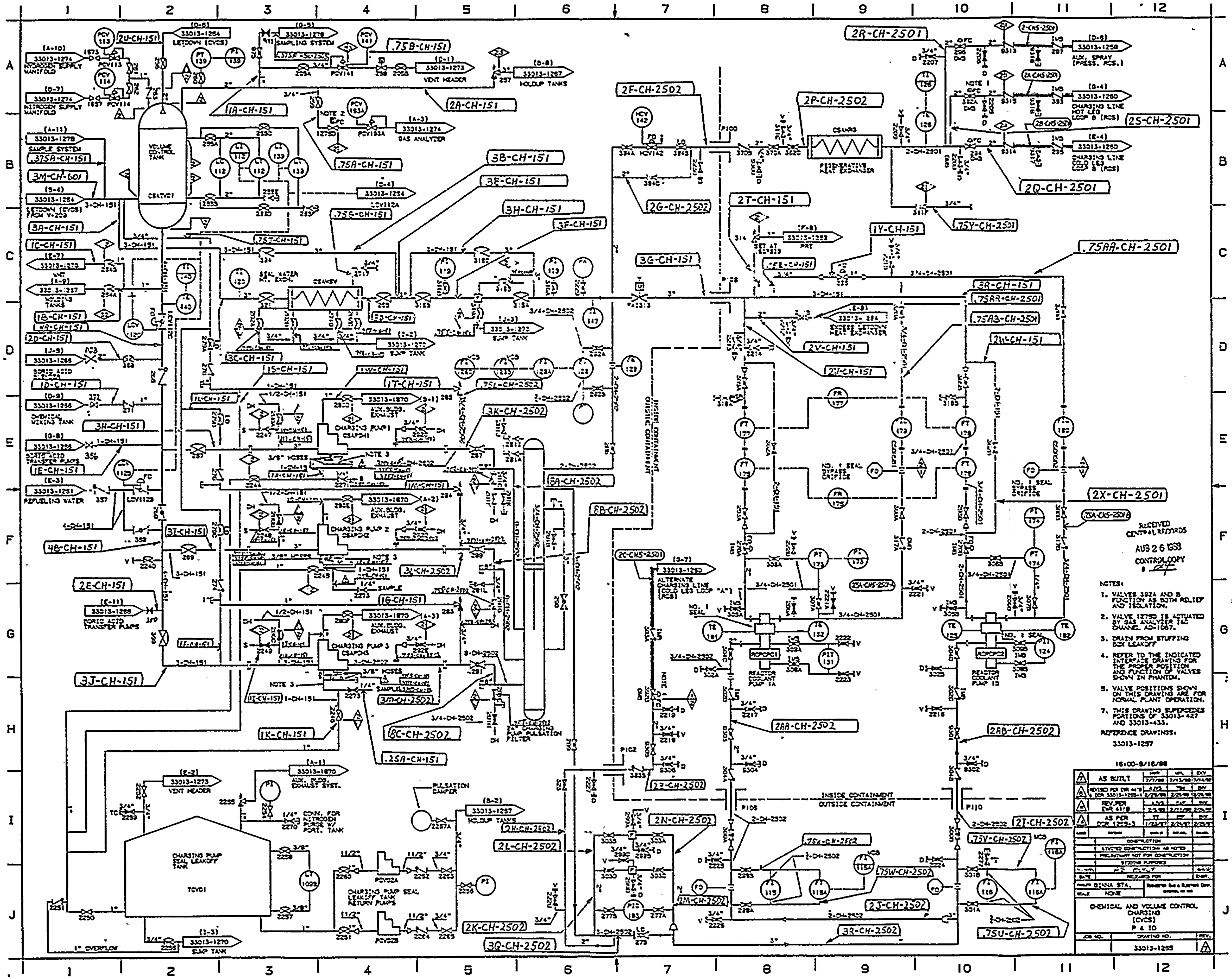

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ATTACHMENT B

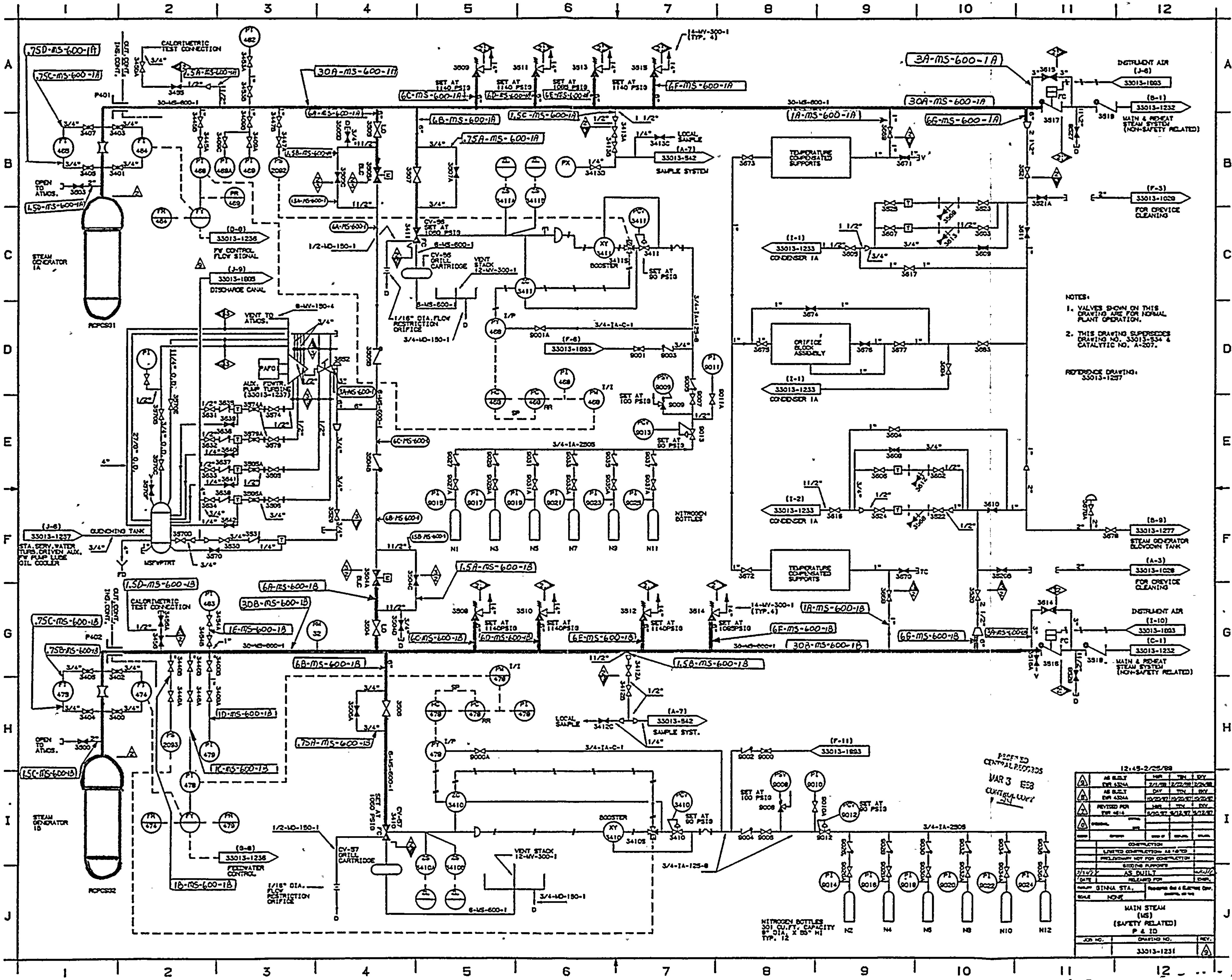
Class 2 Non-Exempted System Drawings

Introduction

All Systems/Lines that require examination by Code Requirements or Rochester Gas and Electric are indicated in red overlay.

The following Class 2 Non-Exempted P&ID Drawings are included after this listing:

33013-1231	Rev.9
33013-1236	Rev.7
33013-1247	Rev.8
33013-1260	Rev.6
33013-1261	Rev.9
33013-1262	Rev.9
33013-1264	Rev.7
33013-1265	Rev.7
33013-1266	Rev.8



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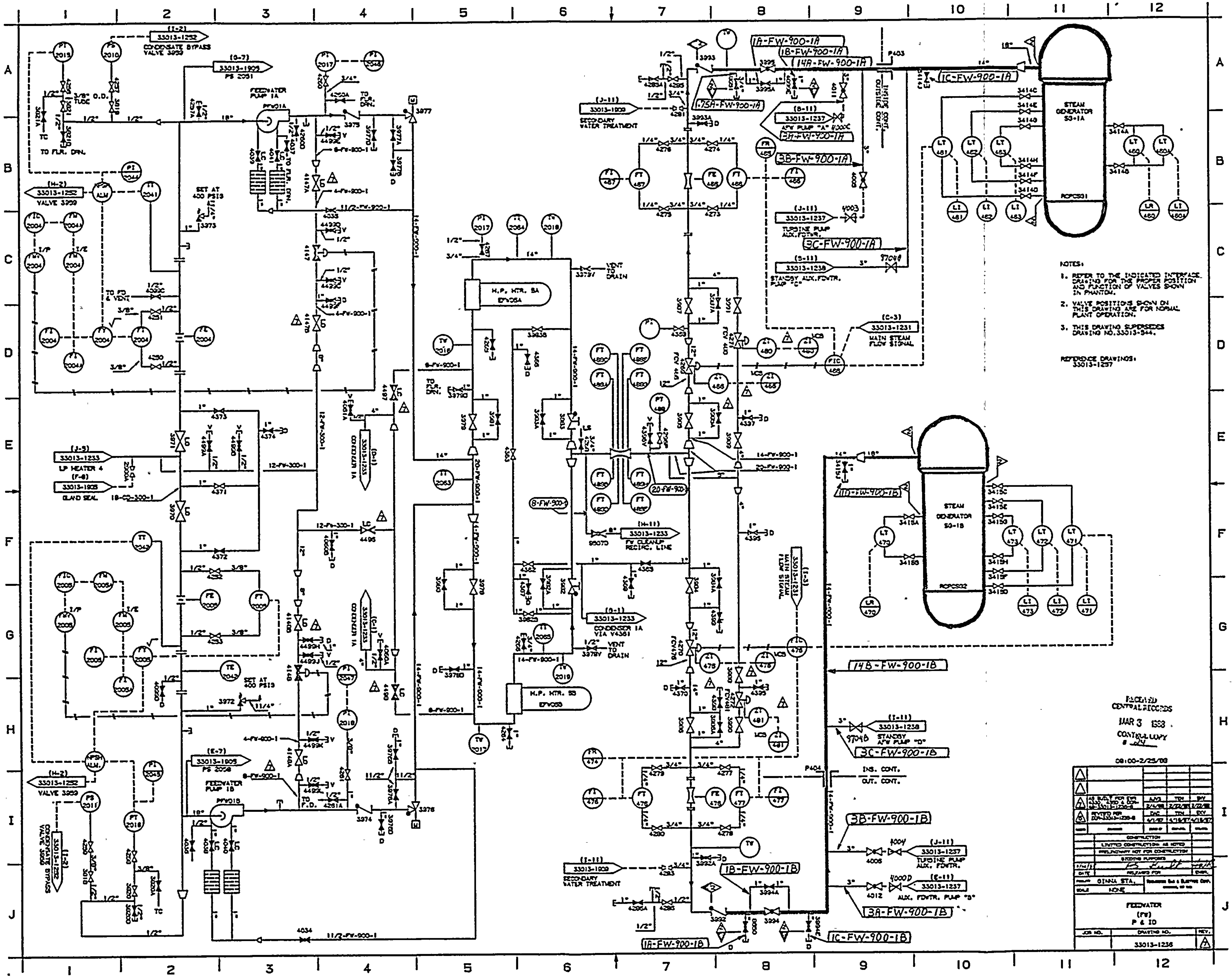

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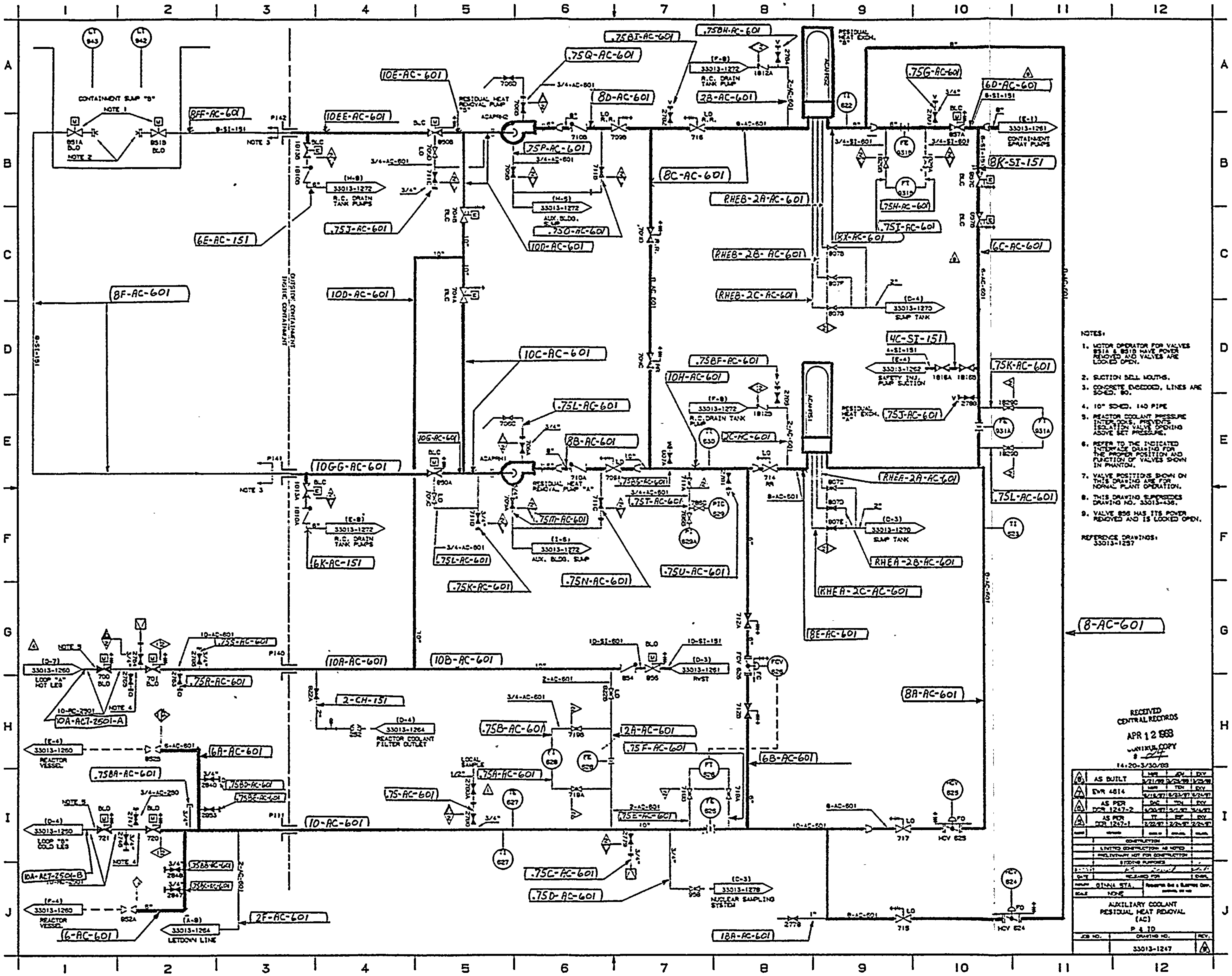

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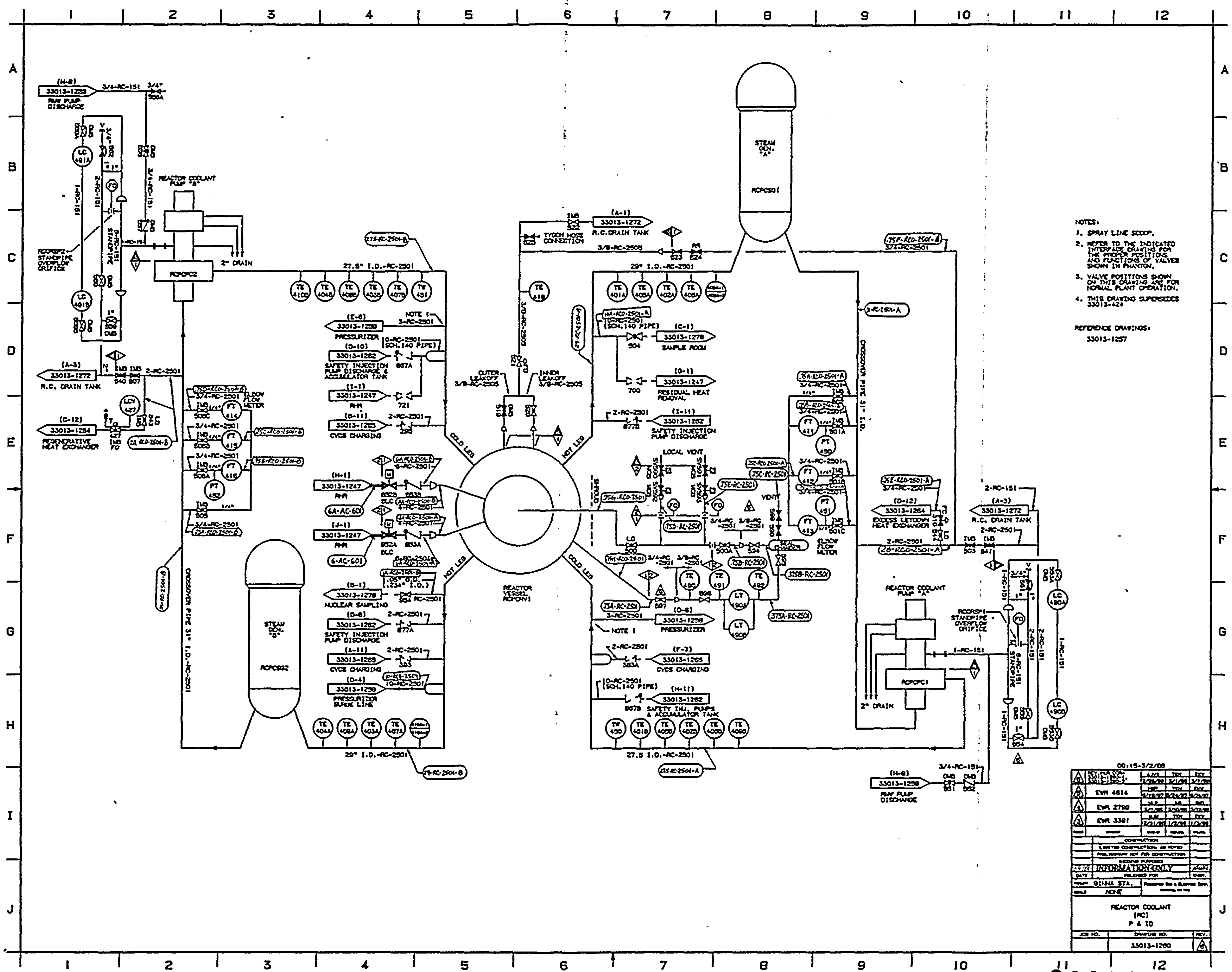

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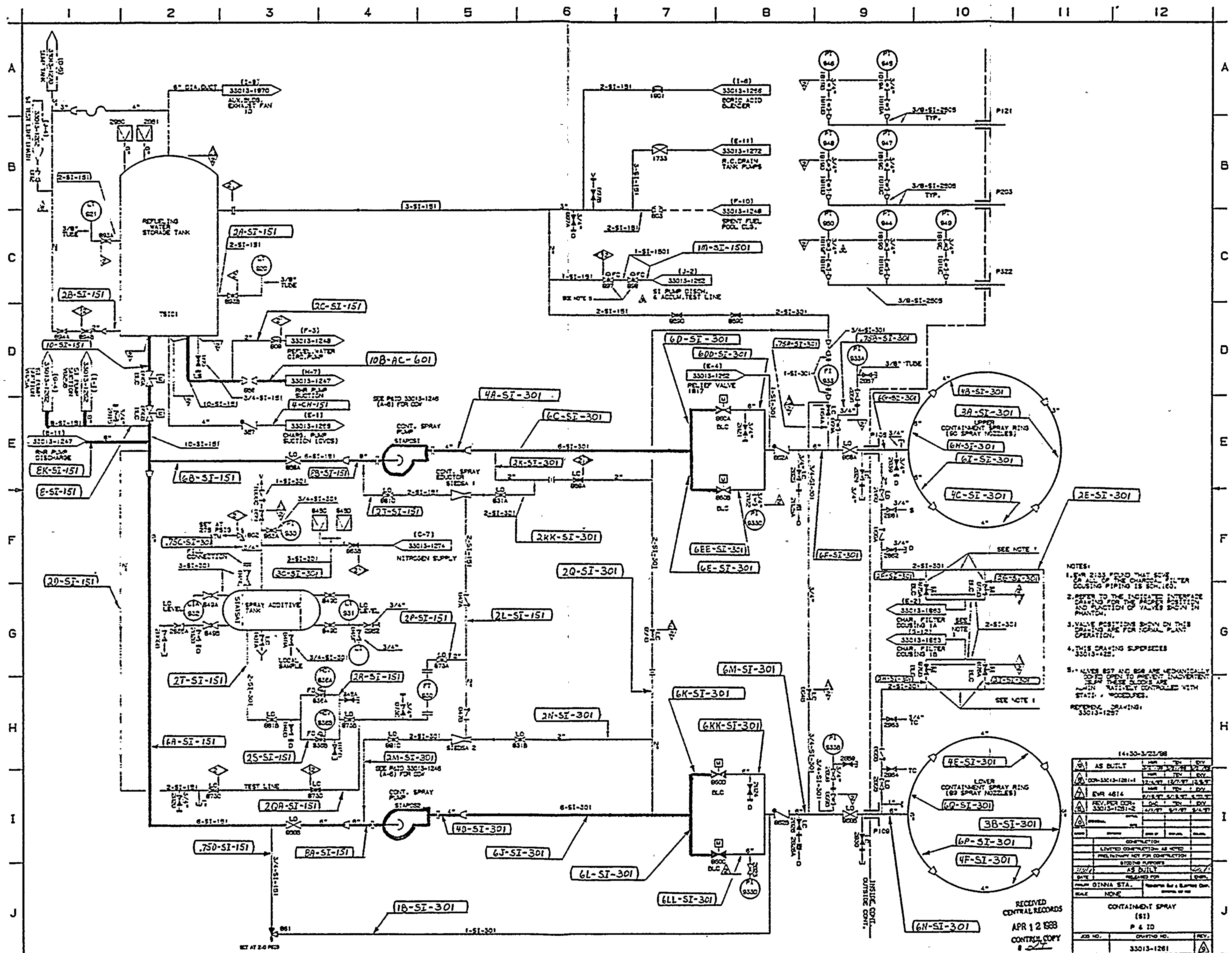

