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ACCESSION NBR: 8605280224 DOC. DATE: 86/05/23 NOTARIZED: NO DOCKET #
FACIL: 50-244 Robert Emmet Ginna Nuclear Plant, Unit 1, Rochester G 05000244
AUTH. NAME AUTHOR AFFILIATION
KOBER, R. W. Rochester Gas & Electric Corp.
RECIP. NAME RECIPIENT AFFILIATION
LEAR, G. E. PWR Project Directorate 1

SUBJECT: Responds to 860409 request for addl info re isolation devices, in order for NRC to complete review of SPDS. New field multiplexers seismically qualified & utilize input cards providing electrical isolation.

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ROGER W. KOBER
VICE PRESIDENT
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TELEPHONE
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May 23, 1986

Director of Nuclear Reactor Regulation
Attention: Mr. George E. Lear, Chief
PWR Project Directorate No. 1
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Safety Parameter Display System
R. E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Lear:

An NRC letter dated April 9, 1986 requested additional information concerning isolation devices in order for the Staff to complete their review of the Safety Parameter Display System (SPDS). The Ginna Station SPDS system utilizes field input cables that formerly terminated in the original plant process computer. The input cables now terminate at the field multiplexers of the new system, which are now in the same physical location as the old computer. The isolation devices provided in the Reactor Protection System (RPS) and Engineered Safety Features Actuation System circuits were not modified during installation of the SPDS.

The existing isolation devices were reviewed by the NRC Staff during the Systematic Evaluation Program (SEP) assessment of SEP Topic VII-1.A, Isolation of Reactor Protection System from Nonsafety Systems, Including Qualification of Isolation Devices. The stated objective of the Staff review was to verify that the Ginna reactor has a reactor protection system design which provides effective and qualified isolation of nonsafety systems from safety systems to assure that safety systems will function as required. The conclusion of the review, stated in a letter dated July 30, 1981 from Dennis M. Crutchfield to John E. Maier, was that "the Staff's position is that suitably qualified isolators have been provided at Ginna."

In order to assure that the RPS isolation devices are not subject to stresses more severe than anticipated in the original design, the new field multiplexers are seismically qualified and utilize input cards which provide electrical isolation sufficient to prevent any credible voltage excursion from propagating to the RPS inputs from other inputs via the multiplexer itself. All multiplexer outputs are fiber optic and, therefore, will not propagate spurious voltages from external sources.

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1/0

DATE May 23, 1986
TO Mr. George E. Lear

The only other conducting cables terminating at the multiplexers are the 120 VAC power supplies from the Technical Support Center Uninterruptable Power Supply (TSC UPS). These cables are routed with 120 VAC and lower voltage cables. Breakers and fuses are also provided to protect the multiplexers in the event of electrical faults. Therefore, this modification has not degraded the isolation provisions reviewed and approved by the NRC Staff under the SEP program and the Staff conclusion remains valid.

Very truly yours,

Roger W. Kober

Roger W. Kober

Distribution:
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PAD#1 R/F w/o Encl.
P. Shuttleworth w/Encl.
PM w/Encl.

May 16, 1986

DOCKET NO(S). 50-244

Mr. Roger K. Kober, Vice President
Electric and Steam Production
Rochester Gas & Electric Corporation
89 East Avenue
Rochester, New York 14649

SUBJECT: R. E. GINNA NUCLEAR POWER PLANT

The following documents concerning our review of the subject facility are transmitted for your information.

☐ Notice of Receipt of Application, dated _____.

☐ Draft/Final Environmental Statment, dated _____.

☐ Notice of Availability of Draft/Final Environmental Statement, dated _____.

☐ Safety Evaluation Report, or Supplement No. _____, dated _____.

☐ Notice of Hearing on Application for Construction Permit, dated _____.

☐ Notice of Consideration of Issuance of Facility Operating License, dated _____.

☐ Monthly Notice; Applications and Amendments to Operating Licenses Involving no Significant Hazards Considerations, dated _____.

☐ Application and Safety Analysis Report, Volume _____.

☐ Amendment No. _____ to Application/SAR dated _____.

☐ Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.

☐ Facility Operating License No. _____, Amendment No. _____, dated _____.

☐ Order Extending Construction Completion Date, dated _____.

☒ Other *(Specify)* Biweekly Notice covering period April 9, 1986. Expiration date for hearing requests and comments May 9, 1986.

Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc: See next page

OFFICE	PAD#1:LA						
SURNAME	P. Shuttleworth						
DATE	5/16/86						

NRC FORM 318 (1/84) NRCM 0240

cc:

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☐ Facility Operating License No. _____, Amendment No. _____, dated _____.

☐ Order Extending Construction Completion Date, dated _____.

☒ Other *(Specify)* Biweekly Notice covering period March 12, 1986. Expiration date for hearing requests and comments April 11, 1986.

Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

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☐ Construction Permit No. CPPR- _____, Amendment No. _____ dated _____.

☐ Facility Operating License No. _____, Amendment No. _____, dated _____.

☐ Order Extending Construction Completion Date, dated _____.

☒ Other *(Specify)* Biweekly Notice covering period February 26, 1986. Expiration date for hearing requests and comments March 28, 1986.

Division of PWR Licensing-A
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