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Furnishes addl info re 781004 meeting to discuss SEP Topic 3-5.A, High Energy Line Breaks Inside Containment. Discusses pipe stress calculations, high energy line break outside containment, generic task A-2 & Asymmetric LOCA loads.

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LEON D. WHITE, JR.  
VICE PRESIDENT

TELEPHONE  
AREA CODE 716 546-2700

November 8, 1978

Director of Nuclear Reactor Regulation  
Attention: Dennis L. Ziemann, Chief  
Operating Reactors Branch No. 2  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: High Energy Line Breaks Inside Containment  
R.E. Ginna Nuclear Power Plant  
Docket No. 50-244

Dear Mr. Ziemann:

On October 4, 1978, members of our Staff and representatives of other utilities involved in the Systematic Evaluation Program (SEP) met with members of your Staff to discuss SEP Topic III-5.A, High Energy Line Breaks Inside Containment. The purpose was to clarify the actions that we and the NRC Staff would be taking to resolve this topic and to discuss tentative schedules. We have reviewed the minutes of the meeting, dated October 20, 1978, which were prepared by your Staff and wish to add additional information regarding the meeting.

The Staff indicated that submittal of the original pipe stress calculations for Ginna was voluntary and not necessary for resolution.

Some utilities within the SEP Owners' Group preferred the Mechanistic Approach. We however, encouraged the Staff to continue to develop other technical approaches which could be used to resolve the topic.

With respect to the Ginna high energy line break outside containment evaluation, modifications performed did follow from the calculations. An augmented inservice inspection provided a valuable technique for addressing that issue.

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ROCHESTER GAS AND ELECTRIC CORP.

SHEET NO.

DATE November 8, 1978

TO Director of Nuclear Reactor Regulation

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The Staff indicated that the phenomenon being evaluated in generic task A-2, Asymmetric LOCA Loads, are not considered to be part of this SEP topic. The work on generic task A-2 includes detailed definitions of the loads due to pipe breaks at critical locations and evaluation of the effect of these loads on major reactor coolant system components, their supports, and reactor internals including fuel. For SEP Topic III-5.A, the NRC staff expected that utilities will evaluate the effects of pipe whip, jet impingement and environmental effects on other systems and components important to safety.

Very truly yours,



L.D. White, Jr.