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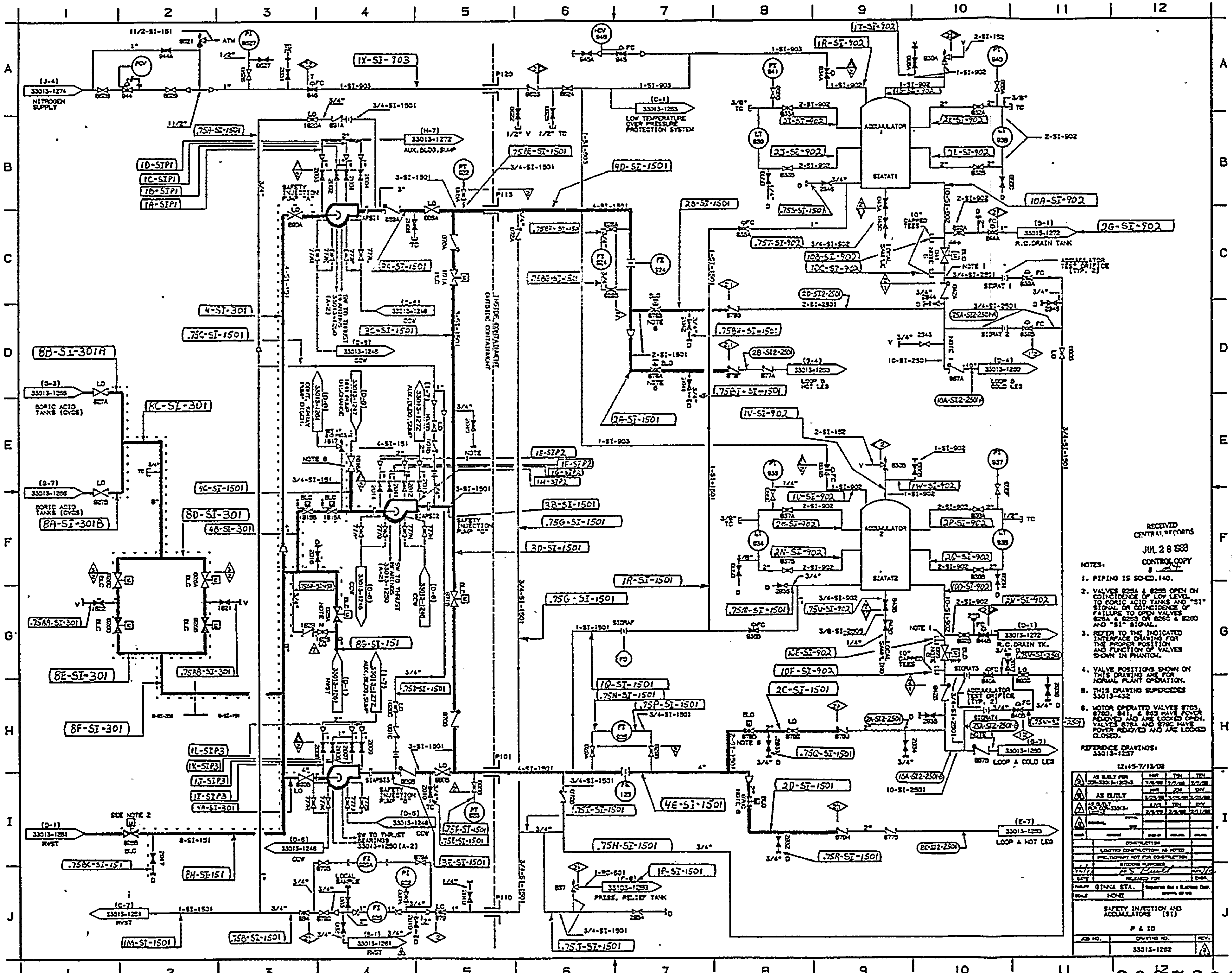

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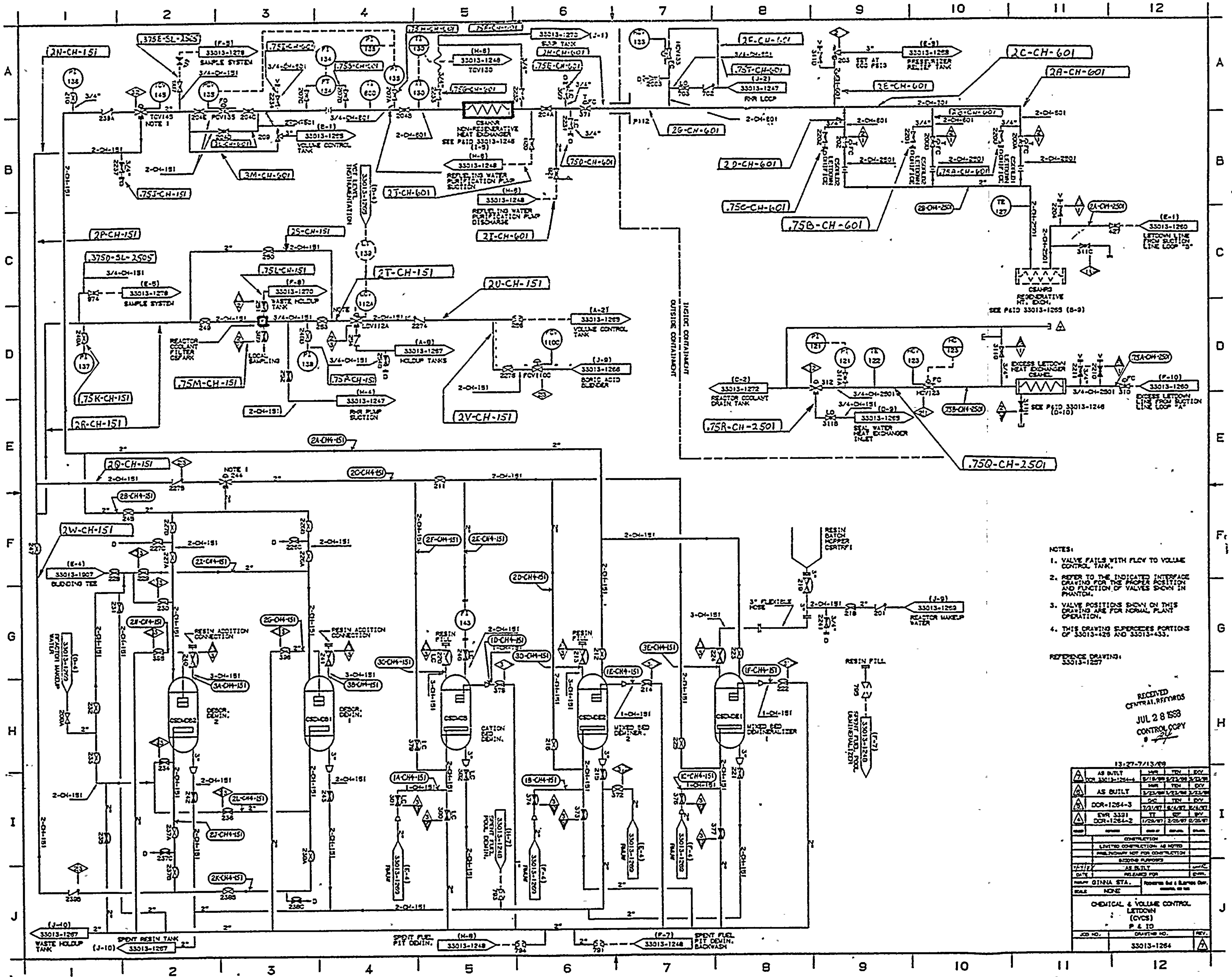

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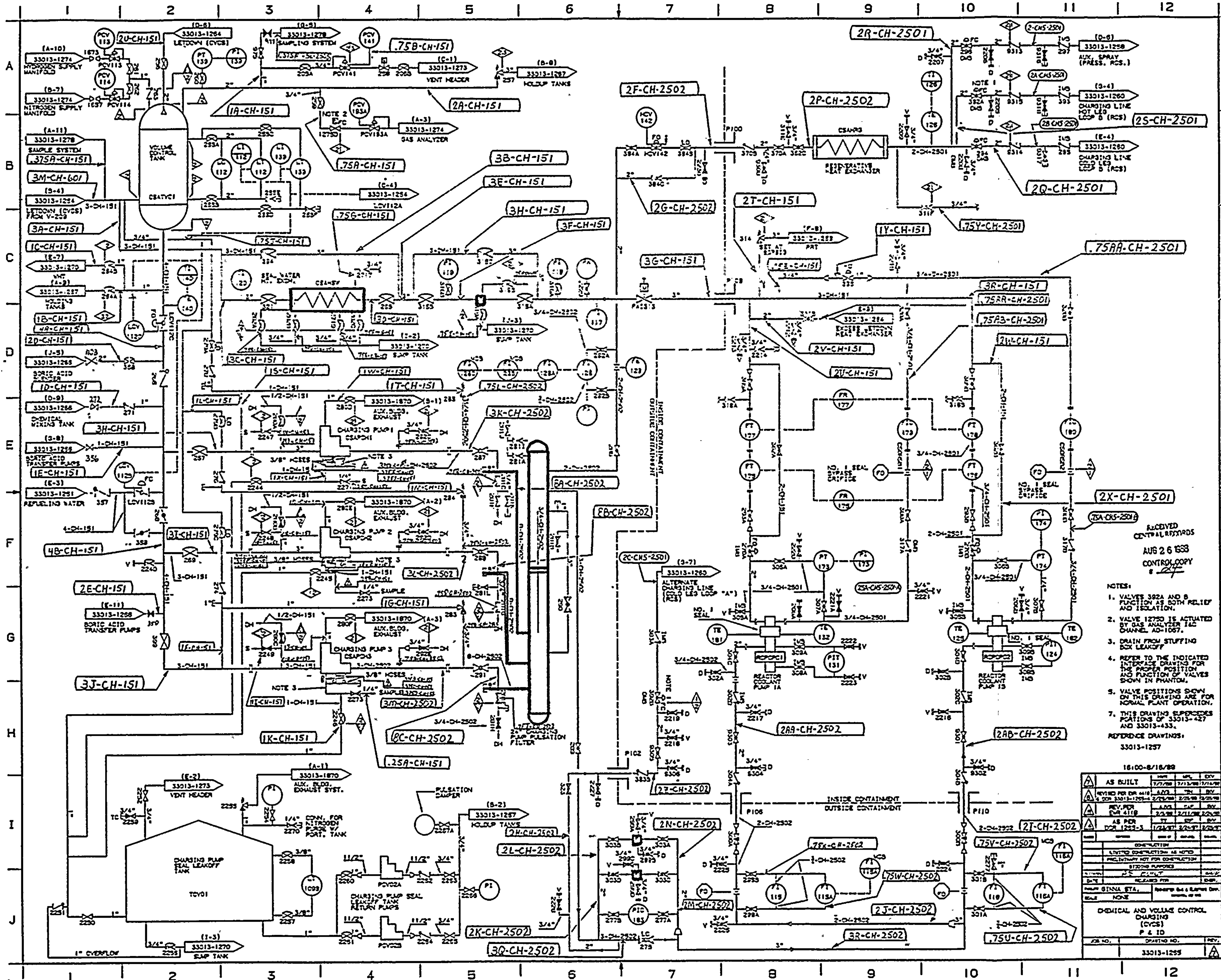

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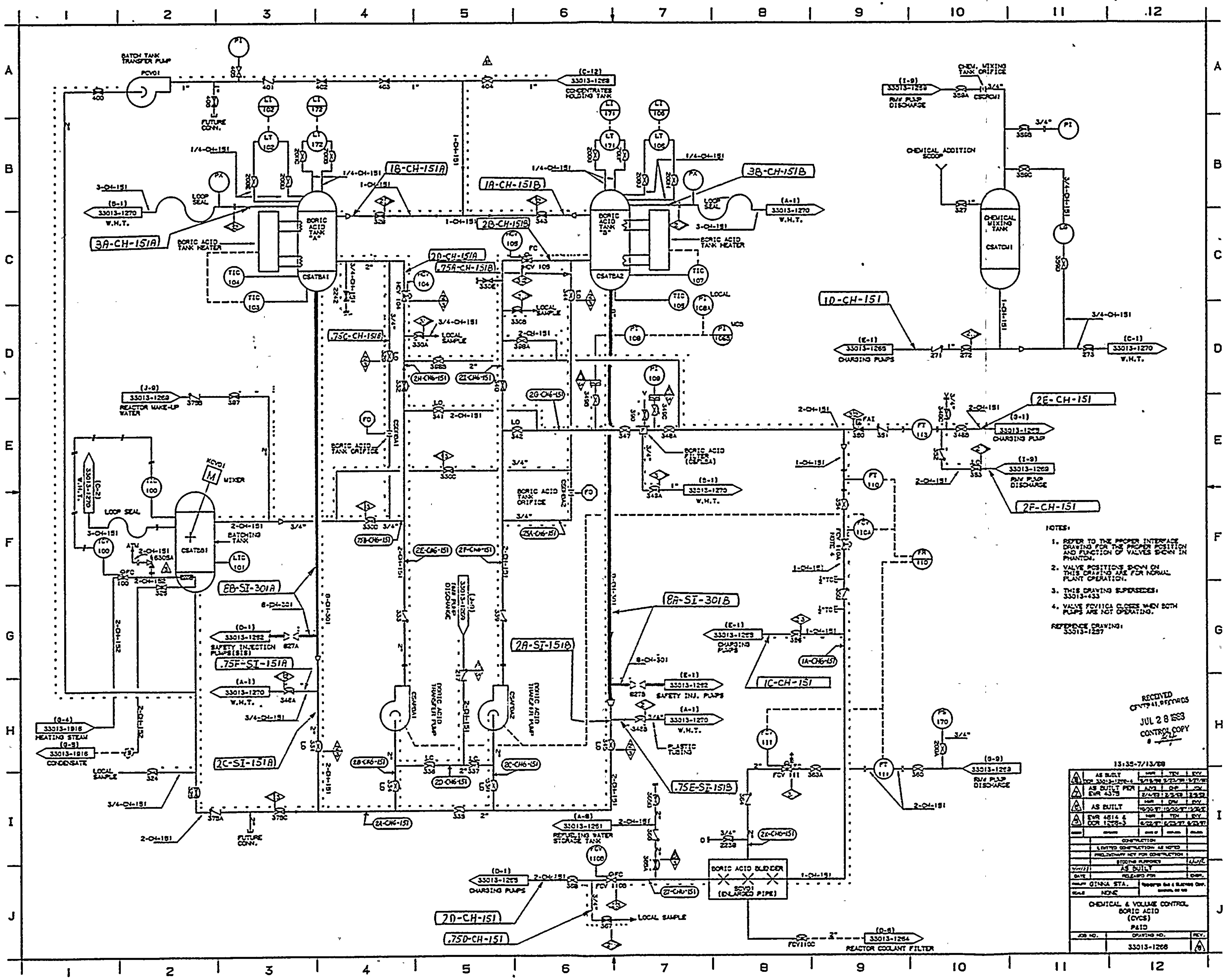

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ATTACHMENT B

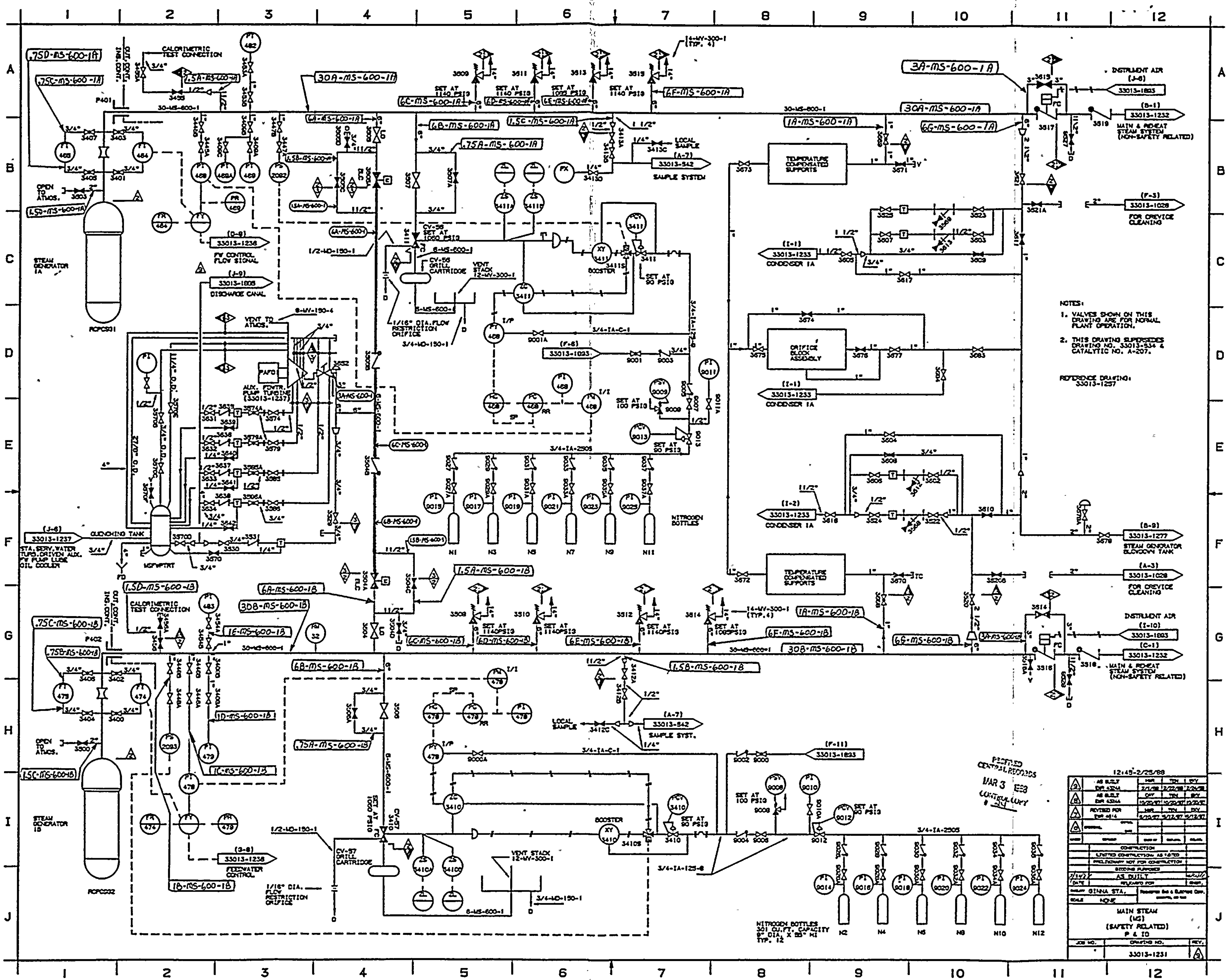
Class 3 Non-Exempted System Drawings

Introduction

All Systems/Lines that require examination by Code Requirements or Rochester Gas and Electric are indicated in green overlay.

