

Attachment A

Revise the Technical Specifications as follows:

1. Replace Sections 3.13, 4.2, and 4.14, and page 6.10-2 with the enclosed proposed sections.

8111270158 811124
PDR ADDCK 05000244
P PDR

3.13

Snubbers

Applicability

Applies to the operability of all safety-related snubbers listed in the Inservice Inspection Program.

Objective

To specify the requirements for operability of snubbers.

Specification

3.13.1 Whenever the reactor is above cold shutdown, all snubbers listed in the Inservice Inspection Program shall be operable. In addition, snubbers which support systems which are required to be operable at cold or refueling shutdown shall be operable.

3.13.2 If one or more snubbers are inoperable when required to be operable, then restore the inoperable snubber(s) to operable status within 72 hours or declare the supported system inoperable and take the appropriate actions for that system.

Basis

Snubbers are required to be operable to ensure that the structural integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads.

4.2

Inservice Inspection

Applicability

Applies to the inservice inspection of Quality Groups A, B and C Components, High Energy Piping Outside of Containment, Steam Generator Tubes, and Snubbers.

Objectives

To provide assurance of the continuing structural and operational integrity of the structures, components and systems in accordance with the requirements of 10 CFR 50.55a(g).

4.2.1

Specification

The inservice inspection program for Quality Groups A, B and C Components, High Energy Piping Outside of Containment, Steam Generator tubes and snubbers shall be in accordance with Appendix B of the Ginna Station Quality Assurance Manual. This inservice inspection program shall define the specific requirements of the edition and Addenda of the ASME Boiler and Pressure Vessel Code, Section XI, which are applicable for the forty month period of the ten year inspection interval. The program ten year inspection intervals shall be based on the following commencing dates.

4.2.1.1 The inspection interval for Quality Group A Components shall be ten year intervals of service commencing on January 1, 1970.

4.2.1.2 The inspection intervals for Quality Group B and C Components shall be ten year intervals of service with the first interval commencing with May 1, 1973, and with subsequent intervals commencing January 1, 1980.

- 4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing with May 1, 1973. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.
- 4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.
- 4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
- 4.2.1.6 The inspection intervals for snubbers shall be as specified in the Inservice Inspection Program.

Basis

The inservice inspection program provides assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The program complies with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures, components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirements of General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.

4.14

Deleted.

- c. Records of plant radiation and contamination surveys.
- d. Records of off-site environmental monitoring surveys.
- e. Record of radiation exposure of all plant personnel, including all contractors and visitors to the plant who enter radiation control areas.
- f. Records of radioactivity in liquid and gaseous material released to the environmental and radioactive waste shipments.
- g. Records of transient or operational cycles for those facility components designed for limited number of transients or cycles.
- h. Records of training and qualification for current station technical and operations staff members.
- i. Records of in-service inspections performed pursuant to these Technical Specifications.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR Section 50.59.
- k. Records of meetings of the PORC and the NSARB.
- l. Records of Quality Assurance activities as required by the QA Manual.
- m. Records for Environmental Qualification which are covered under the provisions of paragraph 6.14.
- n. Records of the service lives of all hydraulic and mechanical snubbers listed in the Inservice Inspection Program including the date at which the service life commences and associated installation and maintenance records.