

(CAR 1222)

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y R E G 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CON'T

0 1 L 6 0 5 0 0 0 2 4 4 7 0 4 2 0 7 9 8 0 4 3 0 7 9 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Following routine sample of boric acid storage tanks, analysis showed less than 12%

0 3 boric acid (BA) in both tanks. (T.S.3.2.3.c) This event is similar to that reported in

0 4 LER 79-009. Reactor power reduction was started. After enriching tank contents and

0 5 analysis showed above 12%, power reduction was stopped.

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0 9

SYSTEM CAUSE CAUSE COMPONENT COMP. VALVE  
CODE CODE SUBCODE CODE SUBCODE SUBCODE  
P C 11 E 12 X 13 V A L V E X 14 C 15 A 16  
9 10 11 12 13 14 15 16 17 18 19 20(17) LER/RO  
REPORT  
NUMBEREVENT YEAR  
7 9  
21 22SEQUENTIAL  
REPORT NO.  
0 1 0  
23 24 25 26OCCURRENCE  
CODE  
0 1  
27 28 29REPORT  
TYPE  
T  
30 31REVISION  
NO.  
0  
32 33ACTION FUTURE  
TAKEN ACTION  
X 18 A 19  
33 34EFFECT  
ON PLANT  
B 20  
35 36SHUTDOWN  
METHOD  
Z 21  
37 38HOURS  
0 0 0 3  
39 40 41 42ATTACHMENT  
SUBMITTED  
Y 23  
43 44NPRD-4  
FORM SUB.  
Y 24  
45 46PRIME COMP.  
SUPPLIER  
A 25  
47 48COMPONENT  
MANUFACTURER  
R 3 4 0 26  
49 50 51 52

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Possible back flow of reactor makeup water (RMW) through blender BA supply check

1 1 valve. Manual valve in blender BA supply line was closed and sample frequency

1 2 increased to every 12 hr. Check valve was replaced with a qualified spare. Valve is

1 3 Rockwell Manufacturing Co. 1 in. 1,500 lb. stainless steel spring check valve.

1 4

1 5

FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)  
E 28 0 9 2 29 NA B 31 chemical analysis  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
Z 33 Z 34 NA NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)  
0 0 0 37 Z 38 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100PERSONNEL INJURIES NUMBER DESCRIPTION (41)  
0 0 0 40 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)  
Z 42 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100PUBLCITY ISSUED DESCRIPTION (45)  
N 44 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

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NRC USE ONLY

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Attachment to LER 79-010/01T-0  
Rochester Gas and Electric Corporation  
R. E. Ginna Nuclear Power Plant, Unit No. 1  
Docket No. 50-244

On April 20, 1979 at approximately 1420 hours the Shift Foreman was notified of the chemical analysis which indicated that both the "A" and the "B" boric acid storage tanks contained 11.8 wt.-% boric acid solution. An orderly load reduction was commenced, and batching operations were initiated to restore the tanks to the required specifications. The tanks were within specifications at 1710 hours that day.

To check for the possibility of back flow of reactor makeup water from the reactor makeup water storage tank through the blender and the blender boric acid supply check valve, a manual valve in the boric acid supply line was closed. With this valve closed the tank concentrations have remained constant. The tank sampling frequency has been increased to once every twelve hours. The check valve was replaced with a qualified spare.