The following Class 3 Non-Exempted P&ID Drawings are included after this listing:

33013-1231	Rev.9
33013-1237	Rev.11
33013-1238	Rev.2
33013-1239	Rev.4
33013-1245	Rev.6
33013-1246	Rev.4
33013-1250	Rev.6
33013-1267	Rev.5



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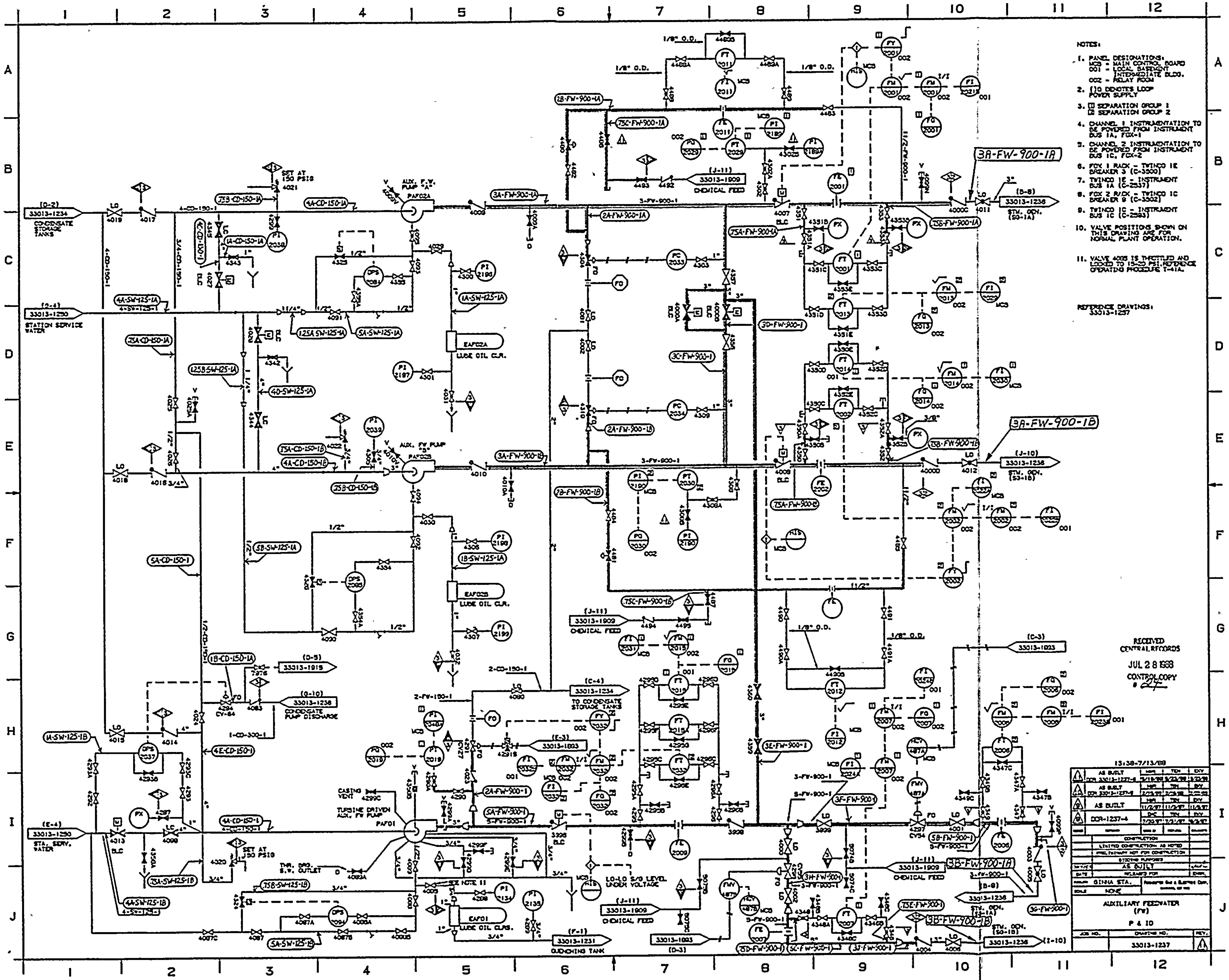

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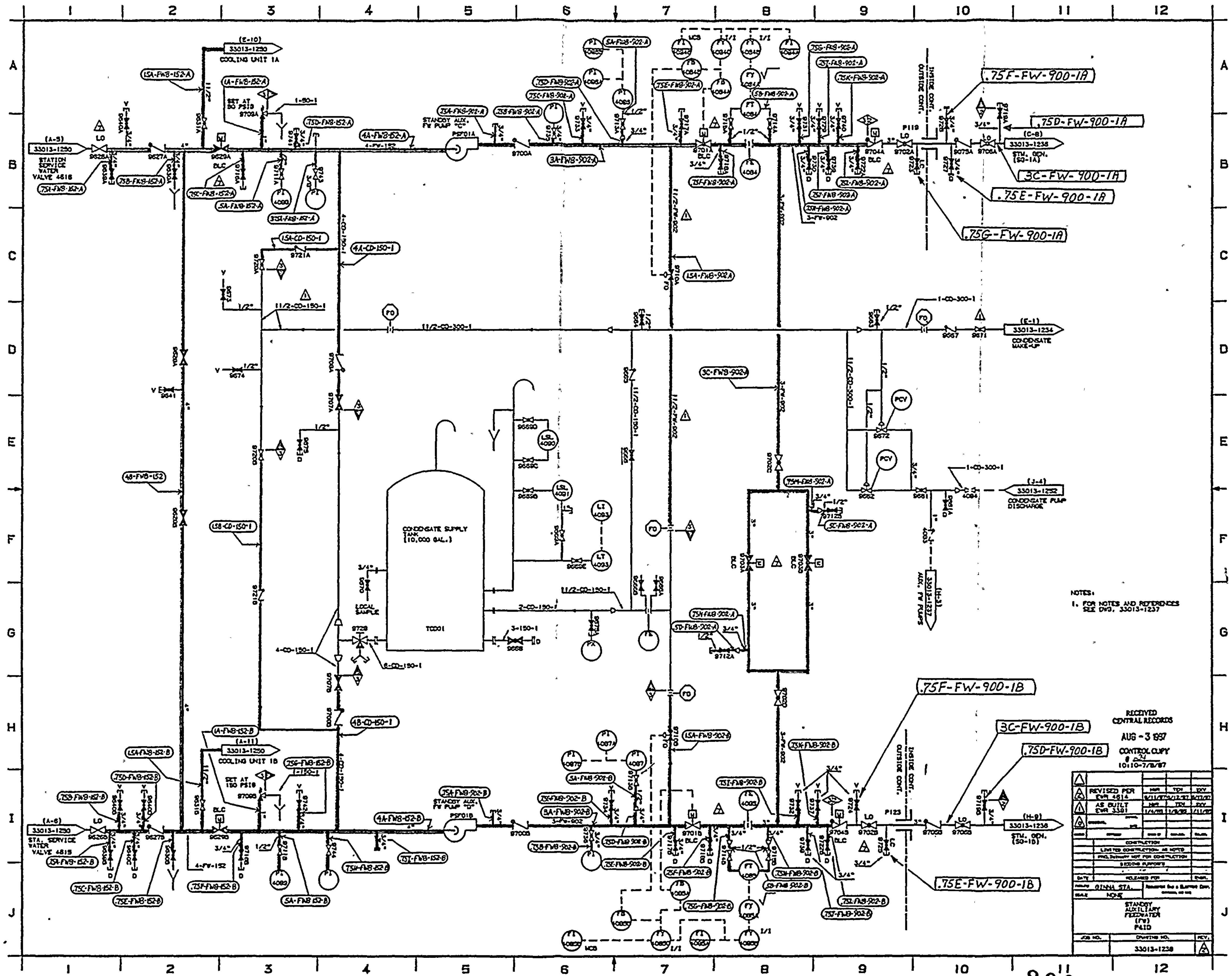

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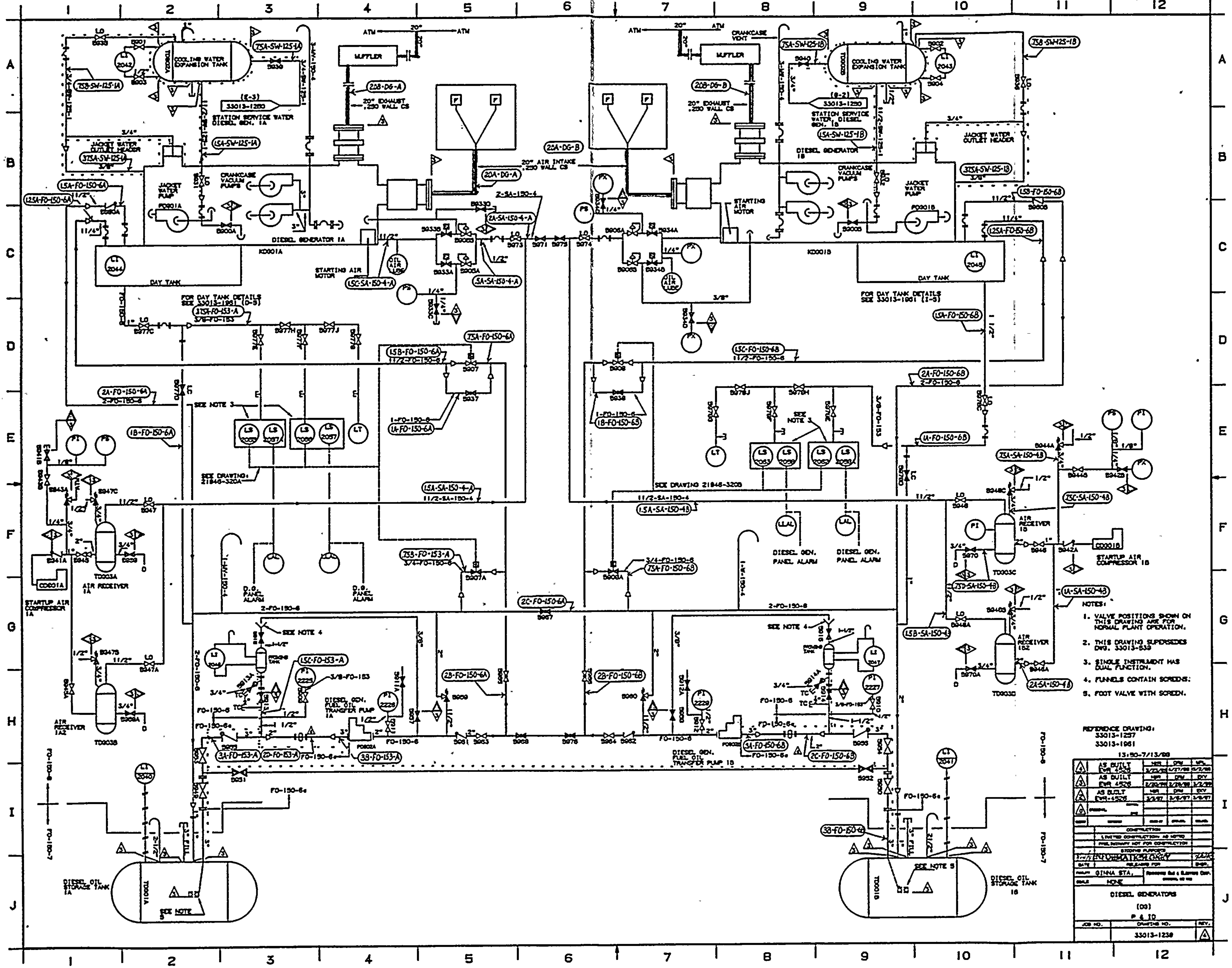

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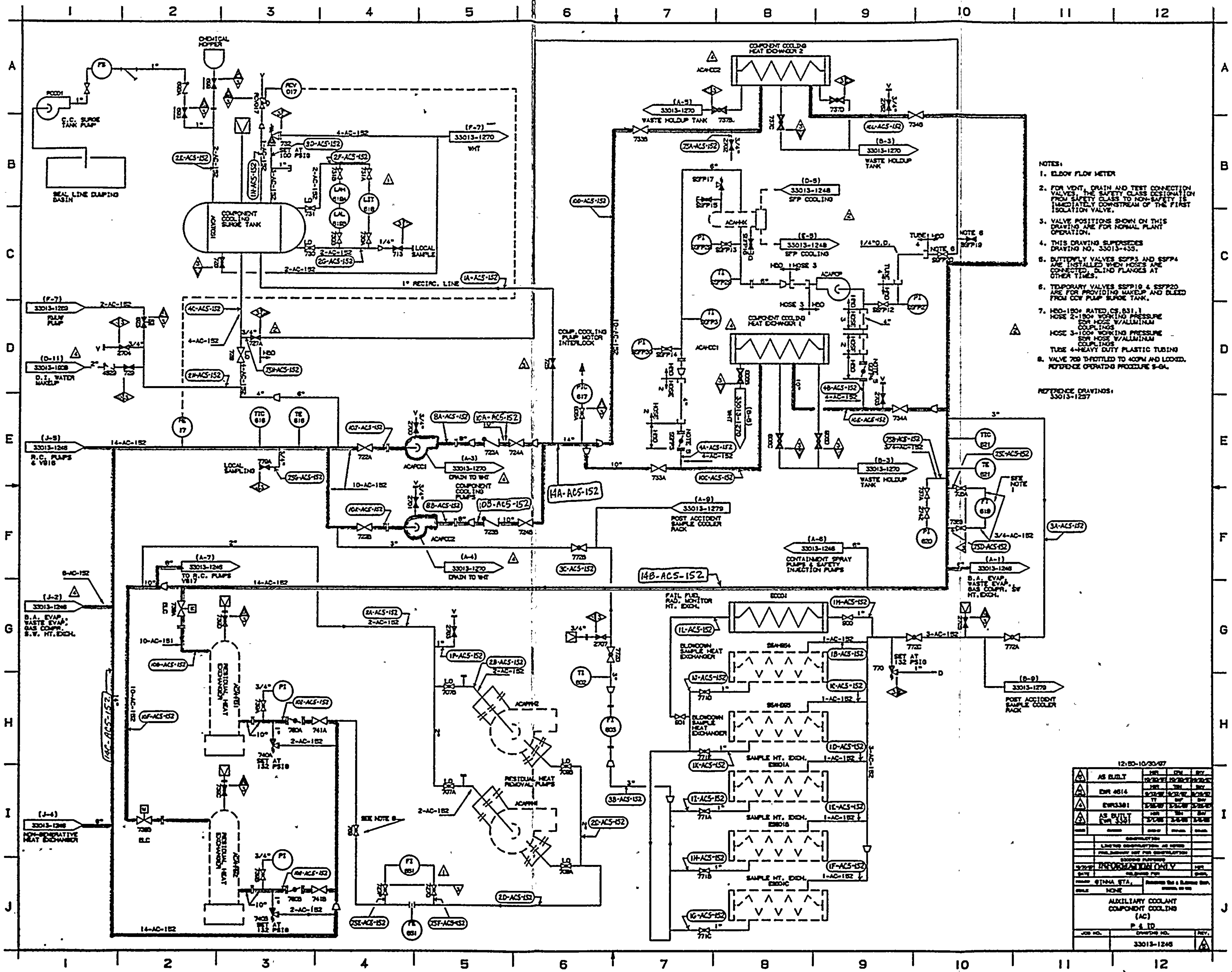

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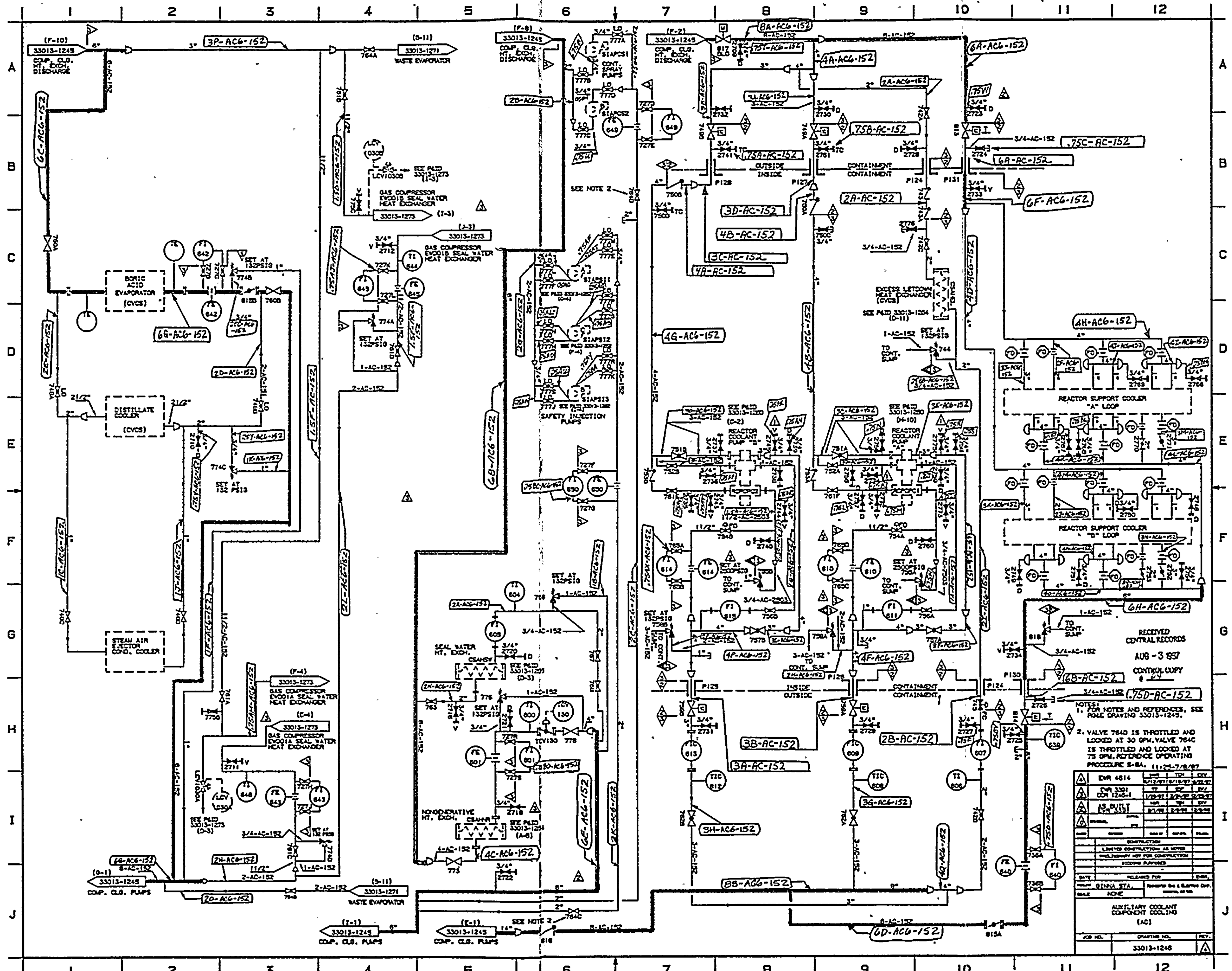

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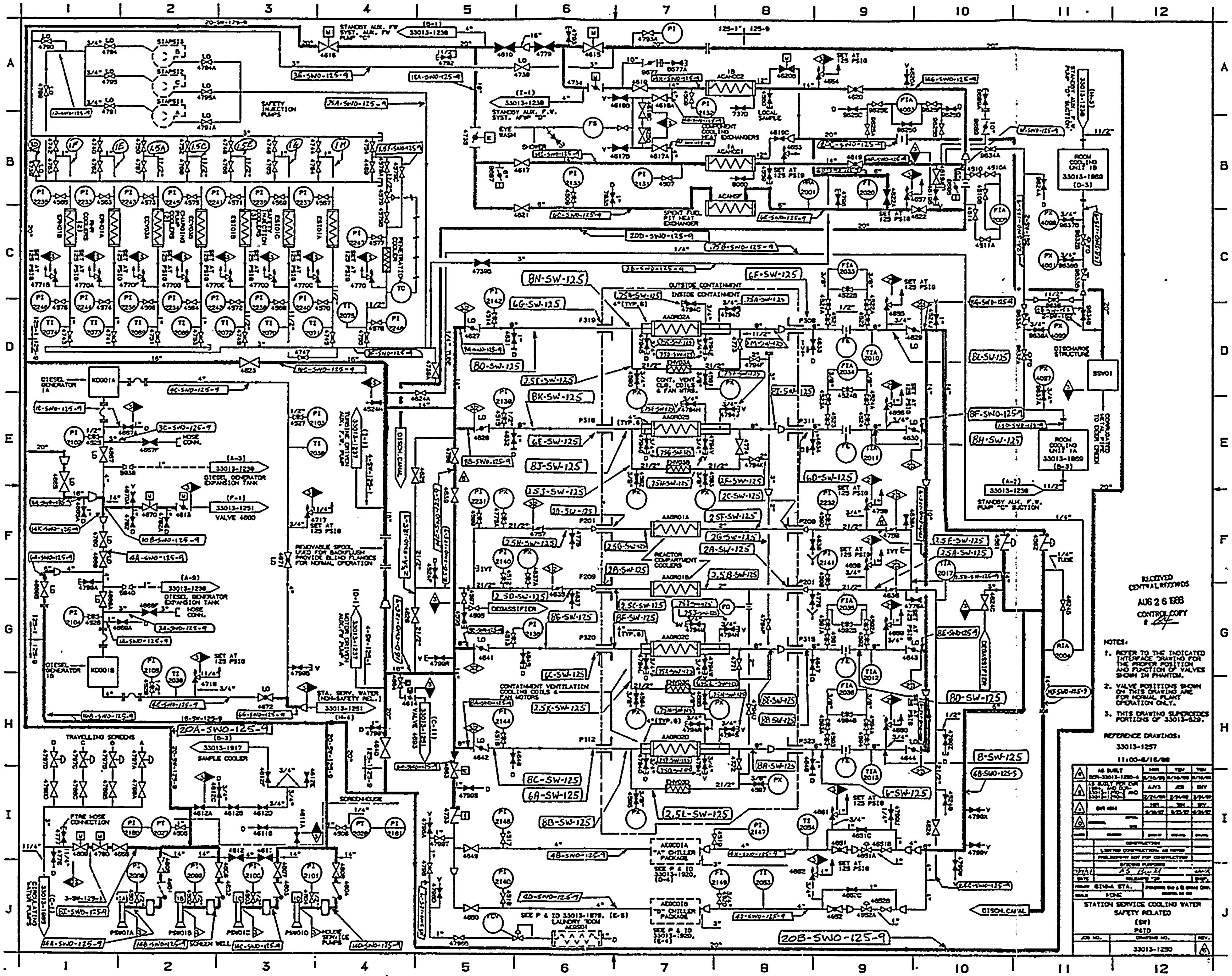

