4.2.1.1 The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.

4.2.1.2 The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing with May 1, 1973, January 1, 1980, 1990 and 2000, respectively.

4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing May 1, 1973, January 1, 1980, 1990 and 2000, respectively. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.

4.2.1.4.a Steam generator tubes that have imperfections greater than 40% through wall, as indicated by eddy current, shall be repaired by plugging or sleeving.

4.2.1.4.b Steam generator sleeves that have imperfections greater than 30% through wall, as indicated by eddy current, shall be repaired by plugging.

The repair criteria of 4.2.1.4.a and 4.2.1.4.b are based on the requirements of USNRC Regulatory Guide 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes" as implemented by RG&E (Reference 1). This guide describes a method acceptable to the NRC staff for establishing the limiting safe conditions of tube degradation of steam generator tubing. The repair criteria is based on structural allowances, an allowance for eddy current measurement error and an allowance for degradation during the operating period. These allowances are added together to determine the repair criteria which is typically 40% for steam generator tubes. Based on calculations the appropriate sleeve plugging limit is a 42% thru wall defect. In order to allow for conservatism, a 30% plugging limit for sleeves will be utilized.

Reference 1: "Steam Generator Rapid Sleeving Program Design Verification Report", R.E. Ginna Nuclear Power Plant, August 1982.