

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8801060249 DOC. DATE: 87/12/30 NOTARIZED: NO DOCKET #
 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
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 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-006-00: on 871130, standby auxiliary feedwater pump 1D
 inoperable for longer than tech specs allow. Caused by
 cooling Unit 1B ac electrical breaker in off position.
 Breaker locked on & new modules to be designed. W/871230 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 8
 TITLE: 50.73 Licensee Event Report, (LER), Incident Rpt, etc.

NOTES: License Exp date in accordance with 10CFR2.2.109(9/19/72). 05000244

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	AEOD/DOA	1 1		AEOD/DSP/NAS	1 1
	AEOD/DSP/ROAB	2 2		AEOD/DSP/TPAB	1 1
	ARM/DCTS/DAB	1 1		DEDRO	1 1
	NRR/DEST/ADS	1 0		NRR/DEST/CEB	1 1
	NRR/DEST/ELB	1 1		NRR/DEST/ICSB	1 1
	NRR/DEST/MEB	1 1		NRR/DEST/MTB	1 1
	NRR/DEST/PSB	1 1		NRR/DEST/RSB	1 1
	NRR/DEST/SQB	1 1		NRR/DLPQ/HFB	1 1
	NRR/DLPQ/QAB	1 1		NRR/DOEA/EAB	1 1
	NRR/DREP/RAB	1 1		NRR/DREP/RPB	2 2
	NRR/DRIS/SIB	1 1		NRR/PMAS/ILRB	1 1
	<u>REG FILE</u> 02	1 1		RES DEPY GI	1 1
	RES TELFORD, J	1 1		RES/DE/EIB	1 1
	RGN1 FILE 01	1 1			
EXTERNAL:	EG&G GROH, M	5 5		FORD BLDG HOY, A	1 1
	H ST LOBBY WARD	1 1		LPDR	1 1
	NRC PDR	1 1		NSIC HARRIS, J	1 1
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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R.E. Ginna Nuclear Power Plant	0 5 0 0 0 2 4 4	8 7	— 0 0 6	— 0 0	0 2	OF	0 7

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. PRE-EVENT PLANT CONDITIONS

The unit was at 100% Reactor Power and the Instrument and Control (I&C) and Electrical Departments were trouble shooting, per Maintenance Work Request 87-6070, The Standby Auxiliary Feedwater Pump (SAFWP) Room Cooling unit 1B for a problem with an inoperable green light (OFF) bulb. The bulb had been replaced but there was still no indication.

II. DESCRIPTION OF EVENT

A. EVENT:

On November 30, 1987 at 1330 EST, the electrician trouble shooting the SAFWP Room 1B Cooling Unit green light (OFF) indication problem found the AC electrical breaker for the above cooling unit in the OFF position. The SAFWP Room 1B Cooling Unit AC Electrical Breaker in the OFF position rendered the 1B Cooling unit inoperable and based upon the Technical Specification (TS) definition of operable - operability, also rendered the 1D SAFWP inoperable as this was the pump that the cooling unit served.

A review of plant records indicates that the SAFWP Room Cooling Unit 1B was verified operable per operating procedure S-30.5 on November 18, 1987. Because the date that the 1B Cooling Unit AC Electrical Breaker was inadvertently turned off could not be established, Rochester Gas and Electric Corporation (RG&E) conservatively assumed that it was off since the last date that it was confirmed ON. TS 3.4.1(e) requires two Standby Auxiliary Feedwater Pumps and associated flow paths operable when Reactor Coolant System (RCS) temperature is at or above 350°F.

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TS 3.4.2(d) allows one Standby Auxiliary Feedwater Pump or flow path to be inoperable as long as the inoperable pump or flow path is restored to operable status within 7 days or be in hot shutdown within the next 6 hours and at an RCS temperature below 350°F within the following 6 hours. RG&E has conservatively assumed that the SAFWP Room Cooling Unit 1B was inoperable from November 18, 1987 (date that it was last verified operable), to November 30, 1987 (date that it was returned to operable status), a total of 12 days. Due to the operable - operability TS the 1D SAFWP was also deemed inoperable for the 12 days thus not satisfying TS 3.4.2(d).

The SAFWP Room Cooling Unit 1B AC Electrical Breaker was turned ON, immediately after the discovery that it was OFF, on November 30, 1987.

B. INOPERABLE STRUCTURES, COMPONENTS, OR SYSTEMS THAT CONTRIBUTED TO THE EVENT:

The SAFWP Room Cooling Unit 1B was inoperable due to its AC Electrical Breaker being in the OFF position.

C. DATES AND APPROXIMATE TIMES FOR MAJOR OCCURRENCES:

- o November 18, 1987, 2004 EST: assumed event date and time
- o November 30, 1987, 1330 EST: discovery date and time
- o November 30, 1987, 1330 EST: SAFWP Room Cooling Unit 1B AC Electrical Breaker returned to the ON position

D. OTHER SYSTEMS OR SECONDARY FUNCTIONS AFFECTED:

- o The Control Power for the SAFWP Room Cooling Unit was also rendered inoperable due to the AC electrical Breaker being in the OFF position. This affected both the remote control of the breaker and the remote indication (i.e. Red and Green indicating lights).

