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 RECIP. NAME RECIPIENT AFFILIATION
 CRUTCHFIELD, D. Operating Reactors Branch 5

SUBJECT: Forwards "Containment Vessel Evaluation of Dome Liner & Studs," re SEP Topic III-7.B, "Load Combinations (Containment Liner Analysis)."

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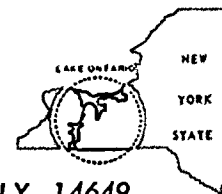
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April 28, 1983

Director of Nuclear Reactor Regulation
Attention: Mr. Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: SEP Topic III-7.B, Load Combinations (Containment Liner
Analysis)
R. E. Ginna Nuclear Power Plant
Docket 50-244

Dear Mr. Crutchfield:

Enclosed is a report "Containment Vessel Evaluation of Dome Liner and Studs" prepared by Gilbert/Commonwealth for Rochester Gas and Electric. The purpose of the report is to describe the analysis performed to evaluate the post-accident behavior of the containment liner and studs. The conclusion reached in the report is that failure of some studs in certain areas of the liner, although not expected, is possible. However, it is determined that this condition would not cause tearing of the liner, and thus would not prevent the liner from maintaining its post-accident leak-tight integrity.

As a point of clarification, it should be noted that the accident pressure and temperature profiles used in NUREG/CR-2580 (the SMA/LLL liner analysis) are for the steam line break condition, rather than a LOCA, as implied. This is obvious from the LLL analysis presented in Appendix A to the Safety Evaluation Report for SEP Topics VI-2.D and VI-3, Mass and Energy Release for Pipe Breaks Inside Containment", November 3, 1981. Nonetheless, as shown by the attached analysis, the liner integrity will be maintained for all accident situations.

Very truly yours,

John E. Maier
John E. Maier

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