

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 7905220377 DOC. DATE: 79/05/16 NOTARIZED: NO DOCKET #
 FACIL: 50-244 ROBERT EMMET GINNA NUCLEAR PLANT, UNIT 1, ROCHESTER G. 05000244
 AUTH. NAME AUTHOR AFFILIATION
 WHITE, L. D. ROCHESTER GAS & ELECTRIC CORP.
 RECIP. NAME RECIPIENT AFFILIATION
 ZIEMUNN, D. L. OPERATING REACTORS BRANCH 2

SUBJECT: COMMENTS ON 790416 LTR DELETING TOPICS FOR FURTHER REVIEW
 FROM SYSTEMATIC EVALUATION PROGRAM.

DISTRIBUTION CODE: A001S COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 3
 TITLE: GENERAL DISTRIBUTION FOR AFTER ISSUANCE OF OPERATING LIC

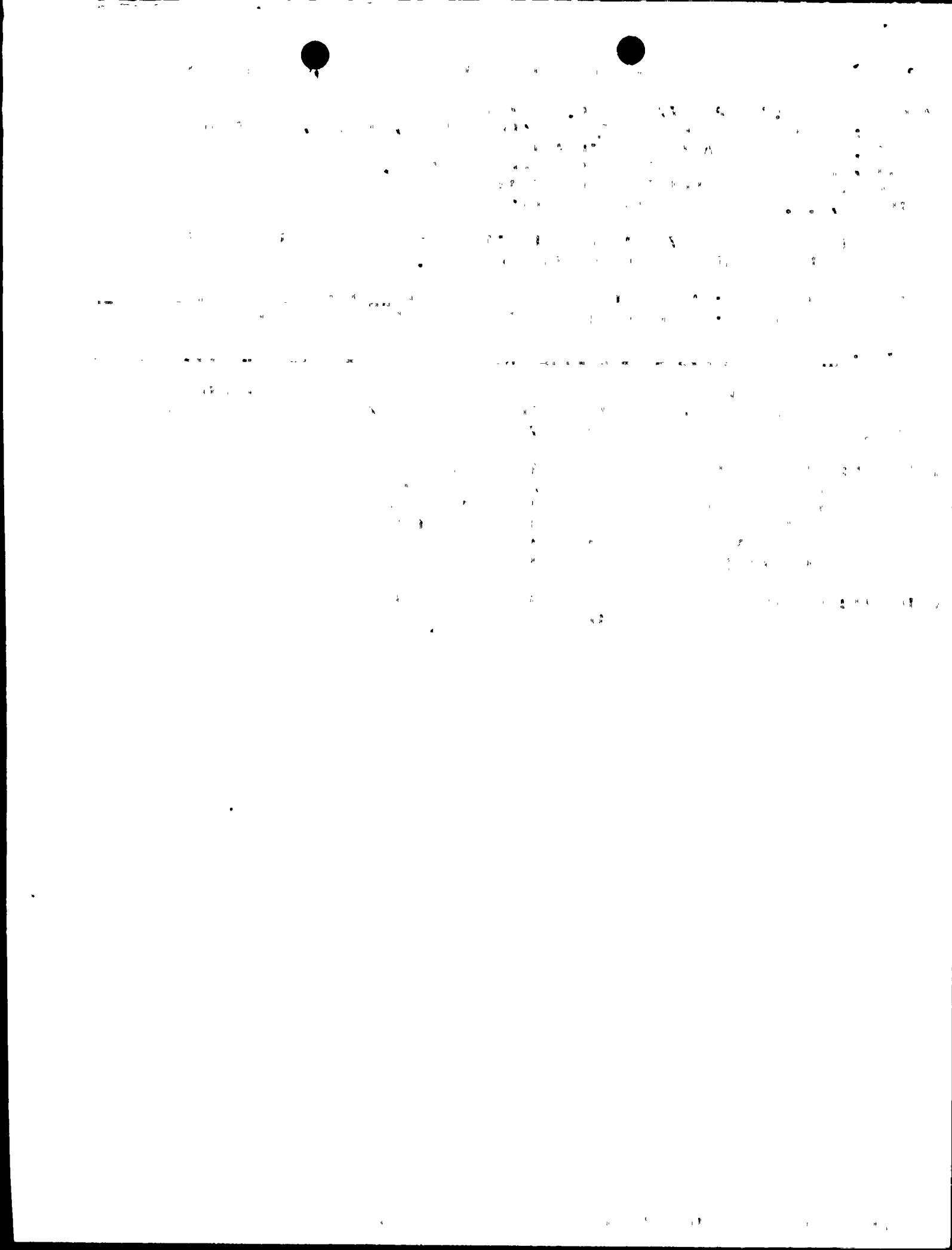
NOTES: LCY: D. ALBISON, C. HOFMAYER

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
ACTION:	05 BC ORB #2	7 7		
INTERNAL:	01 REG FILE	1 1	02 NRC PDR	1 1
	12 I&E	2 2	14 TA/EDO	1 1
	15 CORE PERF BR	1 1	16 AD SYS/PROJ	1 1
	17 ENGR BR	1 1	18 REAC SFTY BR	1 1
	19 PLANT SYS BR	1 1	20 EEB	1 1
	21 EFLT TRT SYS	1 1	22 BRINKMAN	1 1
EXTERNAL:	03 LPDR	1 1	04 NSIC	1 1
	23 ACRS	16 16		