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E. METHOD OF DISCOVERY:

The SAFWP Room Cooling Unit 1B AC Electrical Breaker was discovered in the OFF position by an electrician during trouble shooting of the above units green light OFF indication problem.

F. OPERATOR ACTION:

The electrician that found the breaker in the OFF position notified the Shift Supervisor of this fact. The Shift Supervisor instructed the electrician to turn the breaker on and check the SAFWP Room Cooling Unit for operability. With the breaker on, the Cooling Unit tested satisfactorily.

G. SAFETY SYSTEM RESPONSES:

None.

III. CAUSE OF EVENT

A. IMMEDIATE CAUSE:

SAFWP Room 1B Cooling Unit AC Breaker in OFF position rendering the 1D SAFWP inadvertently inoperable per TS for more than 7 days.

B. INTERMEDIATE CAUSE:

Indeterminate because no reason or action was found for the SAFWP Room 1B Cooling Unit AC Breaker to be in the OFF position.

C. ROOT CAUSE:

The root cause of the event was determined to be the level of awareness concerning operator and maintenance actions as they relate to the relatively new operability Technical Specifications.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

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TEXT (If more space is required, use additional NRC Form 364A's) (17)

IV. ANALYSIS OF EVENT

This event is reportable in accordance with 10 CFR 50.73, Licensee Event Report System, item (a)(2)(i)(B) which requires reporting of, "any operation prohibited by the Plant's Technical Specifications" in that the 1D SAFWP was deemed inoperable per TS operable - operability definition for longer than the allowed time limit.

An assessment was performed considering both the safety consequences and implications of this event with the following results and conclusions:

There were no operational or safety consequences or implications attributed to the SAFWP Room 1B Cooling Unit inoperability because:

- o The 1D SAFWP, itself, was fully operable and capable of performing its intended function throughout the event.
- o The 1C SAFWP was fully operable and capable of performing its intended function throughout the event even though it was deemed inoperable twice for approximately 1 hour and 50 minutes per A-52.4 (Control of Limiting Conditions for Operating Equipment) for calibration of flow and pressure instrumentation. During these times the operational personnel were not aware of the inoperability of the SAFWP Room 1B Cooling Unit.
- o The SAFWP Room 1A Cooling Unit was operable and capable of performing its intended function.
- o All three main Auxiliary Feedwater Pumps were fully operable and capable of performing their intended function.
- o A security evaluation of the event revealed no obvious malicious intent to sabotage TS related equipment.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

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TEXT (If more space is required, use additional NRC Form 368A's) (17)

V. CORRECTIVE ACTION

A. ACTION TAKEN TO RETURN AFFECTED SYSTEMS TO PRE-EVENT NORMAL STATUS:

- o The SAFWP Room Cooling Unit 1B AC Electrical Breaker was turned ON and the Cooling Unit satisfactorily tested.

B. ACTION TAKEN OR PLANNED TO PREVENT RECURRENCE:

- o With the breakers in the ON position, locks were placed on the AC electrical breakers for the SAFWP Room Cooling Units "A" and "B".
- o Procedure changes were submitted, for all procedures involved, to reflect the new locked breaker status of the above AC electrical breakers.
- o Operations Department will review the generic implications to other safety related equipment.
- o Nuclear Assurance Department will review current methodology for assuring that the operability of the redundant component of a system, sub-system, train, component or device is effective.
- o Operations will develop guidelines for and request training, focusing on the relatively new operability Technical Specifications.
- o Training will design and implement new training modules based on the above request.

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VI. ADDITIONAL INFORMATION

A. FAILED COMPONENTS:

None.

B. PREVIOUS LERs ON SIMILAR EVENTS:

A similar LER event historical search was conducted with the following results: No documentation of similar LER events could be identified.

C. SPECIAL COMMENTS:

- o The Standby Auxiliary Feedwater System was installed to provide an independent system capability following a high energy line break event which could render the three main Auxiliary Feedwater Pumps inoperable. The Standby Auxiliary Feedwater System is manually started and controlled from the Control Room in the event that all other means of feedwater are lost. The system pumps lake water from the service water system to the Steam Generators and would only be used after all other feedwater addition alternatives had been exhausted.
- o The corrective action planned will be tracked by Corrective Action Report (CAR) # 1836.

R

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December 30, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: LER 87-006, Inadvertent Attendant Cooling Unit
Inoperability Due To Open Breaker Causes 1D Standby
Auxiliary Feedwater Pump To Be Deemed Inoperable
Beyond The Technical Specification Limit.
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report
System, item (a)(2)(i)(B) which requires a report of, "any
operation prohibited by the Plant's Technical Specification", the
attached Licensee Event Report LER 87-006 is hereby submitted.

This event has in no way affected the public's health and safety.

Very truly yours,

Roger W. Kober
Roger W. Kober

xc: U.S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA 19406

Ginna USNRC Resident Inspector

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CERTIFIED NO. - P566 876 966