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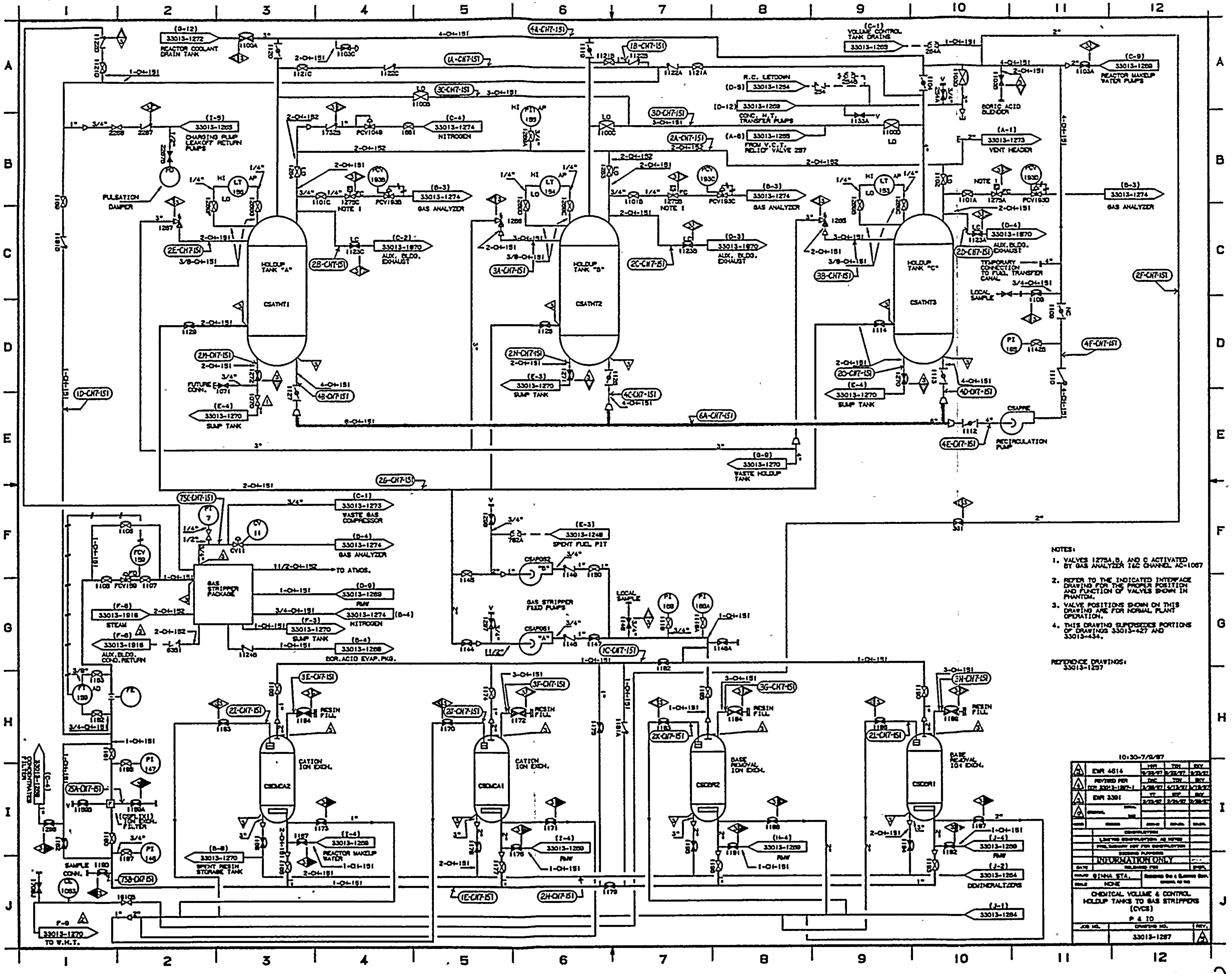

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ATTACHMENT B

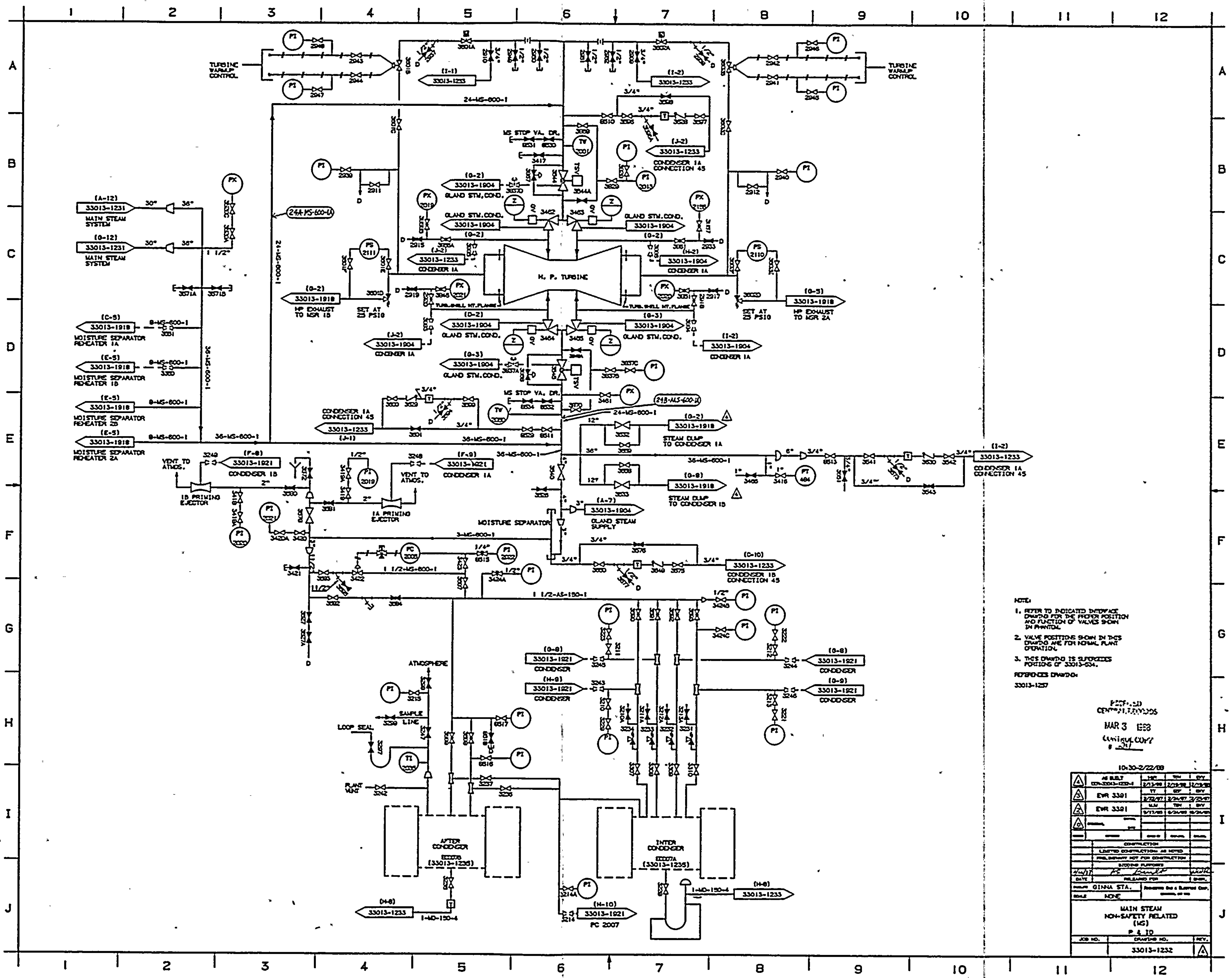
Exempted System Drawings

Introduction

This section lists Class 1, 2, and 3 P&ID Drawings that are exempted from examination. This section is provided to support the Line List Section as found in Attachment A.

The following is a listing of Exempted System P&ID Drawings that is included in this section.

33013-1232	Rev.4
33013-1234	Rev.5
33013-1241	Rev.5
33013-1248	Rev.5
33013-1263	Rev.2
33013-1268	Rev.4
33013-1269	Rev.3
33013-1270	Rev.2
33013-1271	Rev.2
33013-1272	Rev.5
33013-1273	Rev.8
33013-1274	Rev.5
33013-1275	Rev.2
33013-1276	Rev.2
33013-1277	Rev.5
33013-1278	Rev.9
33013-1279	Rev.4
33013-1865	Rev.2
33013-1870	Rev.2



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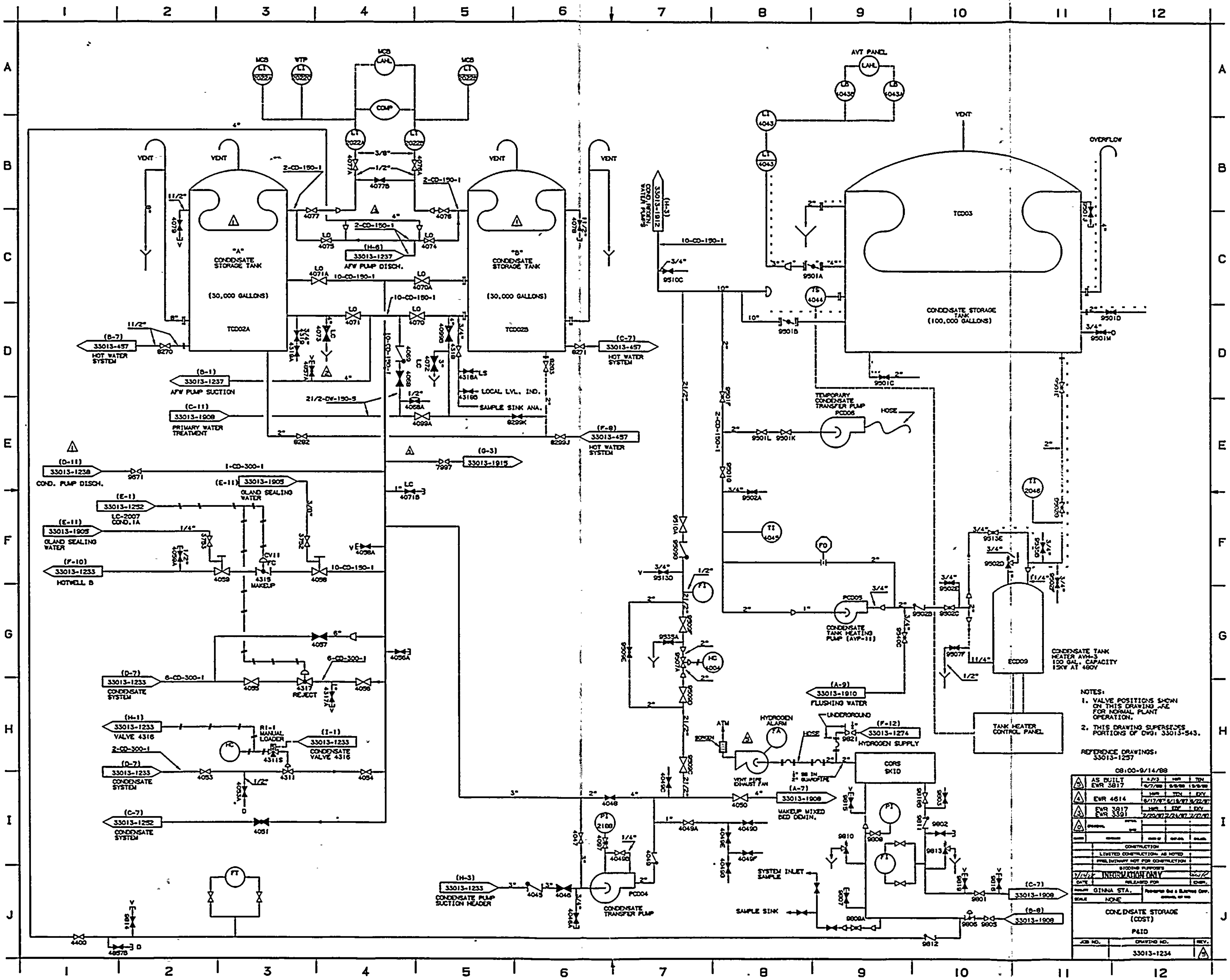

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SI APERTURE CARD

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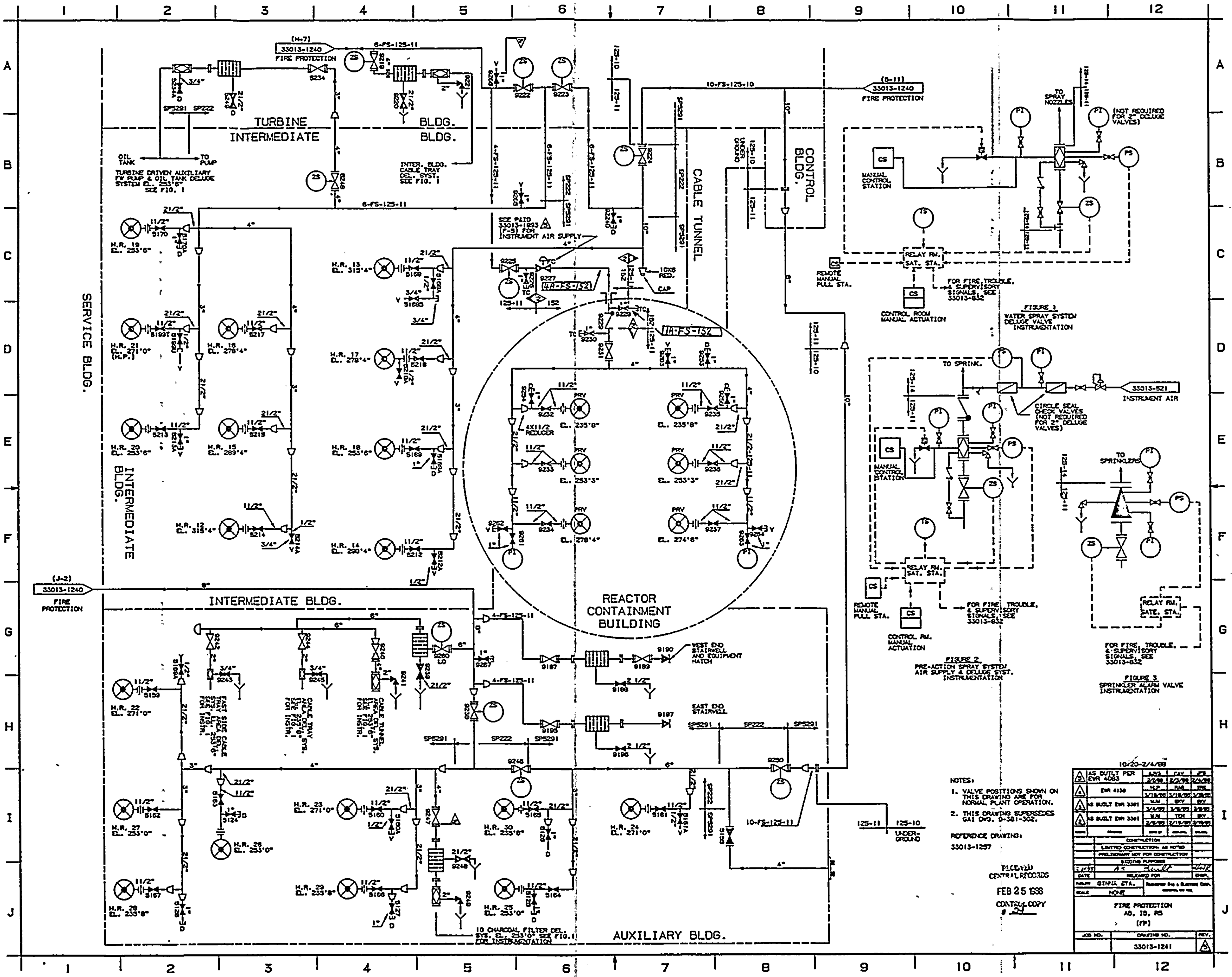

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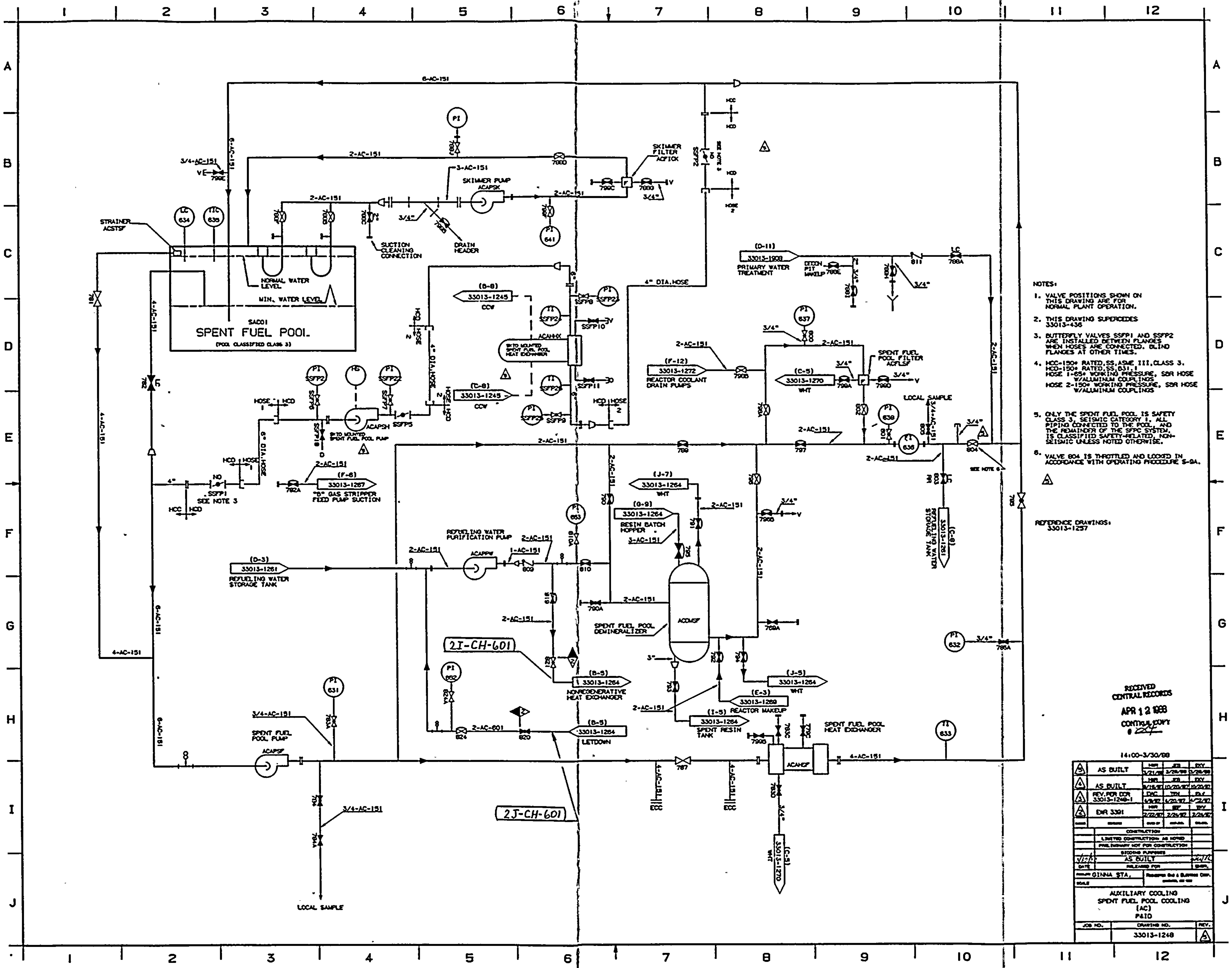
SI APERTURE CARD

Also Available On Aperture Card

8907260180-26



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SI APERTURE CARD

Also Available On Aperture Card

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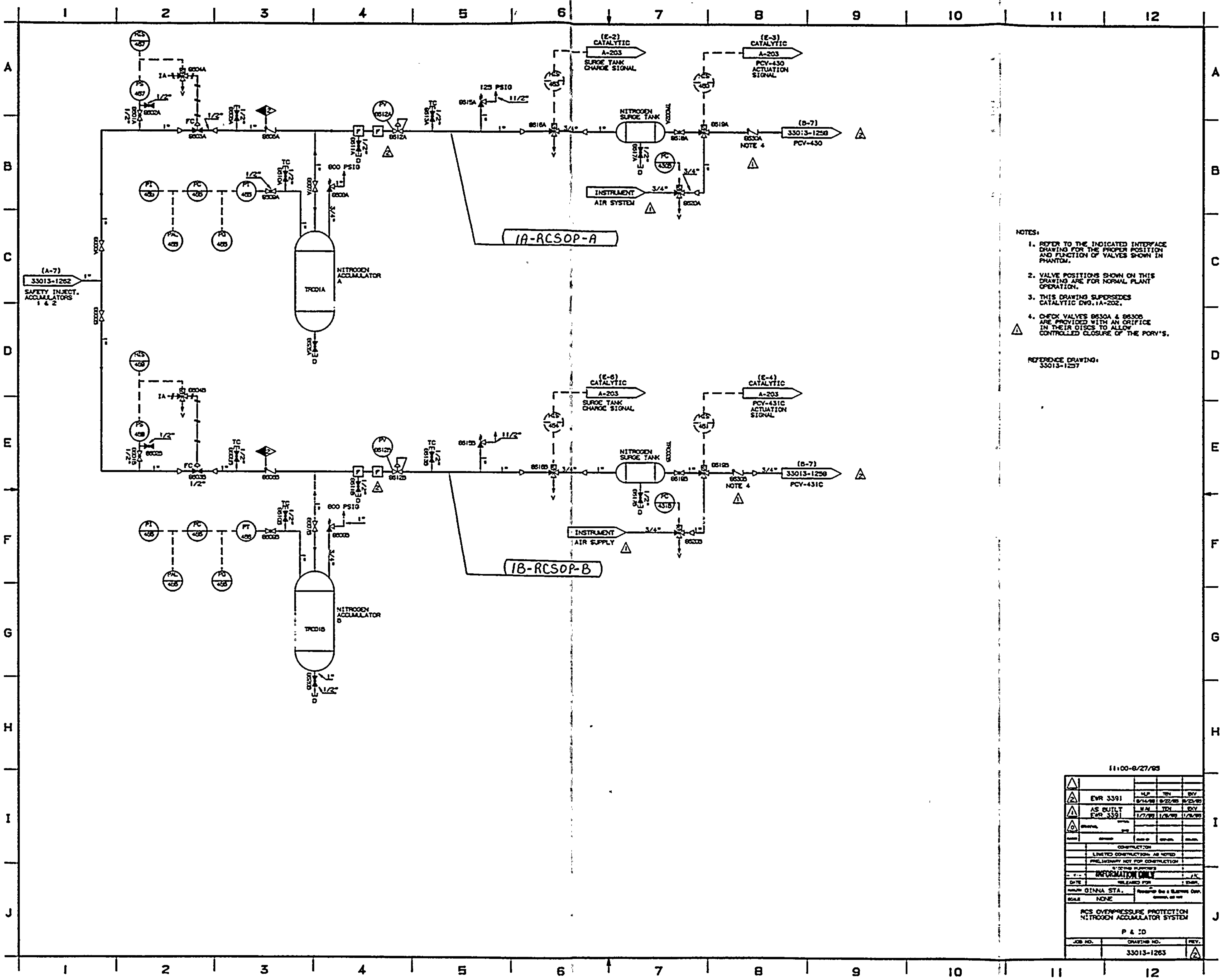

