



Westinghouse
Electric Corporation

Water Reactor
Divisions

Nuclear Technology Division

Box 355
Pittsburgh, Pennsylvania 15230

July 18, 1980
ST-SST-105-
RGE-80-93

50244

Mr. M. P. Lilley
Rochester Gas & Electric
89 East Avenue
Rochester, New York 14649

Dear Mr. Lilley:

ROBERT EMMETT GINNA NUCLEAR GENERATING STATION
R. E. Ginna Seismic Upgrading Program
Residual Heat Removal System, Section 200

Transmitted herein is the piping stress report for the R. E. Ginna Nuclear Generating Station Unit No. 1, Residual Heat Removal System, Section 200 SDTAR-80-05-03.

Analysis results and support loads contained herein reflect the layout arrangement shown on marked isometric C-381-354, Sheet 1, Revision D. Any significant modifications to the layout of this system of piping and or its restraints will warrant reanalysis of this piping system.

Minor design changes or construction modifications will generally not affect stress results and support recommendations. However, case by case review of the discrepancies can be used to determine overall system behavior and effects.

If any questions exist relative to the above transmittal, please contact W. A. Massie of Westinghouse.

Very truly yours,

F. Noon, Manager
Eastern Region WNI Support

W. A. Massie/dj
Attachment

cc: R. E. Smith - 1L, 1A
R. C. Mecredy, RGE - 1L, 1A
M. P. Lilley, RGE - 1L, 1A
T. R. Weis, RGE - 1L, 1A
B. H. Bell, GAI - 1L, 1A
P. M. Cameron, GAI - 1L, 1A
B. A. Snow, RGE - 1L, 1A

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