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 FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
 AUTH. NAME AUTHOR AFFILIATION
 KOBER, R. W. Rochester Gas & Electric Corp.
 RECIP. NAME RECIPIENT AFFILIATION
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 STAHL, C. Project Directorate I-1

SUBJECT: Expresses appreciation for time NRC spent w/members of util
 staff on 871210. Believes discussions were helpful in
 reaching agreement on steam generator snubber replacement
 program scheduled to begin in Feb 1988 outage.

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December 15, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Attn: Mr. Carl Stahle
PWR Project Directorate No. 1
Washington, D.C. 20555

Subject: Management Meeting
Steam Generator Snubber Replacement
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Stahle:

Rochester Gas and Electric appreciates the time that members of the NRC management spent with members of the RG&E Staff on December 10, 1987. Thank you for arranging the meetings amid the many other activities requiring your attention. RG&E believes that these informal discussions are a valuable means for each of us to develop an understanding of our priorities and objectives. Of course, the RG&E Staff is available to you at any time.

The discussions held with Messrs. Boger, Kuo, Hartzman and you were particularly helpful in reaching agreement on the steam generator snubber replacement program scheduled to begin in the February 1988 outage. Although our submittal (the latest revision was submitted November 20, 1987) only addresses at-power conditions as the limiting conditions for the support design, we are prepared to discuss less limiting conditions at cold shutdown with you in another meeting. Although small gaps of less than one half inch may be present in the steam generator upper support system transverse to the remaining snubbers, reactor coolant system integrity is maintained during and following a seismic event at these conditions.

A second area of understanding was developed concerning the location of breaks to be assumed in the analyses supporting the snubber replacement. RG&E is applying section B.1.c.(2)(b)(ii) of MEB 3-1, issued as part of Standard Review Plan 3.6.2 with Generic Letter 87-11, which revised the criteria for arbitrary intermediate break locations. The Ginna main steam and feedwater lines have been analyzed and the stresses determined to be such that breaks need be postulated only at the terminal ends.

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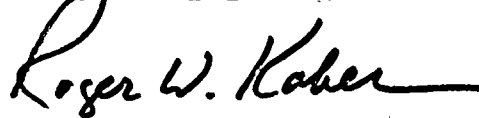
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Finally, the terminal end breaks have been a part of the licensing basis and the design basis for the Ginna model 44 steam generators. The revised criteria for pipe break locations has changed the loads controlling the design of the steam generator supports, however, the break locations controlling the loads on the internals are unchanged. Thus, for the purposes of the proposed support modification, the internals need not be reanalyzed. If the adequacy of the design of steam generator internals is a concern to the Staff, it should be addressed as a separate generic issue unrelated to the snubber replacement modification. RG&E will work to resolve questions concerning steam generator internals if that becomes a generic concern.

Again, we appreciate your efforts to meet with us and discuss high priority issues.

Very truly yours,

A handwritten signature in black ink, reading "Roger W. Kober". The signature is written in a cursive style with a long horizontal flourish at the end.

Roger W. Kober

DEC 10 1987

Distribution:
Docket 50-244 ←
PD I-3 Reading
M. Rushbrook
~~XXXXXXXXXX~~
R. Stahle

DOCKET NO(S). 50-244
Mr. Roger W. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

SUBJECT: R. E. Ginna Nuclear Power Plant

The following documents concerning our review of the subject facility are transmitted for your information.

- ☐ Notice of Receipt of Application, dated _____.
- ☐ Draft/Final Environmental Statement, dated _____.
- ☐ Notice of Availability of Draft/Final Environmental Statement, dated _____.
- ☐ Safety Evaluation Report, or Supplement No. _____ dated _____.
- ☐ Environmental Assessment and Finding of No Significant Impact, dated _____.
- ☐ Notice of Consideration of Issuance of Facility Operating License or Amendment to Facility Operating License, dated _____.
- ☒ Bi-Weekly Notice; Applications and Amendments to Operating Licenses Involving No Significant Hazards Considerations, dated 12/2/87 [see page(s)] 45889.
- ☐ Exemption, dated _____.
- ☐ Construction Permit No. CPPR-_____, Amendment No. _____ dated _____.
- ☐ Facility Operating License No. _____, Amendment No. _____ dated _____.
- ☐ Order Extending Construction Completion Date, dated _____.
- ☐ Monthly Operating Report for _____ transmitted by letter dated _____.
- ☐ Annual/Semi-Annual Report- _____
_____ transmitted by letter dated _____.

Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: with enclosure
See attached Sheet

OFFICE	NRR/PD I-3					
SURNAME	MRushbrook					
DATE	12/10/87					

Mr. Roger W. Kober
Rochester Gas and Electric Corporation

R. E. Ginna Nuclear Power Plant

cc:

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