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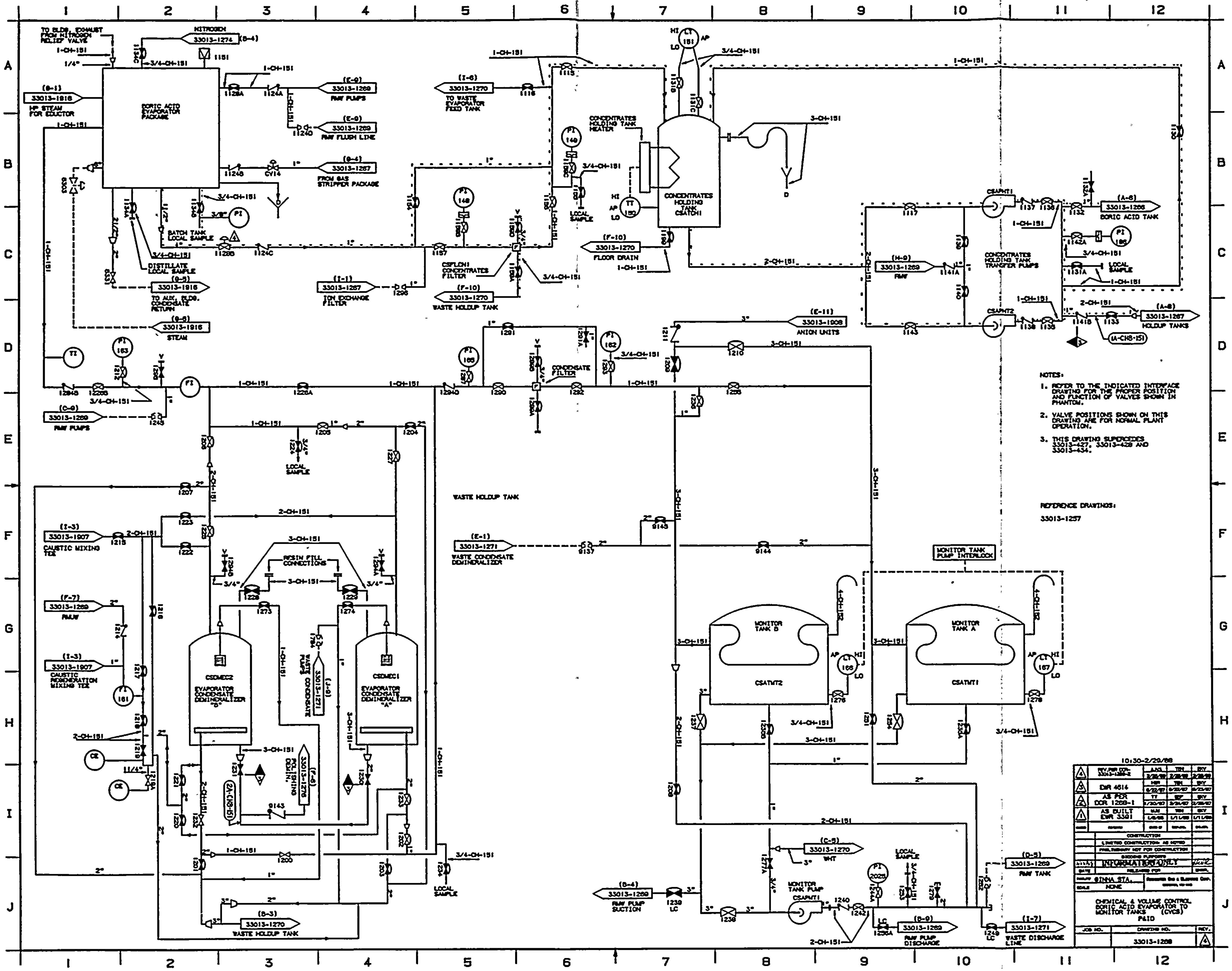

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SI APERTURE CARD

Also Available On Aperture Card

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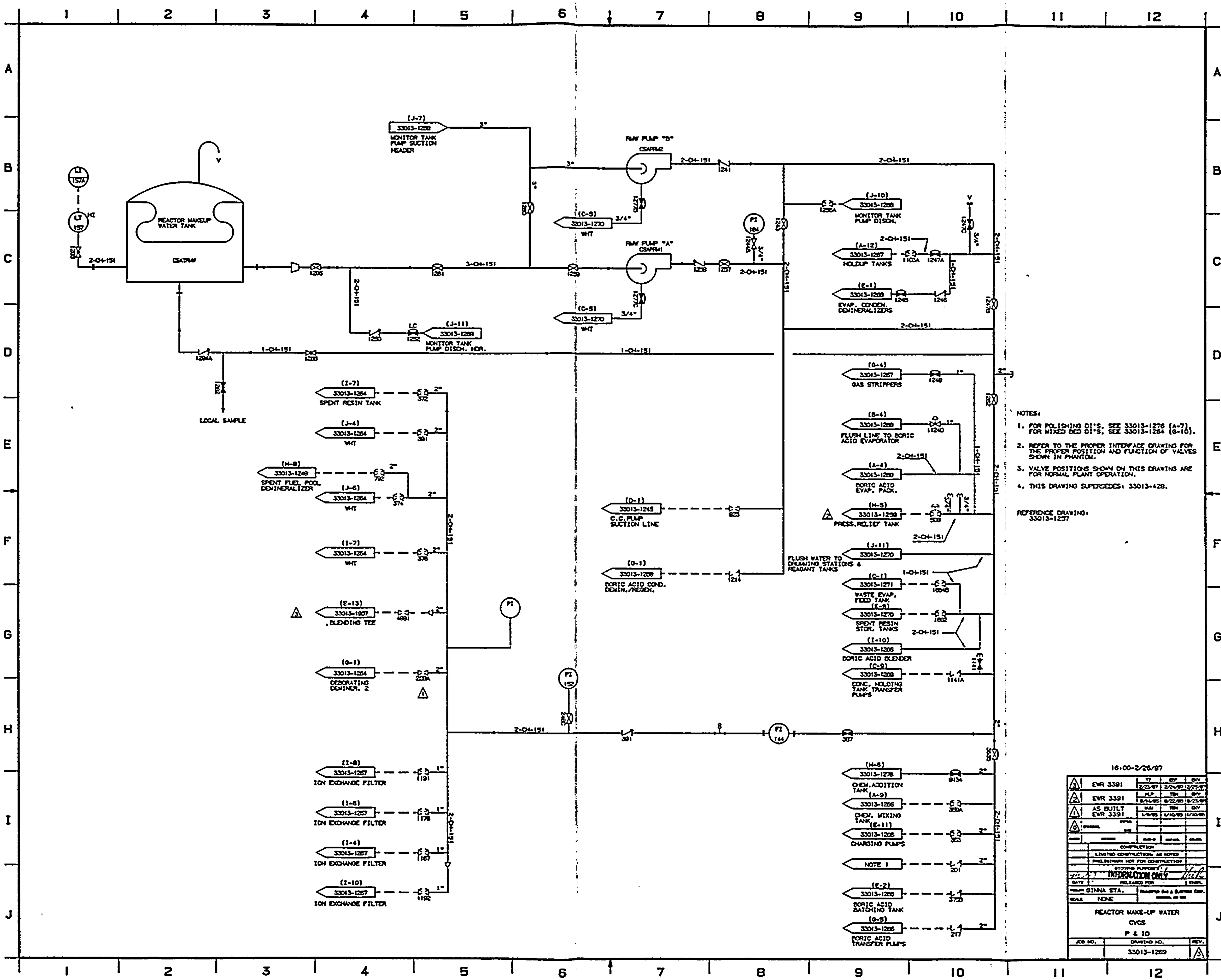

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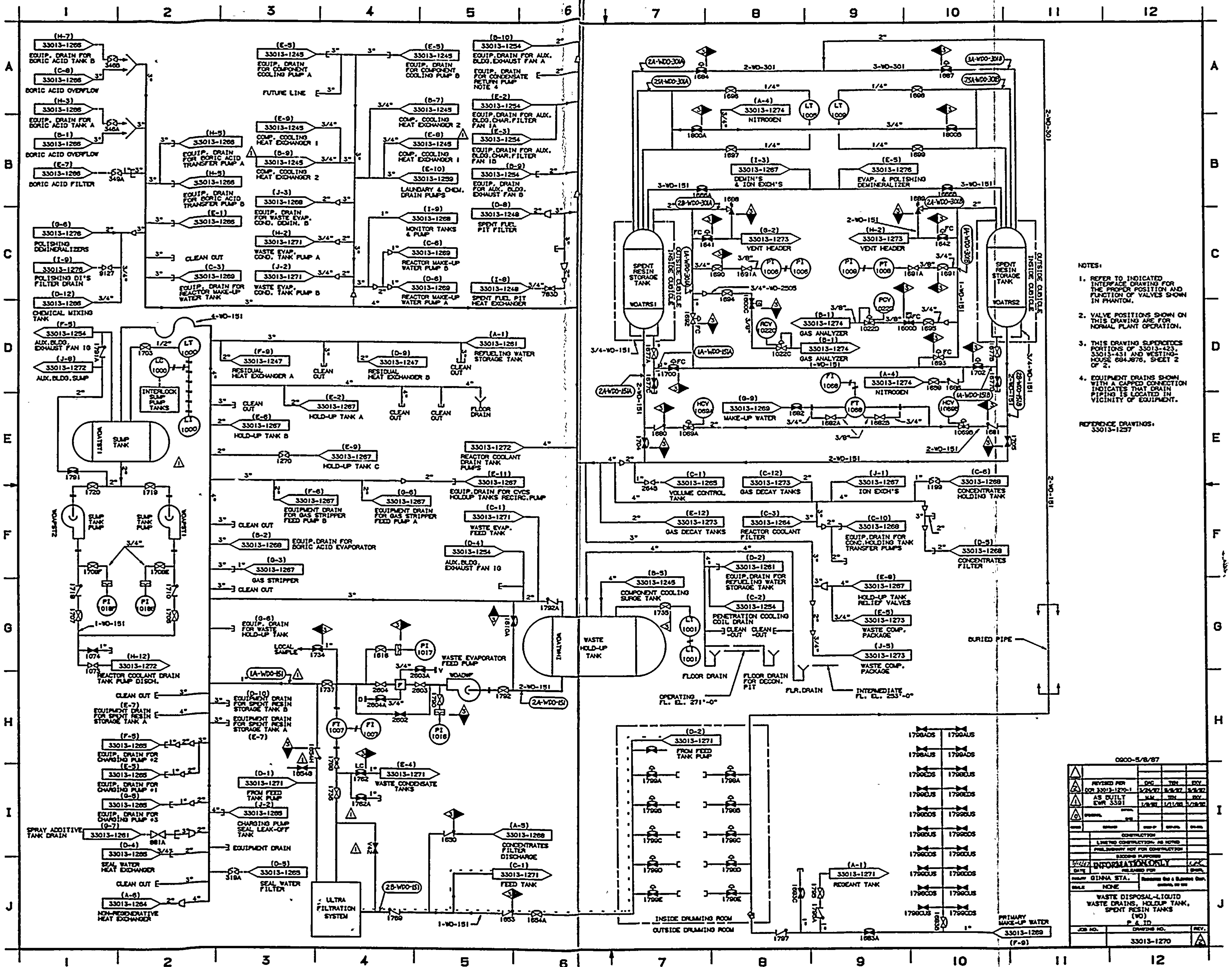

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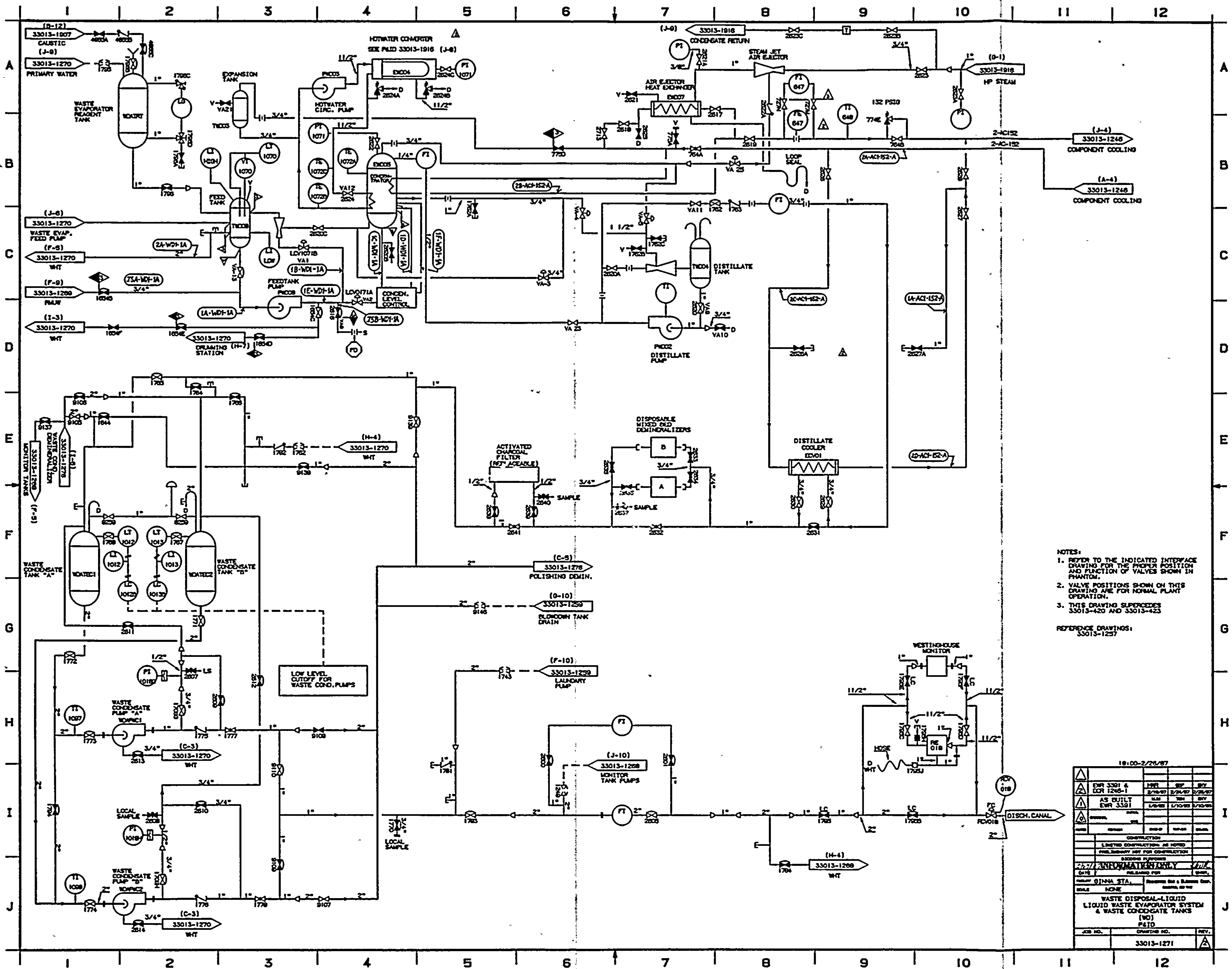

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SI APERTURE CARD

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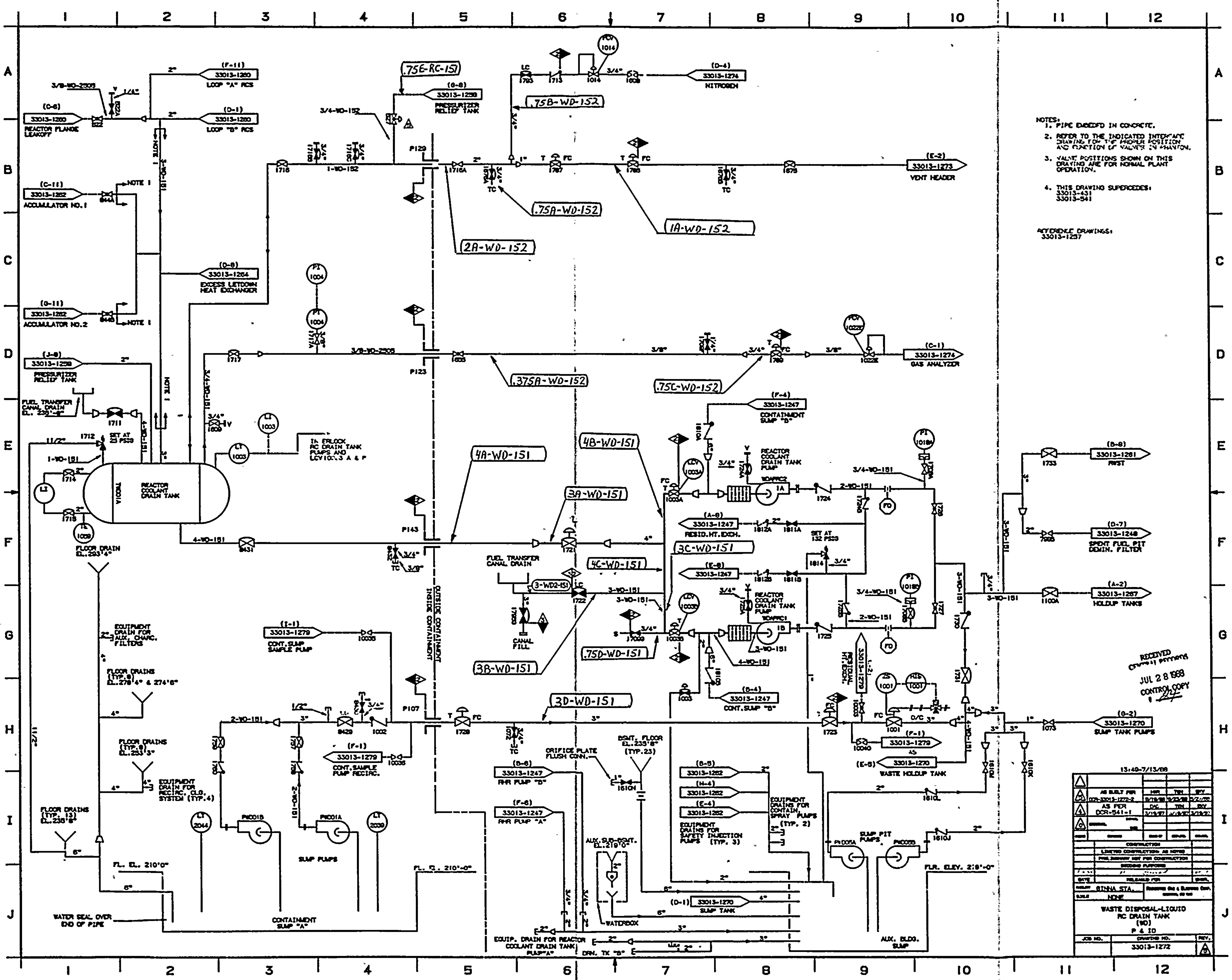

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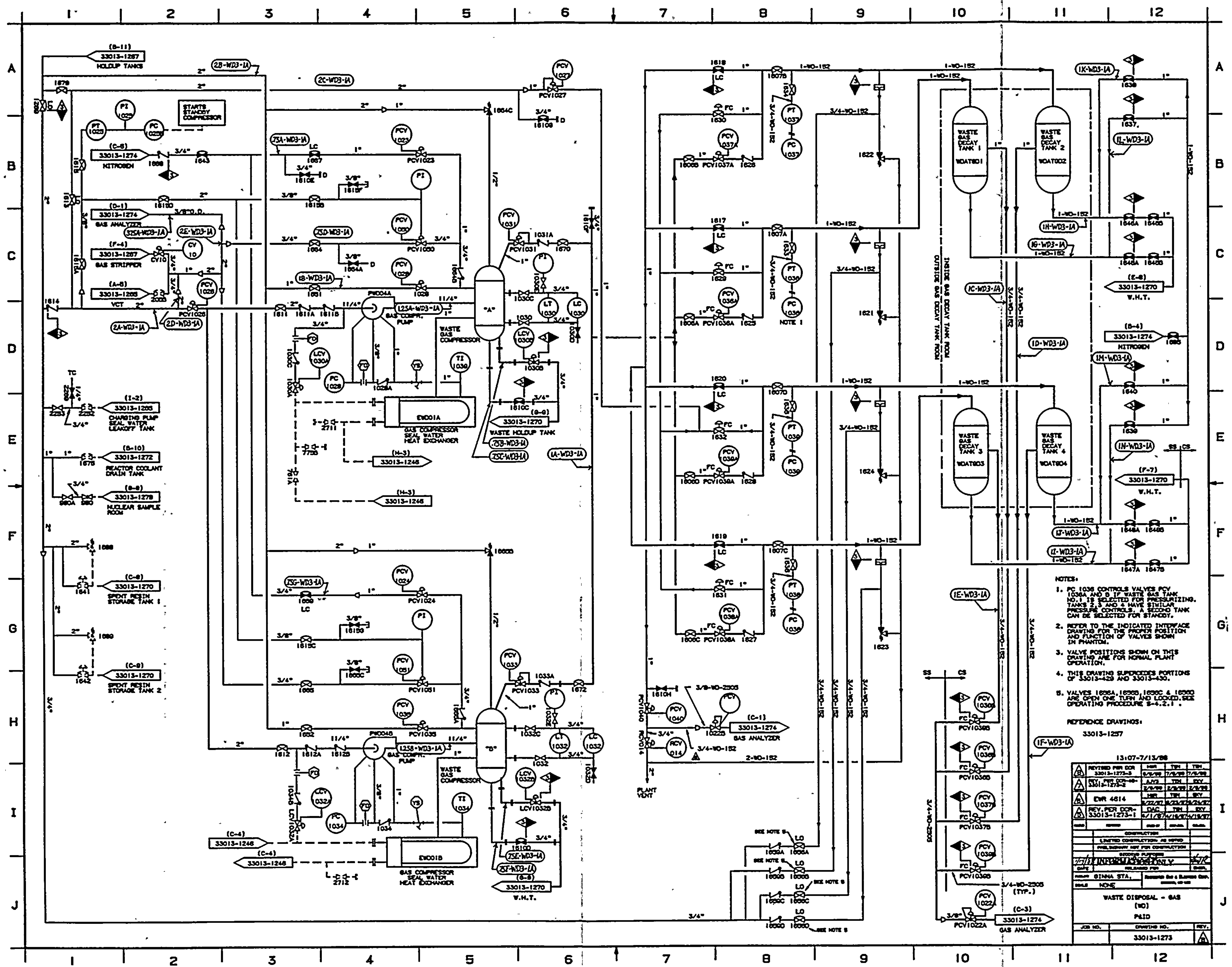