TITAN
 cep

MAY 23 1979

TOTAL NUMBER OF COPIES REQUIRED: LTTR 38 ENCL 0
38





ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR.
VICE PRESIDENT

TELEPHONE
AREA CODE 716 546-2700



May 16, 1979

Director of Nuclear Reactor Regulation
Attention: Mr. Dennis L. Ziemann, Chief
Operating Reactors Branch No. 2
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: SEP Topics
R.E. Ginna Nuclear Power Plant
Docket No. 50-244

Dear Mr. Ziemann:

We have reviewed your letter of April 16, 1979 which deleted a number of topics from further review in the Systematic Evaluation Program, and have several comments for your consideration.

1) A number of generic topics were deleted inasmuch as they are subject to ongoing NRC review:

Topic V-3, Overpressure Protection, was one of these topics. However, this topic should be listed as complete based on the NRC letter dated April 18, 1979. The April 18 letter approved plant modifications and issued Technical Specifications for reactor vessel overpressure protection.

Topic V-7, Reactor Coolant Pump Overspeed, has also been deleted from SEP review. However, since Topic III-10.B, Reactor Coolant Pump Flywheel, is currently under review, and is closely related to Topic V-7, it is not clear how only one of these two topics can be deleted.

Topic V-13, Water Hammer, should be shown as being complete. This is based on the installation of "J-tubes" on the Ginna steam generators and on administrative actions detailed in our letters of January 3 and June 15, 1978, "Steam Generator Water Hammer Prevention".

Topic XIII-2, Safeguards/Industrial Security, should be shown as complete. The Staff Security Safety Evaluation Report was issued as Amendment No. 25 to the Ginna license on February 22, 1979.

7905220377

Aool
S 1/0

DATE May 16, 1979

TO Director of Nuclear Reactor Regulation
Attention: Mr. Dennis L. Ziemann

2

2) Several SEP Topics were determined to be not applicable to Ginna. Your letter indicates that Group I (BWR) DBE's are not applicable. It is our understanding that this group includes part of one topic, XV-3: Inadvertent Closure of MSIV's. It appears that a number of other topics should also be deleted because they are not applicable or because they have been resolved. For example, topic XV-11, Fuel Assembly Misloading, is deleted because it is reviewed on a routine basis as part of reload evaluations. Many other design bases events are also reviewed as part of reload evaluations. The following DBE's were reviewed in Exxon. Topical Report XN-NF-77-40, dated November, 1977:

- XV-2, Spectrum of Steam Line Breaks
- XV-3, Loss of External Load
- XV-7, RC Pump Motor Seizure
- XV-8, Control Rod Misoperation

These analyzed DBE's generally envelope other transients whose consequences are known to be less severe. These other transients are:

- XV-4, Loss of AC Power
- XV-5, Loss of Normal Feedwater Flow
- XV-9, Startup of Inactive Loop
- XV-10, CVCS Malfunction - Reactivity
- XV-14, Inadvertent CVCS Malfunction - Inventory

Some or all of these nine transients can also be removed from the SEP topic list on the same basis as XV-11.

3) It is not clear on what basis Topic VI-7.D, "Long-Term Cooling Passive Failures", is being deleted. The rationale used in the letter would indicate its placement into the ongoing generic topic category.

4) The following topics should also be identified as completed:

Topic II-4.F, Settlement of Foundations and Buried Equipment, should be complete based on the lack of adverse effects of settlement in ten years of operation.

Topic III-3.C, ISI of Water Control Structures, should be deleted since Ginna has no water control structures.

Topic III-7.D, Containment Structural Integrity Test, is complete based on the testing of the Ginna containment which was performed in 1969.

Topic VI-7.A.1, ECCS Reevaluation to Account for Increased Vessel Head Temperature, is complete based on the NRC letter dated May 1, 1978.

ROCHESTER GAS AND ELECTRIC CORP.

SHEET NO.

DATE May 16, 1979

3

TO Director of Nuclear Reactor Regulation
Attention: Mr. Dennis L. Ziemann

Topic V-7.C.3, Effect of PWR Loop Isolation Valve Closure
During LOCA on ECCS Performance, should be deleted since
Ginna does not have loop isolation valves.

We appreciate the opportunity to provide comments on the
SEP topic resolutions.

Very truly yours,



L.D. White, Jr.