

ATTACHMENT A

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages.

REMOVE

3.5-8
3.5-15
4.1-7

INSERT

3.5-8
3.5-15
4.1-7
4.1-7a

8802030538, 880126
PDR ADDCK 05000244
P PDR

TABLE 3.5-1 (Continued)
PROTECTION SYSTEM INSTRUMENTATION

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u>	<u>2</u>	<u>3</u>	<u>4</u>	<u>5</u>	<u>6</u>
		<u>TOTAL NO. of CHANNELS</u>	<u>NO. of CHANNELS TO TRIP</u>	<u>MIN. OPERABLE CHANNELS</u>	<u>PERMISSIBLE BYPASS CONDITIONS</u>	<u>OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 3 CANNOT BE MET</u>	<u>CHANNEL OPERABLE ABOVE</u>
19.	Degraded Voltage 480V Safeguards Bus	2/bus	2/bus	1/bus		7	$T_{RCS} = 350^{\circ}\text{F}$
20.	Automatic Trip Logic Including Reactor Trip Breakers	2	1	2	Note 4	14	When RCCA is withdrawn

NOTE 1: When block condition exists, maintain normal operation.

NOTE 2: Channels should be operable at all modes below the bypass condition with the reactor trip system breakers in the closed position and control rod drive system capable of rod withdrawal.

NOTE 3: Channels shall be operable at all modes below the bypass condition except during refueling defined to be when fuel is in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.

NOTE 4: One reactor trip breaker may be bypassed for surveillance testing provided the other reactor trip breaker is operable.

F.P. = Full Power

12. With the number of operable channels less than the Total Number of Channels, operation may proceed provided the inoperable channel is placed in the tripped condition within 1 hour. Should the next Channel Functional Test require the bypass of an inoperable channel to avoid the generation of an actuation signal, operation may proceed until this Channel Functional Test. At the time of this Channel Functional Test, or if at any time the number of operable channels is less than the Minimum Operable Channels required, be at hot shutdown within 6 hours and at an RCS temperature less than 350°F within 6 hours.
13. With the number of operable channels less than the Minimum Operable Channels required, operation may continue provided the containment purge and exhaust valves are maintained closed.
14. Should one reactor trip breaker or channel of trip logic be inoperable the plant must not be in the operating mode following a six hour time period. If one of the diverse reactor trip breaker trip features (undervoltage or shunt trip attachment) on one breaker is inoperable, restore it to operable status within 48 hours or declare breaker inoperable. If at the end of the 48 hour period one trip feature is inoperable it must be repaired or the plant must not be in the operating mode, and the reactor trip breaker must be open, following an additional six hour time period.

TABLE 4.1-1 (CONTINUED)

Proposed	Channel Description	Check	Calibrate	Test	Remarks
4.1-7	25. Containment Pressure	S	R	M	Narrow range containment pressure (-3.0, +3 psig) excluded
	26. Steam Generator Pressure	S	R	M	
	27. Turbine First Stage Pressure	S	R	M	
	28. Emergency Plan Radiation Instruments	M	R	M	
	29. Environmental Monitors	M	NA	NA	
	30. Loss of Voltage/Degraded Voltage 480 Volt Safeguards Bus	NA	R	M	
	31. Trip of Main Feedwater Pumps	NA	NA	R	
	32. Steam Flow	S	R	M	
	33. T _{AVA}	S	R	M	
	34. Chlorine Detector, Control Room Air Intake	NA	R	M	
	35. Ammonia Detector, Control Room Air Intake	NA	R	M	
	36. Radiation Detectors, Control Room Air Intake	NA	R	M	
	37a. Trip Breaker Logic Channel Testing	NA	NA	M	Notes 1, 2 and 3
	37b. Trip Breaker Logic Channel Testing	NA	NA	R	Note 1