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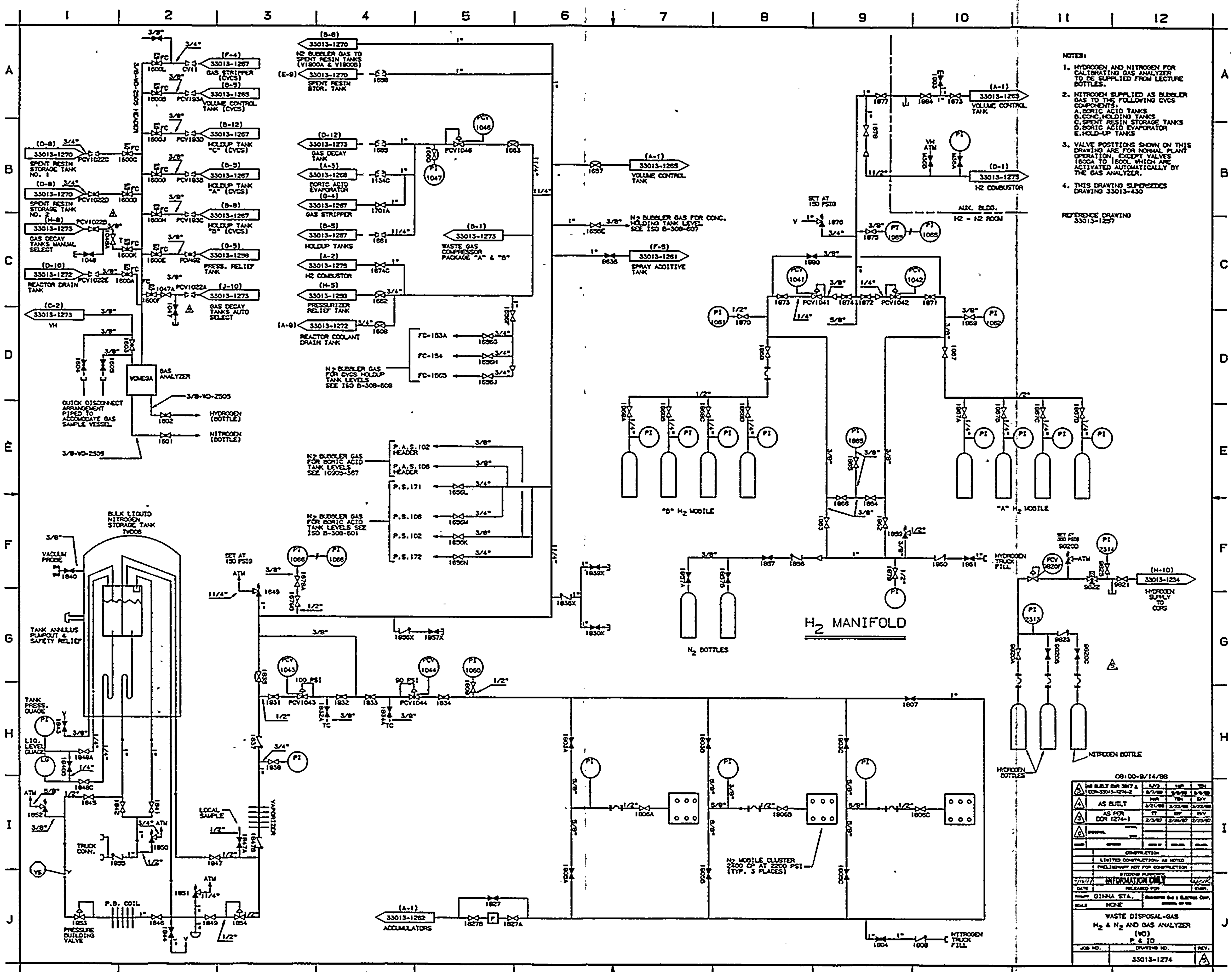

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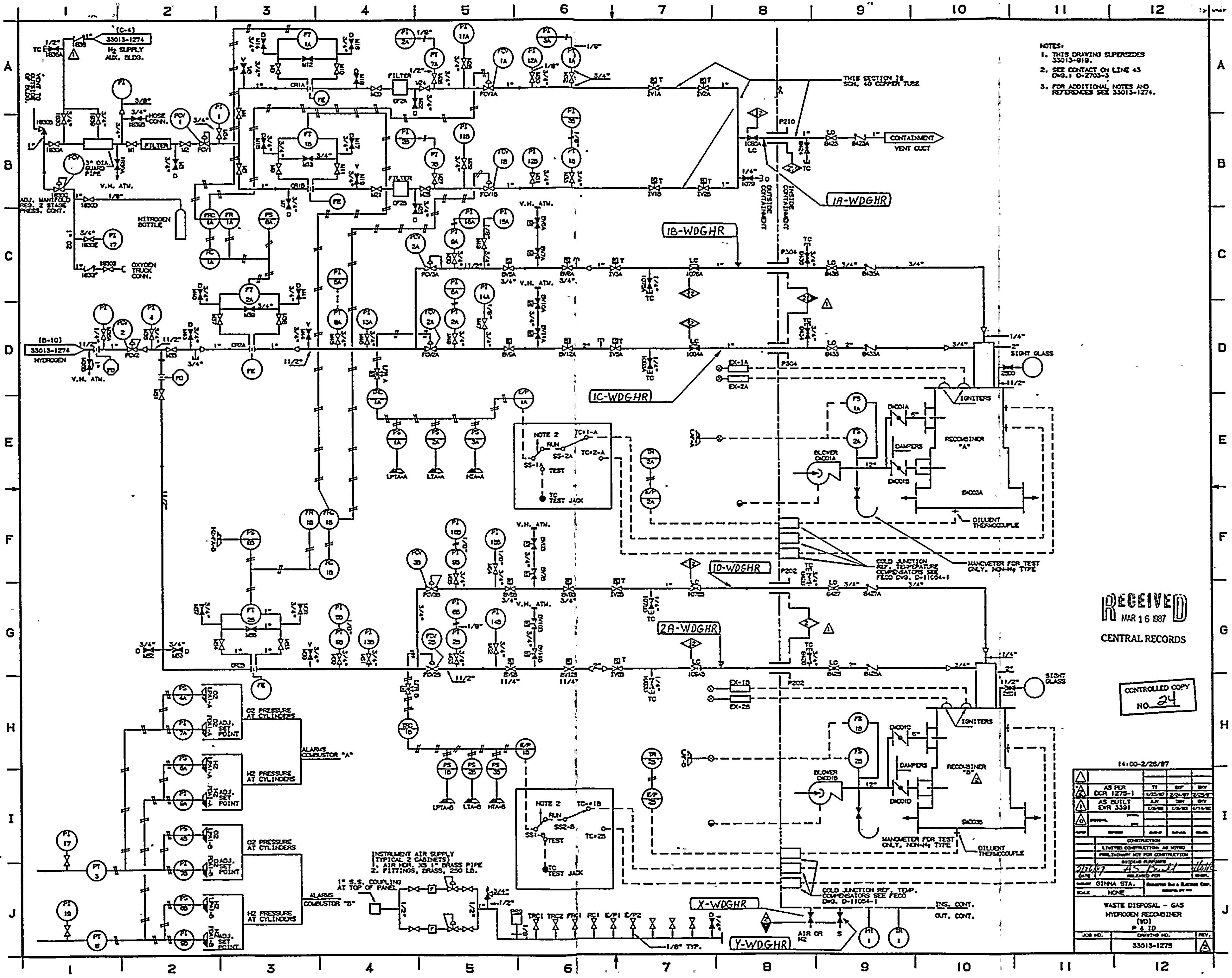

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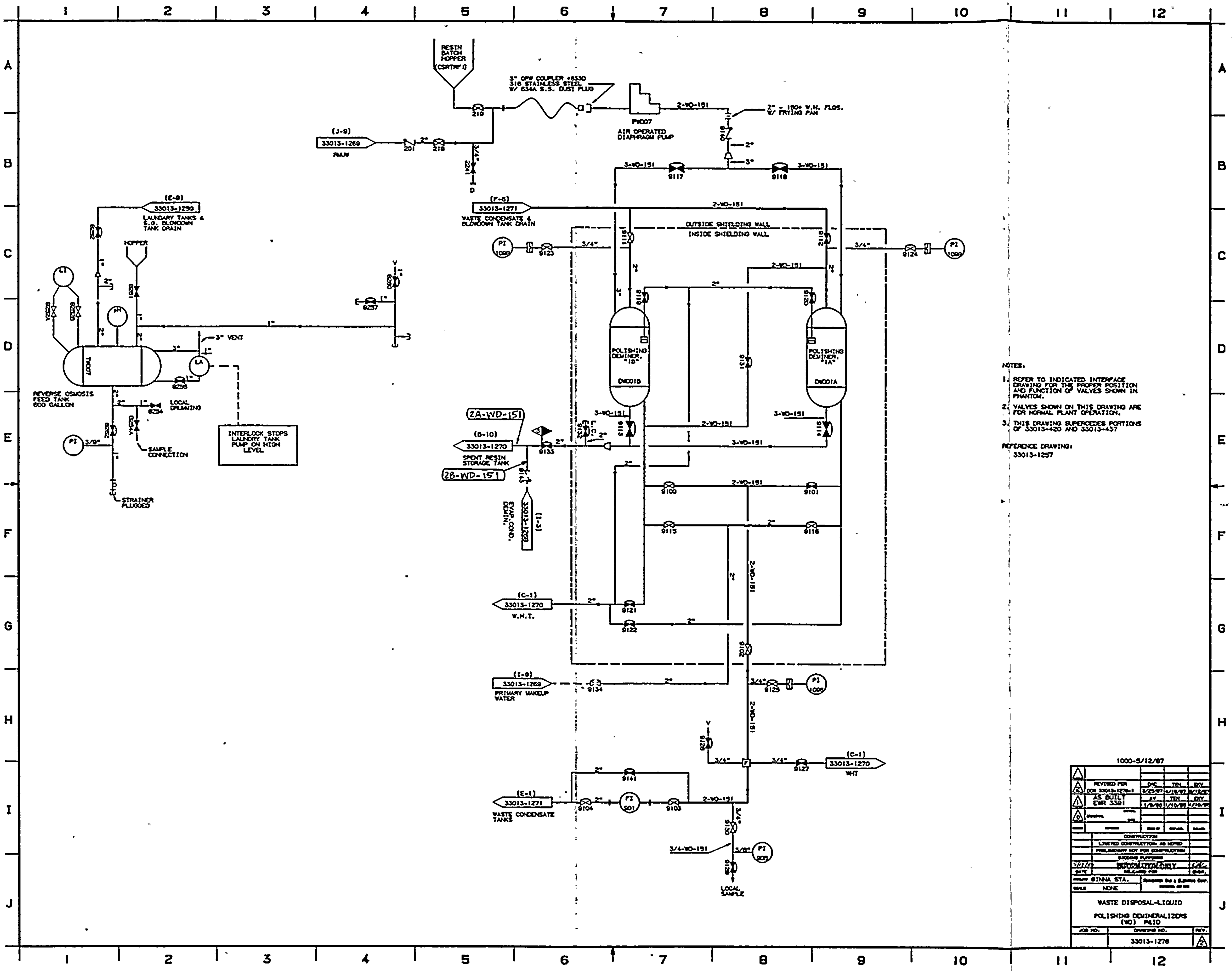

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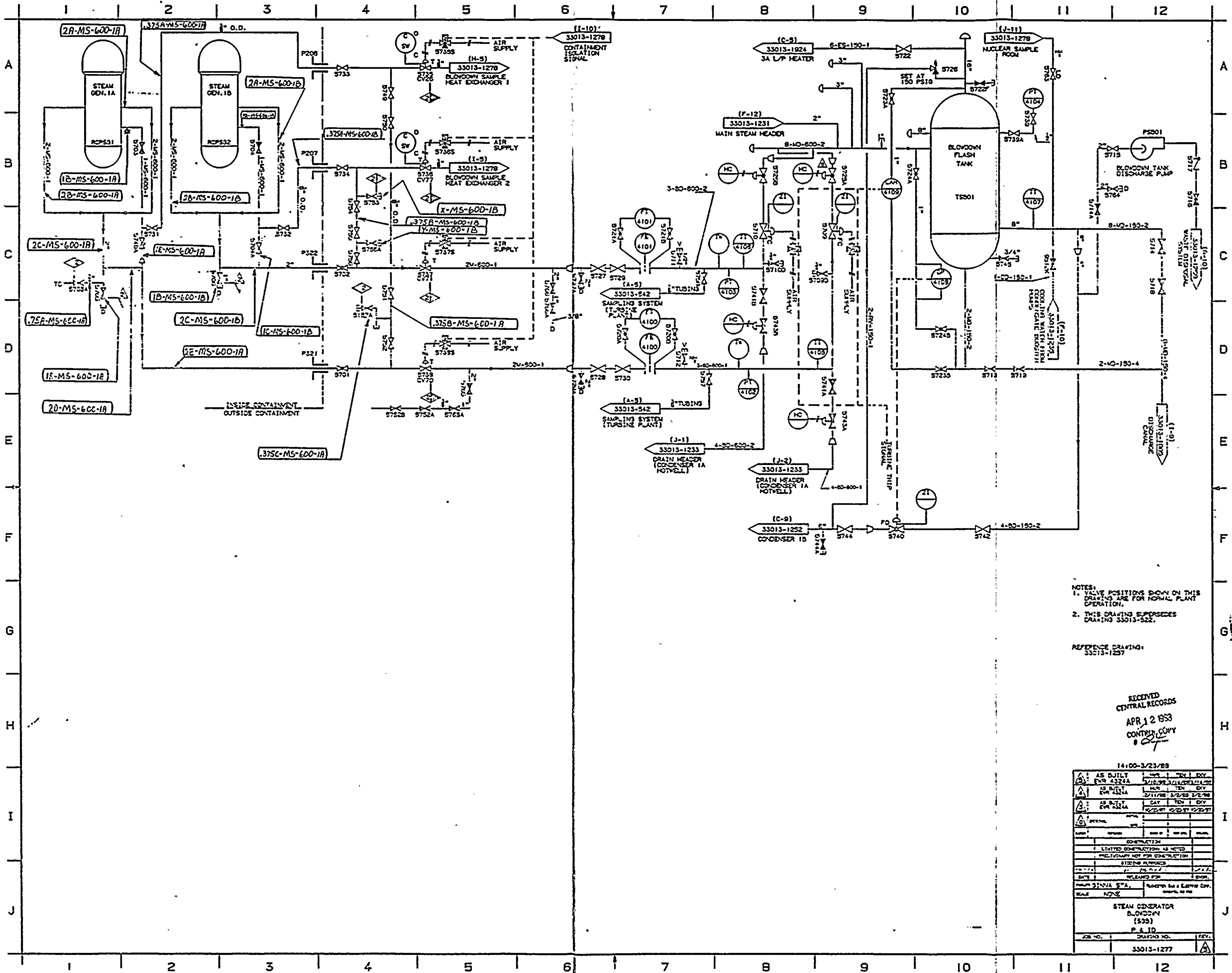

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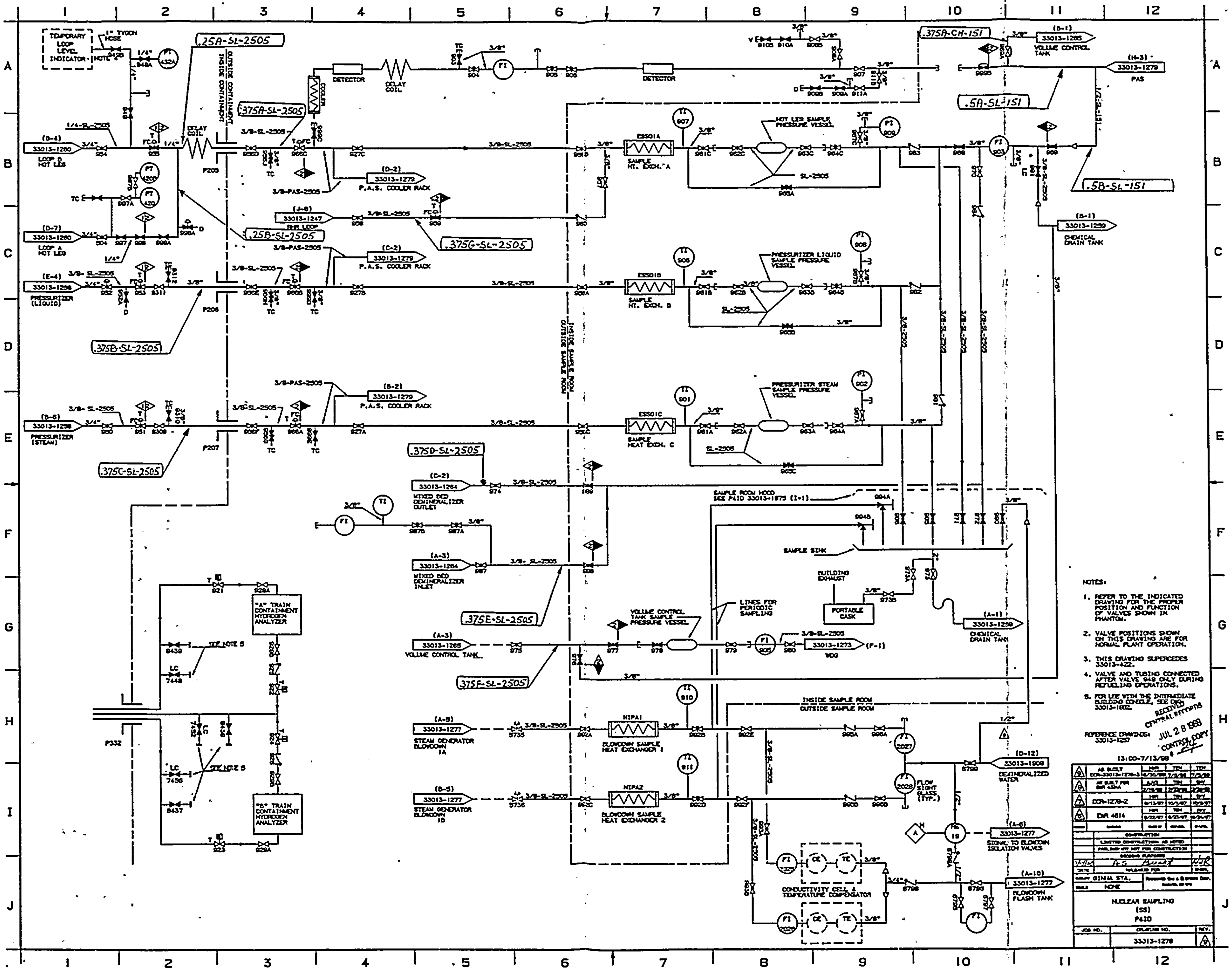

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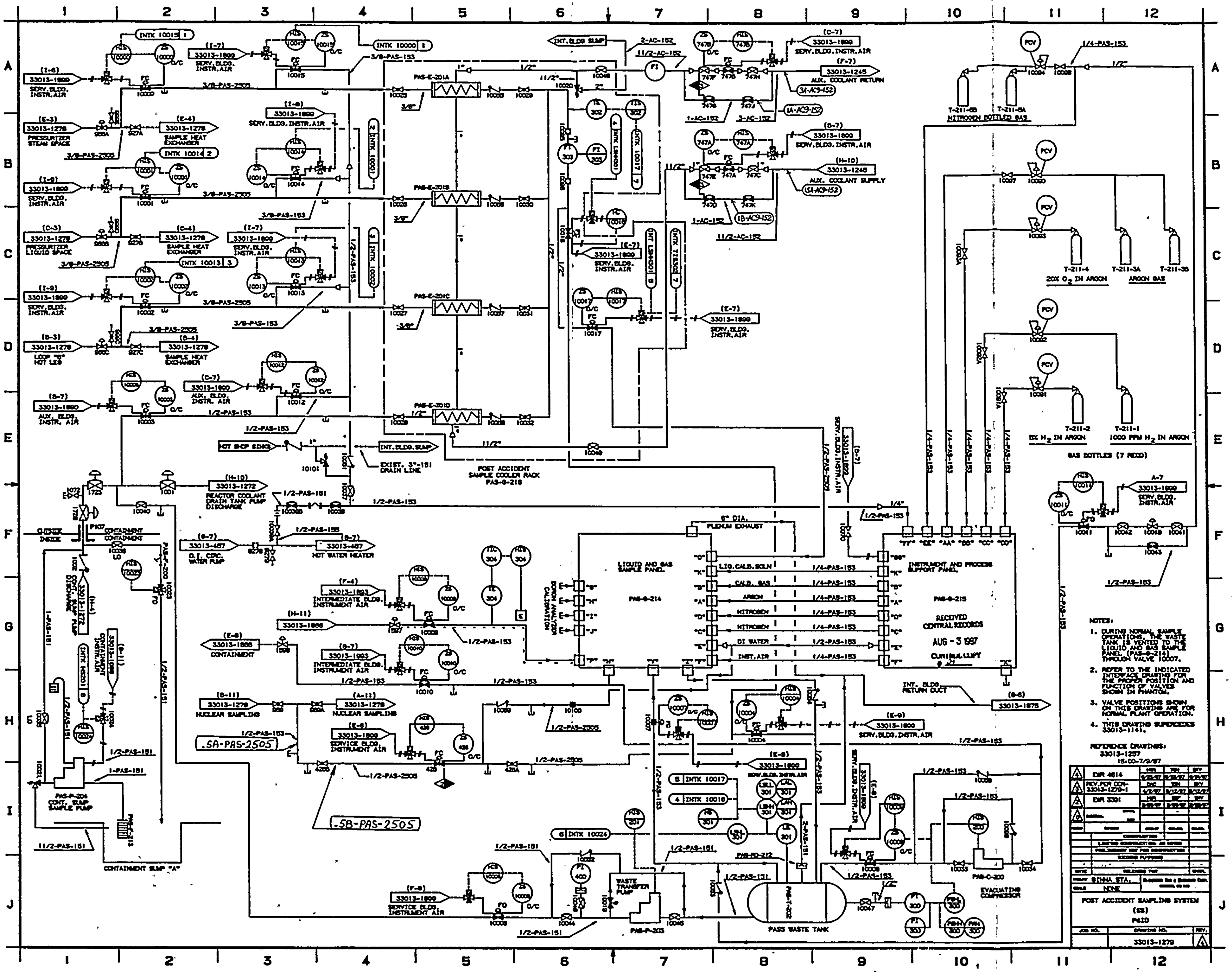

