OPERATING DATA REPORT

DOCKET NO. 50-244
DATE June 9, 1980
COMPLETED BY Andrew E. McNamara
TELEPHONE 1 (315) 524-4446, Ext. 293

OPERATING STATUS

1. Unit Name: GINNA STATION, UNIT #1
2. Reporting Period: May, 1980
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 490
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

Notes: Refueling and Maintenance Shutdown ended 5/23/80. Reactor Power Level was increased to 100% on 5/27 and remained there until the end of the Period.

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	3,647	92,159
12. Number of Hours Reactor Was Critical	231.5	2,346.25	70,026.02
13. Reactor Reserve Shutdown Hours	0	0	1,621.57 *
14. Hours Generator On-Line	192.75	2,307.25	68,263.88
15. Unit Reserve Shutdown Hours	0	0	8.5 *
16. Gross Thermal Energy Generated (MWH)	238,944	3,357,600	91,633,402
17. Gross Electrical Energy Generated (MWH)	77,530	1,098,556	29,784,527
18. Net Electrical Energy Generated (MWH)	73,152	1,043,963	28,205,511
19. Unit Service Factor	25.91%	63.26%	74.07%
20. Unit Availability Factor	25.90%	63.26%	74.08%
21. Unit Capacity Factor (Using MDC Net)	20.92%	60.9%	67.46%
22. Unit Capacity Factor (Using DER Net)	20.92%	60.9%	67.46%
23. Unit Forced Outage Rate	0%	0%	9.61%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
Scheduled - Steam Generator Inspection and Modifications - 10/24 thru 11/23.

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

* Cumulative total commincing January 1, 1975

49-88 (REV. 1/76)

8006160 038

AVERAGE DAILY UNIT POWER UNIT

DOCKET NO. 50-244
UNIT #1, Ginna Station
DATE June 9, 1980
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MONTH May, 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-	17	-
2	-	18	-
3	-	19	-
4	-	20	-
5	-	21	-
6	-	22	-
7	-	23	-
8	-	24	75
9	-	25	273
10	-	26	318
11	-	27	449
12	-	28	481
13	-	29	483
14	-	30	483
15	-	31	484
16			

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWN AND POWER REDUCTIONS

REPORT MONTH May, 1980

No.	Date	Type [1]	Duration (Hours)	Reason [2]	Method of Shutting Down Reactor [3]	Licensee Event Report #	System Code [4]	Component Code [5]	Cause & Corrective Action to Prevent Recurrence
1	5/23/80*	S	1,339.75	C	1				* Annual Refueling and Maintenance Shutdown which began on 3/29/80, ended on 5/23/80.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

49-89 (REV. 1/78)

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

DOCKET NO. 50-244

UNIT Ginna Station, Unit #1

DATE June 9, 1980

COMPLETED BY Andrew E. McNamara
Andrew E. McNamara

TELEPHONE 1 (315) 524-4446
Ext. 293

MONTH May, 1980

The Annual Refueling and Maintenance Shutdown ended on 5/23/80 @ 2315. The Reactor Power Level was maintained at ~25% until the Turbine Overspeed Trip Test was performed on 5/24. The Reactor Power Level was increased to the 100% level by 5/27 and remained there until the end of the Period.

GINNA STATION

MAINTENANCE REPORT FOR APRIL AND MAY, 1980

During the month of April and May the Unit was in its annual refueling and maintenance shutdown. The required testing, inspections and maintenance were completed, including the designated preventive maintenance program. Some of the major safety related maintenance included:

1. Inspection of the 1B RCP motor and seals.
2. Technical Specification required snubber testing and inspection program.
3. Annual inspection of both emergency diesel generator units.
4. Cleaning and inspection and multiamp testing of 3 of the safeguard busses.
5. Replacement of the leaking mechanical seal on the 1A RHR pump.
6. Replacement of the boric acid flow control valve.
7. Replacement of the 1A charging pump drain valve.
8. Replacement of the Regenerative heat exchanger drain valve.
9. Inspection of the SI pump suction valves.
10. Replacement of the plug and seat on the 1B Steam Generator Blowdown Valve.
11. Inspection of boric acid transfer pumps.
12. Replaced the batch circuit board on the pressurizer pressure controller.
13. Replaced one intermediate range and one source range detector on the nuclear instrumentation system.