

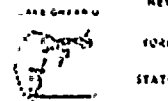


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LEON D. WHITE, JR.  
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CENTRAL FILES



October 24, 1979

Mr. Boyce H. Grier, Director  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region I  
631 Park Avenue  
King of Prussia, PA 19406

Subject: I&E Bulletin 79-17 Final Response  
R.E. Ginna Nuclear Power Plant, Unit 1  
Docket No. 50-244

Dear Mr. Grier:

On August 24, 1979 we forwarded the requested information of I&E Bulletin 79-17 concerning the stagnant borated water systems at R. E. Ginna Nuclear Power Plant. We also stated that stagnant portions of systems that are accessible during normal reactor operation, where surface and volumetric examinations had not been previously performed, would be examined within sixty (60) days in accordance with the requirements of Item 2 of the Bulletin.

These required examinations were completed on October 10, 1979 and included both liquid penetrant and ultrasonic examinations of twenty-four (24) welds. The twenty-four (24) welds consisted of a sampling of over ten (10) percent of the remaining stagnant borated water systems that had not been previously examined. The following is a listing, by system and size, of the welds examined:

CONTAINMENT SPRAY (6" discharge in Aux. Bldg.)

6 - SI - 2020 - 5  
6 - SI - 2020 - 13  
6 - SI - 2020 - 15  
6 - SI - 2020 - 20  
6 - SI - 2020 - 22  
  
6 - SI - 2021 - 4  
6 - SI - 2021 - 9  
6 - SI - 2021 - 12  
6 - SI - 2021 - 16  
6 - SI - 2021 - 23

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DATE October 24, 1979

TO Mr. Boyce H. Grier, Director

## SAFETY INJECTION (3" &amp; 4" discharge in Aux. Bldg.)

3 - SI - 2022 - 8  
3 - SI - 2022 - 13  
3 - SI - 2022 - 20  
4 - SI - 2022 - 24

3 - SI - 2023 - 8  
3 - SI - 2023 - 15  
3 - SI - 2023 - 20  
4 - SI - 2023 - 24

3 - SI - 2024 - 3  
3 - SI - 2024 - 5  
3 - SI - 2024 - 22  
3 - SI - 2024 - 32

## CONTAINMENT SPRAY (10" &amp; 6" suction in Aux. Bldg.)

10 - SI - 2019 - 2  
6 - SI - 2019 - 14

There were not any indications, either surface or volumetric, that the subject welds and adjacent base material had suffered any stress corrosion cracking attack. Therefore, we are considering this correspondence as the final response to I&E Bulletin 79-17.

If you have any further questions, do not hesitate in contacting us.

Very truly yours,



L. D. White, Jr.

AEC:dmaK8

xc: Director  
Division of Operating Reactors  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Washington, DC 20555