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SI APERTURE CARD

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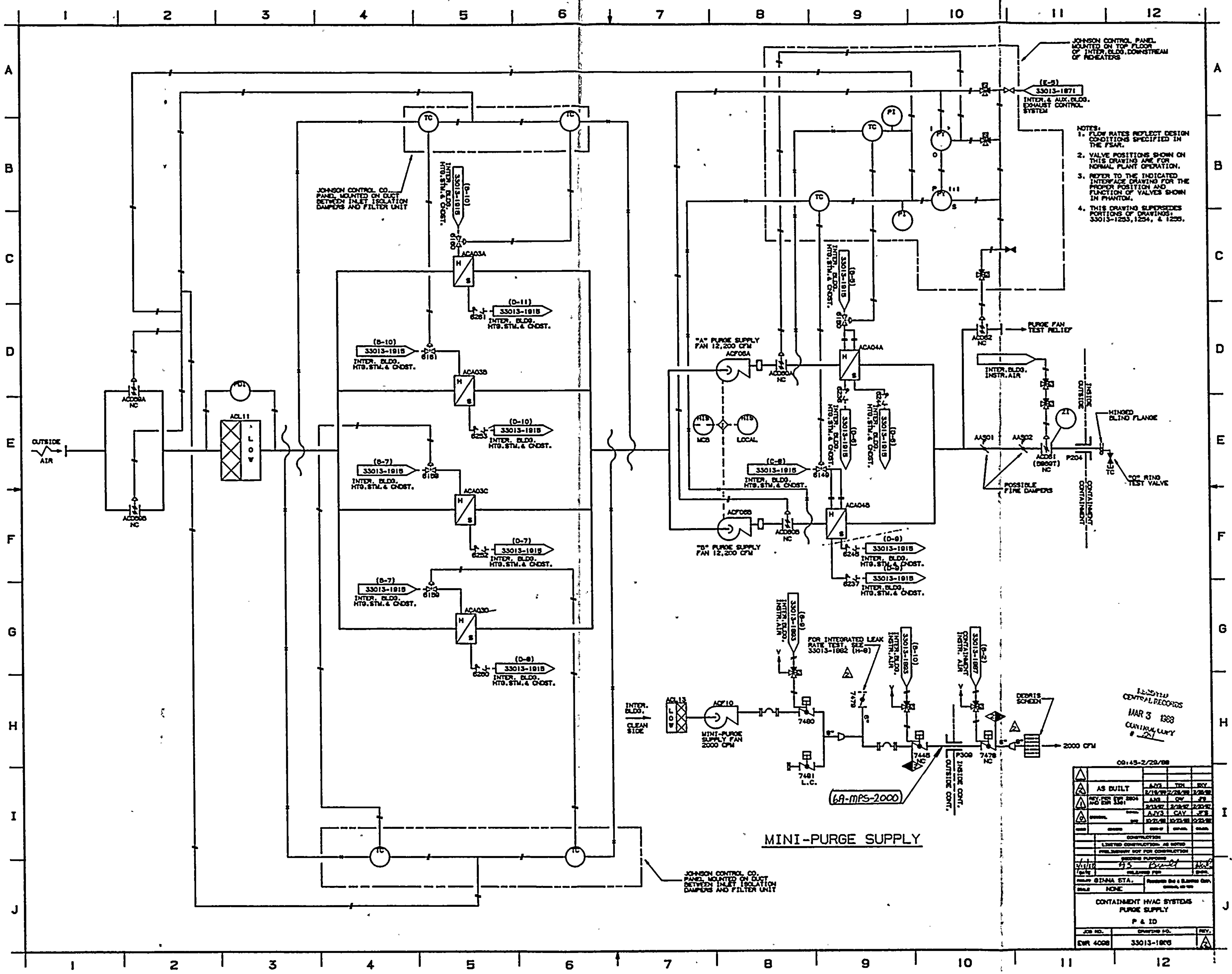

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SI APERTURE CARD

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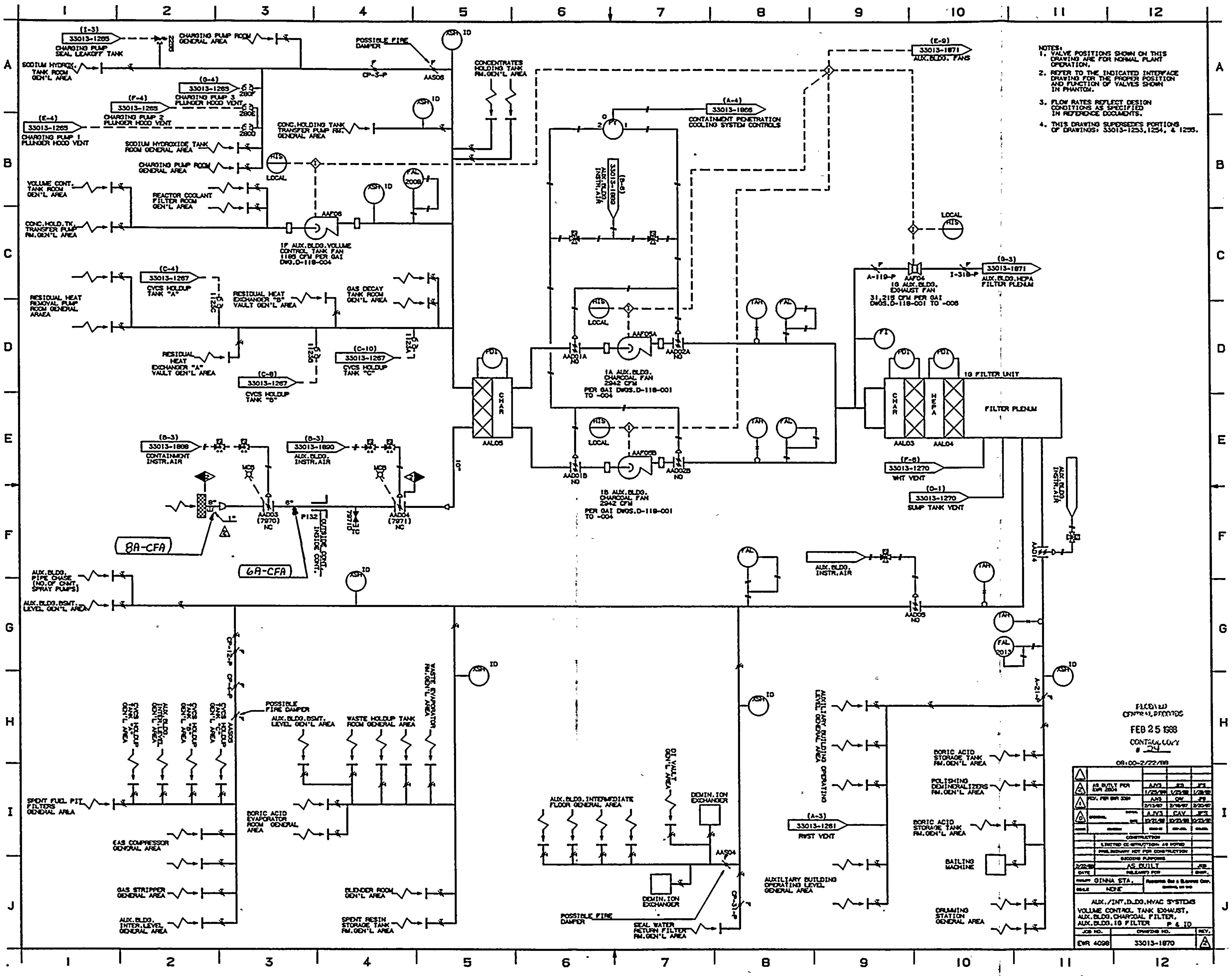

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QUALITY ASSURANCE MANUAL GINNA STATION	REV.	PAGE
ROCHESTER GAS & ELECTRIC CORPORATION	0	1 OF 32C
	EFFECTIVE DATE: January 1, 1990	

TITLE: ATTACHMENT C APPENDIX B GINNA NUCLEAR POWER STATION INSERVICE INSPECTION PROGRAM FOR THE 1990-1999 INTERVAL	SIGNATURE	DATE
PREPARED BY:	Michael J Saporito	7-20-89
QUALITY ASSURANCE REVIEW	C R Anderson	7-20-89
APPROVED BY:	John F. Smith	7/20/89

ATTACHMENT C

Allocation Tables

Introduction

In this Attachment, Allocation Tables have been grouped by Class 1, 2 and 3 components. These tables list the System or Line Number, ASME Examination Category, Examinations Required By Code, Examinations Selected By Rochester Gas and Electric and the Distribution of Examinations By Period during the Third Interval.

Allocation Tables were developed to meet the requirements of IWB, IWC and IWD concerning percentage of examinations completed during each of the three periods, as well as, the number of components that must be examined as indicated by their Associated Category Requirements.

The following Tables for Class 1, 2 and 3 provide this type of information.

GINNA ALLOCATIONS
CLASS 1

VESSELS

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Sched.
RPV	B-A	6	4	4			4	4
	B-D	12	12	12			12	12
	B-F	6	6	6			6	6
	B-G-1	148	145	145	48	48	49	145
	B-H	2	2	2			2	2
	B-N-1	1	1	1			1	1
	B-N-2	1	1	1			1	1
	B-N-3	1	1	1			1	1
	B-O	20	2	2			2	2
	F-A	4	4	4			4	4
Pressurizer	B-B	5	4	4	2	1	1	4
	B-D	5	5	5	2	2	1	5
	B-F	3	3	3	2	0	1	3
	B-G-2	16	16	16	16	16	16	48
	B-H	1	1	1	0	0	1	1
Steam Generators	B-B	2	1	1	1	0	0	1
	B-D	6	4	4	2	2	0	4
	B-F	4	4	4	2	2	0	4
	B-G-2	128	128	128	32	32	64	128
	F-B	8	8	8	0	4	4	8
	F-C	16	16	16	0	8	8	16
RHE	B-B	17	9	9	3	3	3	9
	B-D	3	3	3	0	2	1	3
	F-B	3	3	3	3	0	0	3
	F-C	3	3	3	3	0	0	3
Vessel Totals		421	386	386	116	120	182	418

[illegible]

GINNA ALLOCATIONS
CLASS 1

VESSELS

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Sched.
RPV	B-A	6	4	4	0	0	4	4
	B-D	12	12	12	0	0	12	12
	B-F	6	6	6	0	0	6	6
	B-G-1	148	145	145	48	48	49	145
	B-H	2	2	2	0	0	2	2
	B-N-1	1	1	1	0	0	1	1
	B-N-2	1	1	1	0	0	1	1
	B-N-3	1	1	1	0	0	1	1
	B-O	20	2	2	0	0	2	2
	F-A	4	4	4	0	0	4	4
Pressurizer	B-B	5	4	4	2	1	1	4
	B-D	5	5	5	2	2	1	5
	B-F	3	3	3	2	0	1	3
	B-G-2	16	16	16	16	16	16	48
	B-H	1	1	1	0	0	1	1
Steam Generators	B-B	2	1	1	1	0	0	1
	B-D	6	4	4	2	2	0	4
	B-F	4	4	4	2	2	0	4
	B-G-2	128	128	128	32	32	64	128
	F-B	8	8	8	0	4	4	8
	F-C	16	16	16	0	8	8	16
RHE	B-B	17	9	9	3	3	3	9
	B-D	3	3	3	0	2	1	3
	F-B	3	3	3	3	0	0	3
	F-C	3	3	3	3	0	0	3
Vessel Totals		421	386	386	116	120	182	418

GINNA ALLOCATIONS
CLASS 1 (CONT'D)

PIPING

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Sched.
29-RC-2501-A	B-J	3	0	0	0	0	0	0
31-RC-2501-A	B-J	11	2	2	1	0	1	2
	F-C	1	1	1	0	0	1	1
27.5-RC-2501-A	B-J	4	1	1	0	1	0	1
29-RC-2501-B	B-J	4	1	1	1	0	0	1
31-RC-2501-B	B-J	10	3	3	1	1	1	3
	F-C	1	1	1	0	0	1	1
27.5-RC-2501-B	B-J	4	1	1	1	0	0	1
10-RC8-2501	B-F	2	2	2	1	0	1	2
	B-J	2	0	0	0	0	0	0
	F-C	1	1	1	0	0	1	1
3A-RC8-2501-A	B-J	27	7	8	1	2	5	8
	B-K-1	1	1	1	1	0	0	1
	F-A	2	2	2	1	1	0	2
	F-C	9	9	9	3	3	3	9
3C-RC8-2501	B-J	12	3	3	0	2	1	3
	F-C	3	3	3	1	2	0	3
3A-RC8-2501-B	B-J	15	4	4	1	2	1	4
	B-K-1	1	1	1	0	1	0	1
	F-C	8	8	8	0	8	0	8
2-CH5-2501	B-J	22	5	6	2	1	3	6
	B-K-1	3	3	3	1	2	0	3
	F-A	2	2	2	0	2	0	2
	F-C	11	11	11	2	7	2	11

PIPING (Cont'd)

4-RC8-2501	B-F	1	1	1	0	0	1	1
	B-J	4	1	0	0	0	0	0
3A-RC8-2501	B-J	4	1	1	0	1	0	1
	F-C	1	1	1	0	1	0	1
3B-RC8-2501	B-J	4	1	1	0	0	1	1
	F-C	1	1	1	0	0	1	1
4A-RC8-2501-B	B-J	16	4	4	2	1	1	4
10A-AC7-2501-B	B-J	13	3	4	1	1	2	4
	B-K-1	1	1	1	0	1	0	1
	F-A	1	1	1	0	0	1	1
	F-C	1	1	1	0	0	1	1
10A-RC0-2501-A	B-J	3	0	0	0	0	0	0
10A-AC7-2501-A	B-J	19	4	4	1	2	1	4
	F-C	5	5	5	0	3	2	5
10A-SI2-2501-A	B-J	12	4	4	2	1	1	4
	F-A	2	2	2	0	1	1	2
	F-C	4	4	4	1	1	2	4
10A-SI2-2501-B	B-J	12	3	4	2	0	2	4
	B-K-1	1	1	1	0	1	0	1
	F-C	4	4	4	1	0	3	4
6A-RC-2501-B	B-J	4	1	1	1	0	0	1
	B-K-1	1	1	1	1	0	0	1
	F-A	1	1	1	1	0	0	1
4A-RC-2501-B	B-J	8	2	2	1	1	0	2
6A-RC-2501-A	B-J	3	1	1	1	0	0	1
	B-K-1	1	1	1	0	0	1	1
	F-A	1	1	1	0	0	1	1
4A-RC-2501-A	B-J	6	1	1	0	1	0	1

PIPING (Cont'd)

2A-SI2-2501	B-J	8	2	2	1	1	0	2
2C-SI2-2501	B-J	14	4	4	1	1	2	4
	F-C	2	2	2	0	1	1	2
2D-SI2-2501	B-J	6	1	2	0	0	2	2
2B-SI2-2501	B-J	8	2	2	1	1	0	2
	F-C	2	2	2	0	1	1	2
2B-RC0-2501-A	B-J	3	1	1	1	0	0	1
	B-K-1	1	1	1	0	0	1	1
2A-RC0-2501-B	B-J	10	3	3	1	1	1	3
	F-C	3	3	3	0	1	2	3
2A-CH4-2501	B-J	32	8	8	2	3	3	8
	B-K-1	2	2	2	0	1	1	2
	F-A	1	1	1	0	1	0	1
	F-C	8	8	8	2	2	4	8
2B-CH4-2501	B-J	32	8	8	3	3	2	8
	B-K-1	1	1	1	0	1	0	1
	F-C	3	3	3	0	3	0	3
2B-CH-2501	B-J	12	3	3	1	1	1	3
	F-C	1	1	1	0	1	0	1
2A-CH5-2501	B-J	40	10	10	3	3	4	10
	B-K-1	2	2	2	0	2	0	2
	F-A	4	4	4	1	1	2	4
	F-C	5	5	5	2	1	2	5
2B-RC0-2501-A	B-J	5	1	1	0	1	0	1
2C-RC0-2501-B	B-J	5	1	2	1	1	0	2
Piping Totals (All)		505	203	208	53	83	72	208
B-J Only		397	97	102	34	33	35	102

GINNA ALLOCATIONS
CLASS 1 (CONT'D)

PUMPS

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Sched.
Reactor Coolant Pumps	B-L-1	6	0	0	*	*	*	0
	B-L-2	2	0	0	*	*	*	0*
	B-G-1	96	96	96	0	48	48	96
	B-K-1	6	6	6	3	0	3	6
	Flywheel	2	0	6	2	2	2	6
	Anchor Bolts	168	0	168	0	168	0	168
	F-B	6	6	6	0	3	3	6
Pump Totals		286	108	282	5	221	56	282

* Reference: Relief Request Number 4

GINNA ALLOCATIONS
CLASS 1 (CONT'D)

VALVES

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Sched.
	B-M-1	5	1	1	0	0	1	1
	B-M-2	12	3	3	0	0	3	3*
	B-G-2	28	3	3	0	0	3	3*
Valve Totals		45	7	7	0	0	7	7

GINNA ALLOCATIONS
CLASS 2

VESSELS

System/Component	ASME Category	Total	Total Reqr'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Steam Generator	C-A	10	5	5	2	2	1	5
	C-B	8	4	4	0	2	2	4
	C-C	8	4	4	0	0	4	4
Residual Heat Exchanger	C-A	4	2	2	1	1	0	2
	C-B	10	5	5	3	1	1	5
	C-C	6	3	3	3	0	0	3
	F-A	6	3	3	3	0	0	3
	Augmented	2	0	2	1	1	0	2
Seal Water Inject. Filter	C-A	4	2	2	0	2	0	2
	C-C	6	3	3	1	1	1	3
	F-B	6	3	3	1	1	1	3
	Augmented	2	0	2	0	1	1	2
Seal Water Heat Exchanger	C-A	2	2	2	0	1	1	2
	C-C	3	3	3	1	1	1	3
	F-B	3	3	3	1	1	1	3
	Augmented	1	0	1	0	0	1	1
Nonregener. Heat Exchanger	C-A	1	1	1	0	1	0	1
	C-C	1	1	1	0	1	0	1
	F-A	3	3	3	0	3	0	3
	Augmented	1	0	1	1	0	0	1
Seal Water Return Filter	C-A	3	3	3	1	1	1	3
	C-C	4	4	4	1	2	1	4
	F-B	4	4	4	1	2	1	4
	Augmented	1	0	1	0	1	0	1
Reactor Cool. Filt.	C-A	3	3	3	1	0	2	3
	C-C	4	4	4	1	2	1	4
	F-B	4	4	4	1	2	1	4
	Augmented	1	0	1	0	1	0	1

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

VESSELS (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Pulse Dampener	C-A	4	4	4	1	2	1	4
	C-B	6	6	6	2	2	2	6
	C-C	2	2	2	1	0	1	2
	F-B	2	2	2	1	0	1	2
Containment Spray	F-B	2	1	1	0	1	0	1
Pumps	Augmented	4	0	4	2	2	0	4
Safety Injection	F-B	3	1	1	0	1	0	1
Pumps	Augmented	9	0	9	3	3	3	9
Resd. Heat Removal	F-B	6	3	3	0	0	3	3
Pumps	Augmented	4	0	4	0	1	3	4

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Main Steam Piping:								
30-A-MS-600-1A	C-F-2	53	6	17	6	5	6	17
	C-C	7	7	7	2	2	3	7
	F-B	5	5	5	3	0	2	5
	F-C	13	13	13	5	2	6	13
6A-MS-600-1A	C-F-2	4	1	1	1	0	0	1
6B-MS-600-1A	C-F-2	3	1	1	0	1	0	1
	C-C	1	1	0	0	1	0	1
	F-C	1	1	0	0	1	0	1
	Augmented	1	0	1	1	0	0	1
6C-MS-600-1A	C-F-2	1	0	0	0	0	0	0
	Augmented	1	0	1	0	0	1	1
6D-MS-600-1A	C-F-2	1	0	0	0	0	0	0
	Augumented	1	0	1	0	0	1	1
6E-MS-600-1A	C-F-2	1	1	1	0	0	1	1
	Augmented	1	0	1	0	1	0	1
6F-MS-600-1A	C-F-2	1	0	0	0	0	0	0
	Augmented	1	0	1	0	1	0	1
6G-MS-600-1A	C-F-2	1	0	0	0	0	0	0

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
30B-MS-600-1B	C-F-2	51	6	19	7	6	6	19
	C-C	14	14	14	4	4	6	14
	F-B	8	8	8	5	0	3	8
	F-C	23	23	23	8	4	11	23
6A-MS-600-1B	C-F-2	3	0	0	0	0	0	0
6B-MS-600-1B	C-F-2	3	0	0	0	0	0	0
	F-C	1	1	1	0	0	1	1
	Augmented	2	0	2	0	2	0	2
6C-MS-600-1B	C-F-2	1	1	1	0	1	0	1
	Augmented	1	0	1	0	0	1	1
6D-MS-600-1B	C-F-2	1	1	1	0	1	0	1
	Augmented	1	0	1	0	0	1	1
6E-MS-600-1B	C-F-2	1	0	0	0	0	0	0
	Augmented	1	0	1	0	0	1	1
6F-MS-600-1B	C-F-2	1	1	1	0	1	0	1
	Augmented	1	0	1	0	1	0	1
6G-MS-600-1B	C-F-2	1	0	0	0	0	0	0

