



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Lankford

REGULATORY DOCKET FILE COPY

December 13, 1977

BBS LTR #1154-77



James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #77-051/03L-0 Docket #050-249 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(a), engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

B.B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:dlz

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

DEC 15 1977

773460090

SENSEE EVENT REPORT

CONTROL BLOCK:

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
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 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T
REPORT SOURCE:

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 During normal monthly surveillance DIS 500-2, setpoint for reactor level switch LIS
03 3-263-58B found at 116.8" H2O dp. T.S. Table 3.2.1 setpoint limit is 114.3" H2O dp.
04 Safety significance minimal since remaining three level switches would have isolated
05 reactor at conservative level setting. Similar occurrence reported in Docket
06 #050-237, 50-237/1976-57.
07
08

SYSTEM CODE:

1	2	3	4	5	6	7	8	9	10
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 CAUSE CODE:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 CAUSE SUBCODE:

1	2	3	4	5	6	7	8	9	10
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 COMPONENT CODE:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 COMP. SUBCODE:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 VALVE SUBCODE:

1	2	3	4	5	6	7	8	9	10
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LER/RO REPORT NUMBER:

1	2	3	4	5	6	7	8	9	10
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 EVENT YEAR:

1	2	3	4	5	6	7	8	9	10
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 SEQUENTIAL REPORT NO.:

1	2	3	4	5	6	7	8	9	10
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 OCCURRENCE CODE:

1	2	3	4	5	6	7	8	9	10
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 REPORT TYPE:

1	2	3	4	5	6	7	8	9	10
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 REVISION NO.:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

ACTION TAKEN:

1	2	3	4	5	6	7	8	9	10
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 FUTURE ACTION:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 EFFECT ON PLANT:

1	2	3	4	5	6	7	8	9	10
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 SHUTDOWN METHOD:

1	2	3	4	5	6	7	8	9	10
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 HOURS:

1	2	3	4	5	6	7	8	9	10
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 ATTACHMENT SUBMITTED:

1	2	3	4	5	6	7	8	9	10
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 NRPD-4 FORM SUB.:

1	2	3	4	5	6	7	8	9	10
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 PRIME COMP. SUPPLIER:

1	2	3	4	5	6	7	8	9	10
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 COMPONENT MANUFACTURER:

1	2	3	4	5	6	7	8	9	10
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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 Cause was setpoint drift. Switch reset to specifications and was repeatable within
11 Dresden setpoint band of 112+/-1 inches dp. All four reactor low low level isolation
12 switches will continue to be tested monthly via DIS 500-2.
13
14

FACILITY STATUS:

1	2	3	4	5	6	7	8	9	10
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 % POWER:

1	2	3	4	5	6	7	8	9	10
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 OTHER STATUS:

1	2	3	4	5	6	7	8	9	10
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 METHOD OF DISCOVERY:

1	2	3	4	5	6	7	8	9	10
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 DISCOVERY DESCRIPTION:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

ACTIVITY RELEASED:

1	2	3	4	5	6	7	8	9	10
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 CONTENT OF RELEASE:

1	2	3	4	5	6	7	8	9	10
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 AMOUNT OF ACTIVITY:

1	2	3	4	5	6	7	8	9	10
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 LOCATION OF RELEASE:

1	2	3	4	5	6	7	8	9	10
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PERSONNEL EXPOSURES:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 NUMBER:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 TYPE:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 DESCRIPTION:

1	2	3	4	5	6	7	8	9	10
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PERSONNEL INJURIES:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 NUMBER:

1	2	3	4	5	6	7	8	9	10
---	---	---	---	---	---	---	---	---	----

 DESCRIPTION:

1	2	3	4	5	6	7	8	9	10
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LOSS OF OR DAMAGE TO FACILITY:

1	2	3	4	5	6	7	8	9	10
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 TYPE:

1	2	3	4	5	6	7	8	9	10
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 DESCRIPTION:

1	2	3	4	5	6	7	8	9	10
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PUBLICITY:

1	2	3	4	5	6	7	8	9	10
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 ISSUED:

1	2	3	4	5	6	7	8	9	10
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 DESCRIPTION:

1	2	3	4	5	6	7	8	9	10
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Carl Lindberg

NRC USE ONLY

ATTACHMENT TO LICENSEE EVENT REPORT 77-051/03L-0
COMMONWEALTH EDISON COMPANY (CWE)
DRESDEN UNIT 3 (ILDRS-3)
DOCKET # 050-249

During routine surveillance of reactor level switch LIS 3-263-58B a setpoint of 116.8 inches of water dp was found. This is above the Tech Spec limit of 114.3 inches of water dp stated in Table 3.2.1. The switch was exercised from normal operating to the trip and repeated at 113.2 inches dp. The switch was promptly reset to 112.4 inches dp which is within the Dresden established limit of 112+1 inches of water dp.

The function of LIS 3-263-58B is to initiate a reactor isolation. The switch is redundant with LIS-3-263-57B for isolation Channel "B". Two other level switches in isolation Channel "A" are similarly arranged so as to provide a one-out-of-two-taken-twice isolation logic.

This switch is physically mounted on the same movement as the scram switch LIS-3-263-58B, which was found within the Dresden setpoint band. Analysis of the setpoint history graph for LIS-3-263-58B (isolation) indicates acceptable switch performance in that this was the first Reportable Occurrence since initial installation. A similar occurrence on Unit 2 LIS-2-263-57B (isolation) was reported under R.O. #50-237/1976-57. No further corrective action is deemed necessary.

The switch is calibrated monthly using DIS 500-2. Analysis of the remaining three level switches indicate that a reactor isolation would have occurred at 111.2 inches of water dp which is 3.1 inches of water dp conservative to the Tech Spec Limit.