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 FACIL: 50-244 Robert Emmet Ginne Nuclear Plant, Unit 1, Rochester G 05000244
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 ZIEMANN, D. L. Office of Nuclear Reactor Regulation
 Operating Reactors Branch 2

SUBJECT: Forwards facility Emergency Procedures E1.1, E1.2 & E1.3, in response to NRC 800221 request. Procedures include instruction for use of safety-related & nonsafety related equipment.

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ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649



LEON D. WHITE, JR.
VICE PRESIDENT

TELEPHONE
AREA CODE 716 546-2700

February 26, 1980

Director of Nuclear Reactor Regulation
Attn: Mr. Dennis L. Ziemann, Chief
Operating Reactors Branch #2
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: SEP Topic III-12, "Environmental Qualification"
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Ziemann:

During the February 21, 1980 SEP Owners Group meeting concerning "Environmental Qualification", it was requested the R.E. Ginna Emergency Procedure dealing with the LOCA and Steam/Feed Line Break accidents be transmitted to NRC for use in the establishment of a list of necessary mitigation equipment. Accordingly, enclosed for your information are eight (8) copies of the following Ginna Emergency Procedures:

- E1.1, Immediate Actions and Diagnostics for Spurious Actuation of SI, LOCA, Loss of Secondary Coolant, and Steam Generator Tube Rupture, Rev. 13
- E1.2, Loss of Reactor Coolant, Rev. 18
- E1.3, Loss of Secondary Coolant, Rev. 7

These emergency procedures have been written in conformance with the Westinghouse Owners Group guidelines, which were thoroughly reviewed and approved by the NRC's Bulletins and Orders Task Force.

It should be emphasized that the content of these emergency procedures includes instructions for the use of all available equipment, under any conceivable circumstances, recognizing that the expected sequence of events following an accident/transient may not tend to follow the sequence assumed in the conservative plant safety analyses. These procedures do not reflect only safety-related equipment, although they do include the safety-related equipment. A definition of the safety-related equipment requires detailed knowledge of the particular plant configuration, licensing commitments, and safety analyses - information which cannot be gleaned from a review of these procedures.

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DATE February 26, 1980

TO Mr. Dennis L. Ziemann, Chief

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Please consider this during the course of your review.

Very truly yours,

L. D. White, Jr.

L. D. White, Jr.

LDW:np