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING . (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Low Pressure Safety Injection:								
4-SI-301	Augmented	10	0	2	1	0	1	2
	"	1	0	1	0	1	0	1
	"	2	0	2	1	1	0	2
	"	2	0	2	0	1	1	2
4A-SI-301	"	6	0	2	0	1	1	2
	"	1	0	1	1	0	0	1
	"	1	0	1	1	0	0	1
4B-SI-301	"	12	0	3	2	1	0	3
	"	1	0	1	0	0	1	1
	"	1	0	1	0	0	1	1
4C-SI-151	"	0	0	0	0	0	0	0
6-AC-601	C-C	1	1	1	0	1	0	1
	C-F-1	6	0	0	0	0	0	0
	F-B	3	3	3	0	2	1	3
	F-C	2	2	2	1	1	0	2
6A-AC-601	C-C	3	3	3	0	1	2	3
	C-F-1	17	1	1	1	0	0	1
	F-B	10	10	10	5	2	3	10
	F-C	1	1	1	1	0	0	1
6A-SI-151	Augmented	4	0	0	0	0	0	0
6B-SI-151	"	2	0	1	1	0	0	1

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
8A-SI-301B	Augmented	15	0	2	1	1	0	2
	"	2	0	2	1	0	1	2
8B-SI-301A	"	11	0	2	0	2	0	2
	"	1	0	1	0	0	1	1
8A-SI-151	"	4	0	2	1	1	0	2
	"	1	0	1	0	0	1	1
	"	1	0	1	0	0	1	1
8B-SI-151	"	4	0	2	0	1	1	2
	"	1	0	1	1	0	0	1
	"	1	0	1	1	0	0	1
8C-SI-301	"	9	0	1	0	0	1	1
	"	2	0	2	1	1	0	2
8D-SI-301	"	13	0	0	0	0	0	0
	"	1	0	1	1	0	0	1
	"	2	0	2	1	1	0	2
8E-SI-301	"	11	0	2	0	0	2	2
	"	1	0	1	0	1	0	1
	"	2	0	2	0	2	0	2
8F-SI-301	"	13	0	0	0	0	0	0
	"	1	0	1	0	0	1	1
	"	1	0	1	0	0	1	1
8G-SI-151	"	12	0	2	2	0	0	2
	"	2	0	2	0	1	1	2
	"	2	0	2	0	1	1	2

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
8H-SI-151	Augmented	13	0	1	1	1	0	1
	"	2	0	2	1	1	0	2
	"	3	0	3	1	1	1	3
	"	2	0	2	1	0	1	2
8K-SI-151	C-C	1	1	1	0	1	0	1
	C-F-1	2	1	1	0	0	1	1
	F-B	1	1	1	0	1	0	1
10-AC-601	C-F-1	10	3	4	0	3	1	4
	F-B	3	3	3	1	0	2	3
	F-C	1	1	1	0	1	0	1
10-SI-151	Augmented	20	0	3	1	1	1	3
	"	3	0	3	1	2	0	3
	"	3	0	3	1	2	0	3
	"	2	0	2	0	0	2	2

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Feedwater:								
14A-FW-900-1A	C-F-2	29	9	9	2	4	3	9
	C-C	3	3	3	1	2	0	3
	F-B	3	3	3	2	0	1	3
	F-C	7	7	7	4	2	1	7
14B-FW-900-1B	C-F-2	38	10	10	2	2	6	10
	C-C	5	5	5	2	1	2	5
	F-B	11	11	11	5	3	3	11
	F-C	14	14	14	4	2	8	14

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Containment Spray:								
6C-SI-301	C-F-1	3	1	3	1	1	1	3
6D-SI-301	C-F-1	5	2	2	1	1	0	2
6E-SI-301	C-C	1	1	1	1	0	0	1
	C-F-1	6	1	1	0	0	1	1
	F-B	1	1	1	1	0	0	1
6J-SI-301	C-F-1	3	0	2	1	1	0	2
6K-SI-301	C-C	1	1	1	0	0	1	1
	C-F-1	6	0	0	0	0	0	0
	F-B	1	1	1	0	0	1	1
6L-SI-301	C-F-1	5	1	1	0	1	0	1

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
CVCS Piping:								
8A-CH-2502	C-F-1	6	1	3	1	1	1	3
	C-C	2	2	2	0	2	0	2
	F-B	2	2	2	0	2	0	2
8B-CH-2502	C-F-1	6	1	4	2	1	1	4
	C-C	2	2	2	2	0	0	2
	F-B	2	2	2	2	0	0	2
8C-CH-2502	C-F-1	8	0	4	1	1	2	4
	C-C	2	2	2	0	0	2	2

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
High Pressure Safety Injection:								
2A-SI-1501	C-F-1	19	9	9	3	3	3	9
	C-C	1	1	1	0	1	0	1
	F-B	4	4	4	1	2	1	4
2B-SI-1501	C-F-1	17	9	9	3	3	3	9
	F-B	5	5	5	1	2	2	5
2C-SI-1501	C-F-1	21	9	9	2	5	2	9
	F-B	1	1	1	0	1	0	1
2D-SI-1501	C-F-1	19	9	9	3	3	3	9
	C-C	1	1	1	0	1	0	1
	F-B	2	2	2	0	2	0	2
3A-SI-1501	C-F-1	19	4	5	1	2	2	5
	C-C	2	2	2	0	2	0	2
	F-B	10	10	10	2	6	2	10
	F-C	1	1	1	0	1	0	1
3B-SI-1501	C-F-1	17	4	6	2	2	2	6
	C-C	2	2	2	0	0	2	2
	F-B	8	8	8	1	5	2	8
	F-C	1	1	1	0	0	1	1
3C-SI-1501	C-F-1	11	3	4	1	2	1	4
	C-C	1	1	1	1	0	0	1
	F-B	2	2	2	1	0	1	2

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
3D-SI-1501	C-F-1	12	4	4	2	1	1	4
	F-B	2	2	2	1	0	1	2
	C-C	1	1	1	1	0	0	1
3E-SI-1501	C-F-1	21	4	5	1	3	1	5
	C-C	2	2	2	1	1	0	2
	F-B	11	11	11	4	4	3	11
	F-C	1	1	1	1	0	0	1
4D-SI-1501	C-F-1	24	4	4	1	1	2	4
	F-B	8	8	8	3	3	2	8
	C-C	3	3	3	1	2	0	3
4E-SI-1501	C-F-1	33	5	5	1	3	1	5
	C-C	1	1	1	1	0	0	1
	F-B	10	10	10	3	4	3	10
	F-C	5	5	5	2	1	2	5

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
Residual Heat Removal Piping:								
6B-AC-601	C-C	2	2	2	0	1	1	2
	C-F-1	14	1	1	0	1	0	1
	F-B	3	3	3	0	2	1	3
	F-C	1	1	1	1	0	0	1
6C-AC-601	C-C	3	3	3	3	0	0	3
	C-F-1	31	1	3	1	1	1	3
	F-B	7	7	7	3	3	1	7
6D-AC-601	C-C	3	3	3	2	0	1	3
	C-F-1	21	1	2	0	2	0	2
	F-B	5	5	5	2	2	1	5
	F-C	2	2	2	1	0	1	2
6E-AC-601	Augmented	8	0	1	0	0	1	1
	"	1	0	1	0	1	0	1
	"	4	0	4	2	2	0	4
6K-AC-601	"	4	0	1	1	0	0	1
	"	1	0	1	0	1	0	1
	"	3	0	3	2	1	0	3
8-AC-601	C-F-1	4	0	0	0	0	0	0
8A-AC-601	C-F-1	12	1	3	1	1	1	3
	F-B	1	1	1	0	1	0	1
8B-AC-601	C-F-1	9	1	1	1	0	0	1
	F-B	1	1	1	0	0	1	1
	F-C	1	1	1	0	0	1	1

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
8C-AC-601	C-C	1	1	1	0	0	1	1
	C-F-1	23	2	3	1	1	1	3
	F-B	3	3	3	0	1	2	3
	F-C	2	2	2	0	1	1	2
8D-AC-601	C-F-1	5	1	2	1	1	0	2
8E-AC-601	C-F-1	7	1	3	1	1	1	3
8F-AC-601	Augmented	13	0	3	1	1	1	3
8FF-AC-601	"	13	0	3	1	1	1	3
	"	1	0	1	0	1	0	1
8X-AC-601	C-F-1	9	1	2	1	1	0	2
10-AC-601	C-F-1	20	1	4	1	1	2	4
	F-B	2	2	2	0	1	1	2
	F-C	2	2	2	2	0	0	2
10A-AC-601	C-F-1	24	2	7	3	2	2	7
	C-C	2	2	2	0	1	1	2
	F-B	9	9	9	2	1	6	9
10B-AC-601	C-F-1	12	1	4	2	0	2	4
	C-C	2	2	2	1	0	1	2
	F-B	4	4	4	1	2	1	4
	F-C	1	1	1	0	0	1	1
10C-AC-601	C-C	1	1	1	0	1	0	1
	C-F-1	8	1	3	1	1	1	3
	F-C	2	2	2	0	1	1	2

GINNA ALLOCATIONS
CLASS 2 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Required by ASME Section XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
10D-AC-601	C-C	1	1	1	0	0	1	1
	C-F-1	12	1	2	1	0	1	2
	F-B	1	1	1	0	1	0	1
	F-C	2	2	2	0	1	1	2
10E-AC-601	C-F-1	5	0	0	0	0	0	0
	F-B	2	2	2	0	1	1	2
	F-C	1	1	1	0	0	1	1
10EE-AC-601	Augmented	2	0	0	0	0	0	0
10G-AC-601	C-F-1	4	0	0	0	0	0	0
	F-C	3	3	3	2	1	0	3
10GG-AC-601	Augmented	2	0	0	0	0	0	0
	"	1	0	1	0	1	0	1
	"	1	0	1	0	1	0	0
10H-AC-601	C-C	3	3	3	1	0	2	3
	C-F-1	13	1	2	1	1	0	2
	F-B	5	5	5	2	0	3	5
Total Areas:		1439	526	731	235	253	243	731

GINNA ALLOCATIONS
CLASS 3

PIPING

System/Component	ASME Category	Total	Total Req'd by ASME Sect. XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
8B-AC5-152	D-B	1	1	1	1	0	0	1
	F-B	1	1	1	1	0	0	1
10A-AC5-152	F-A	2	0	0	0	0	0	0
8A-AC5-152	D-B	1	1	1	1	0	0	1
	F-B	1	1	1	1	0	0	1
10D-AC5-152	D-B	1	1	1	1	0	0	1
	F-A	1	1	1	1	0	0	1
6E-AC5-152	D-B	2	2	2	2	0	0	2
	F-C	8	2	2	2	0	0	2
6B-AC6-152	D-B	2	2	2	2	0	0	2
	F-A	1	1	1	1	0	0	1
	F-B	1	0	0	0	0	0	0
	F-C	8	1	1	1	0	0	1
10L-AC5-152	D-B	1	1	1	0	1	0	1
	F-A	1	1	1	0	1	0	1
	F-C	2	0	0	0	0	0	0
14B-AC5-152	D-B	2	2	2	0	2	0	2
	F-A	2	2	2	0	2	0	2
	F-C	5	0	0	0	0	0	0
10G-AC5-152	D-B	1	1	1	0	1	0	1
	F-A	2	1	1	0	1	0	1
10F-AC5-152	F-A	2	0	0	0	0	0	0
10E-AC5-152	F-C	1	0	0	0	0	0	0

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
8A-AC6-152	D-B	1	1	1	0	1	0	1
	F-A	1	0	0	0	0	0	0
	F-C	5	1	1	0	1	0	1
10J-AC5-152	D-B	1	1	1	0	1	0	1
	F-B	1	1	1	0	1	0	1
14C-AC5-152	D-B	2	2	2	0	2	0	2
	F-C	5	2	2	0	2	0	2
10H-AC5-152	D-B	1	1	1	0	1	0	1
	F-A	1	0	0	0	0	0	0
	F-C	1	1	1	0	1	0	1
10I-AC5-152	D-B	2	2	2	0	2	0	2
	F-A	1	1	1	0	1	0	1
	F-C	1	1	1	0	1	0	1
6C-AC6-152	D-B	2	2	2	0	2	0	2
	F-A	2	1	1	0	1	0	1
	F-B	1	1	1	0	1	0	1
	F-C	3	0	0	0	0	0	0
6G-AC6-152	D-B	2	2	2	0	2	0	2
	F-A	2	1	1	0	1	0	1
	F-C	6	1	1	0	1	0	1
8B-AC6-152	D-B	1	1	1	0	1	0	1
	F-A	1	1	1	0	1	0	1
	F-C	6	0	0	0	0	0	0
6D-AC6-152	F-C	4	0	0	0	0	0	0

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
6F-AC6-152	D-B	1	1	1	0	1	0	1
	F-A	1	1	1	0	1	0	1
	F-C	1	0	0	0	0	0	0
6H-AC6-152	D-B	1	1	1	0	1	0	1
	F-A	1	1	1	0	1	0	1
	F-C	1	0	0	0	0	0	0
20D-SWO-125-9	D-C	1	1	1	0	0	1	1
	F-A	6	1	1	0	0	1	1
	F-C	6	0	0	0	0	0	0
14F-SWO-125-9	D-C	1	1	1	0	0	1	1
	F-C	4	1	1	0	0	1	1
14G-SWO-125-9	D-C	1	1	1	0	0	1	1
	F-C	1	1	1	0	0	1	1
6E-SWO-125-9	D-C	2	2	2	0	0	2	2
	F-A	3	0	0	0	0	0	0
	F-C	8	2	2	0	0	2	2
6D-SWO-125-9	D-C	2	2	2	0	0	2	2
	F-A	3	2	2	0	0	2	2
14H-SWO-125-9	D-C	4	4	4	0	0	4	4
	F-A	3	0	0	0	0	0	0
	F-B	3	3	3	0	0	3	3
	F-C	2	1	1	0	0	1	1

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
20B-SWO-125-9	D-C	2	2	2	0	0	2	2
	F-A	2	0	0	0	0	0	0
	F-B	2	2	2	0	0	2	2
	F-C	1	0	0	0	0	0	0
20A-SWO-125-9	F-A	2	0	0	0	0	0	0
18A-SWO-125-9	F-C	1	0	0	0	0	0	0
141-SWO-125-9	F-A	2	0	0	0	0	0	0
	F-C	4	0	0	0	0	0	0
20A-SWO-125-9	D-C	2	2	2	2	0	0	2
	F-A	1	0	0	0	0	0	0
	F-B	2	2	2	2	0	0	2
	F-C	3	0	0	0	0	0	0
6C-SWO-125-9	D-C	1	1	1	1	0	0	1
	F-A	4	0	0	0	0	0	0
	F-B	1	0	0	0	0	0	0
	F-C	5	1	1	1	0	0	1
10B-SWO-125-9	D-B	2	2	2	0	2	0	2
	F-A	3	0	0	0	0	0	0
	F-B	2	2	2	0	2	0	2
	F-C	1	0	0	0	0	0	0
8D-SWO-125-9B	D-B	1	1	1	0	1	0	1
	F-C	2	1	1	0	1	0	1
10A-SWO-125-9B	F-C	1	0	0	0	0	0	0

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
14J-SWO-125-9B	D-B	2	2	2	1	0	1	2
	F-A	1	1	1	0	0	1	1
	F-C	6	1	1	1	0	0	1
8G-SWO-125-9B	F-C	1	0	0	0	0	0	0
8F-SWO-125-9B	F-C	1	0	0	0	0	0	0
8E-SWO-125-9B	D-B	1	1	1	0	1	0	1
	F-C	2	1	1	0	1	0	1
16C-SWO-125-9	D-B	2	2	2	1	0	1	2
	F-A	3	2	2	1	0	1	2
	F-C	11	0	0	0	0	0	0
20B-SWO-125-9	D-B	1	1	1	0	1	0	1
	F-C	8	1	1	0	1	0	1
14E-SWO-125-9	D-B	2	2	2	0	2	0	2
	F-A	2	1	1	0	1	0	1
	F-C	6	1	1	0	1	0	1
6H-SWO-125-9	D-B	1	1	1	0	1	0	1
	F-C	3	1	1	0	1	0	1
8C-SWO-125-9	F-C	1	0	0	0	0	0	0
8A-SWO-125-9	F-C	2	0	0	0	0	0	0
6H-SWO-125-9	D-B	3	3	3	0	0	3	3
	F-C	8	3	3	0	0	3	3

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	2	3	Total Scheduled
16D-SW0-125-9	D-B	1	1	1	0	0	1	1
	F-A	1	0	0	0	0	0	0
	F-B	1	1	1	0	0	1	1
20A-SW0-125-9	D-B	5	5	5	3	0	2	5
	F-A	5	5	5	3	0	2	5
20B-SW0-125-9	D-B	2	2	2	0	0	2	2
	F-A	2	2	2	0	0	2	2
14D-SW0-125-9	D-B	3	3	3	0	0	3	3
	F-A	3	3	3	0	0	3	3
14C-SW0-125-9	D-B	3	3	3	0	1	2	3
	F-A	3	3	3	0	1	2	3
14B-SW0-125-9	D-B	3	3	3	1	1	1	3
	F-A	3	3	3	1	1	1	3
14A-SW0-125-9	D-B	3	3	3	2	1	0	3
	F-A	3	3	3	2	1	0	3
8I-SW0-125-9	D-B	2	2	2	2	0	0	2
	F-A	3	2	2	2	0	0	2
6B-SW0-125-9	D-B	3	3	3	3	0	0	3
	F-A	3	1	1	1	0	0	1
	F-C	8	2	2	2	0	0	2
6G-SW0-125-9	F-C	1	0	0	0	0	0	0

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
3A-FW7-900-1B	D-B	6	6	6	3	3	0	6
	F-A	7	5	5	3	2	0	5
	F-C	20	1	1	0	1	0	1
3D-FW7-900-1	D-B	3	3	3	2	1	0	3
	F-A	4	3	3	2	1	0	3
	F-C	2	0	0	0	0	0	0
3E-FW7-900-1	D-B	1	1	1	1	0	0	1
	F-A	4	1	1	1	0	0	1
	F-C	5	0	0	0	0	0	0
3C-FW7-900-1	F-A	1	0	0	0	0	0	0
	F-C	2	0	0	0	0	0	0
3A-FW7-900-1A	D-B	5	5	5	1	2	2	5
	F-A	9	4	4	1	1	2	4
	F-C	4	1	1	0	1	0	1
	Augmented	3	0	3	0	1	2	3
5A-FW7-900-1	D-B	2	2	2	0	0	2	2
	F-A	3	1	1	0	0	1	1
	F-C	5	1	1	0	0	1	1
	F-B	1	0	0	0	0	0	0
3H-FW7-900-1	D-B	1	1	1	0	0	1	1
	F-B	1	1	1	0	0	1	1
	F-C	3	0	0	0	0	0	0
3J-FW7-900-1	D-B	3	3	3	1	1	1	3
	F-A	7	2	2	0	1	1	2
	F-C	7	1	1	1	0	0	1
	Augmented	1	0	1	0	1	0	1

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
3F-FW7-900-1	D-B	1	1	1	0	0	1	1
	F-B	1	1	1	0	0	1	1
	F-C	3	0	0	0	0	0	0
3G-FW7-900-1	D-B	2	2	2	1	0	1	2
	F-A	7	2	2	1	0	1	2
	F-C	6	0	0	0	0	0	0
	Augmented	2	0	2	0	0	2	2
3A-FW8-902-A	D-B	4	4	4	2	0	2	4
	F-A	12	4	4	2	0	2	4
	F-C	10	0	0	0	0	0	0
	Augmented	1	0	1	1	0	0	1
3C-FW8-902-A	D-B	1	1	1	0	0	1	1
	F-A	10	1	1	0	0	1	1
3A-FW8-902-B	D-B	5	5	5	0	0	5	5
	F-A	35	5	5	0	0	5	5
4A-SWO-125-1B	Augmented	3	0	3	0	0	3	3
4D-CD-150-1	Augmented	2	0	2	0	0	2	2
4C-CD-150-1	Augmented	2	0	2	2	0	0	2
4A-FW8-152-A	D-B	5	5	5	3	1	1	5
	F-A	28	5	5	3	1	1	5
	F-C	3	0	0	0	0	0	0
4A-FW8-152-B	D-B	5	5	5	3	1	1	5
	F-A	19	5	5	3	1	1	5
	F-C	2	0	0	0	0	0	0

[illegible]

[illegible]

[illegible]

GINNA ALLOCATIONS
CLASS 3 (CONT'D)

PIPING (CONT'D)

System/Component	ASME Category	Total	Total Req'd by ASME Sect.XI	Total Selected by RGE	Period 1	Period 2	Period 3	Total Scheduled
4B-FW8-152	D-B	3	3	3	1	2	0	3
	F-A	6	3	3	1	2	0	3
6C-MS-600-1	D-B	3	3	3	1	1	1	3
	F-A	2	2	2	0	1	1	2
	F-B	1	1	1	1	0	0	1
	F-C	1	0	0	0	0	0	0
6B-MS-600-1	D-B	6	6	6	2	2	2	6
	F-A	5	5	5	2	2	1	5
	F-C	6	1	1	0	0	1	1
6A-MS-600-1	D-B	4	4	4	2	1	1	4
	F-A	3	2	2	1	0	1	2
	F-C	6	2	2	1	1	0	2
2A-FW7-900-1A	F-C	2	0	0	0	0	0	0
2B-FW7-900-1A	F-C	4	0	0	0	0	0	0
2A-FW7-900-1B	F-C	2	0	0	0	0	0	0
2B-FW7-900-1B	F-C	3	0	0	0	0	0	0
Total Areas		650	282	296	95	94	107	296