Proposed

4.1-7

Amendment No. 8

TABLE 4.1-1 (CONTINUED)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
38. Reactor Trip Breakers	NA	NA	M	Function test - Includes independent testing of both undervoltage and shunt trip attachment of reactor trip breakers. Each of the two reactor trip breakers will be tested on alternate months. Note 2
39. Manual Reactor Trip	NA	NA	R	Includes independent testing of both undervoltage and shunt trip circuits. The test shall also verify the operability of the bypass breaker.
40a. Reactor Trip Bypass Breaker	NA	NA	M	Using test switches in the reactor protection rack manually trip the reactor trip bypass breaker using the shunt trip coil.
40b. Reactor Trip Bypass Breaker	NA	NA	R	Automatically trip the under-voltage trip attachment.
NOTE 1: Logic trains will be tested on alternate months corresponding to the reactor trip breaker testing. Monthly logic testing will verify the operability of all sets of reactor trip logic actuating contacts on that train (See Note 3). Refueling shutdown testing will verify the operability of all sets of reactor trip actuating contacts on both trains. In testing, operation of one set of contacts will result in a reactor trip breaker trip; the operation of all other sets of contacts will be verified by the use of indication circuitry.				
NOTE 2: Testing shall be performed monthly, unless the reactor trip breakers are open, or shall be performed prior to startup if testing has not been performed within the last 30 days.				
NOTE 3: The source range trip logic may be excluded from monthly testing provided it is tested within 30 days prior to startup.				

ATTACHMENT B

This proposed Technical Specification change reflects the request made in the January 9, 1987 NRC letter from Dominic C. DiIanni to Roger W. Kober concerning the "Review of Design for Automatic Shunt Trip for Reactor Trip Breakers (Generic Letter 83-28 Item 4.3)". The proposed changes incorporate on-line reactor trip breaker testing into the Technical Specifications. A detailed description of all the changes is presented on Table 1.

This change in Technical Specifications clearly is not a request for a change in the authorized power level as it is related only to operational and testing requirements of the reactor trip breakers within the plant. Further, the major possible consequence of this change that can be foreseen at this time is that over a period of time additional plant trips may occur through actions taken during monthly testing of the reactor trip breakers. However, occasional trips are considered within the normal operating conditions at a power plant.

A significant hazards analysis has been accomplished against the criteria in 10CFR50.92. Because the proposed changes constitute additional limitations and control on both operation and testing they do not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) involve a significant reduction in a margin of safety.

Therefore, Rochester Gas and Electric submits that the issues associated with this amendment request warrant a no significant hazards finding.

TABLE 1

Detailed Technical Specification Changes

<u>Location</u>	<u>Description of Change</u>	<u>Reason for Change</u>
p. 3.5-8, Table 3.5-1 Page 3 of 3	Add Item 20 - Automatic Trip Logic including Reactor Trip Breakers.	This change reflects new operating condition requirements for the Reactor Trip Breakers and reactor trip logic.
	Change Note 3 - the fifth word of sentence should be changed from "of" to "at".	Correct typographical error.
	Add Note 4 to the bottom of this page.	This Note is needed to explain the bypass condition in the new Item 20.
p. 3.5-15 Action Statement	Add Number 14 to the bottom of this page.	This note is required to explain fully the operator actions required if the requirement of column 3 for line 20 of Table 3.5-1 cannot be met.
p. 4.1-7 Table 4.1-1	Add Items 37a and 37b concerning logic testing on a monthly and refueling outage basis.	This addition reflects new testing requirements in response to NRC letter 83-28.
p. 4.1-7a Table 4.1-1	Add Item 38 concerning the testing of Reactor Trip Breakers.	This addition reflects new testing requirements in response to NRC letter 83-28.
	Add Item 39 concerning the testing of the manual tripping of the Reactor Trip Breakers.	This addition reflects new testing requirements in response to NRC letter 83-28.
	Add Items 40a and 40b concerning the testing of the Reactor Trip Bypass Breaker.	This addition reflects new testing requirements in response to NRC letter 83-28.
	Add Notes 1, 2, 3	These notes are required to explain the testing requirements of the logic associated with the Reactor Trip Breakers.

