



**Commonwealth Edison**  
One First National Plaza, Chicago, Illinois  
Address Reply to: Post Office Box 767  
Chicago, Illinois 60690

November 1, 1979

Director of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: Dresden Station Units 2 and 3  
Quad-Cities Station Units 1 and 2  
Proposed Amendment to Facility  
Operating License Nos. DPR-19, 25,  
29 and 30, Associated with Expanding  
the Power/Flow Operating Region  
NRC Docket Nos. 50-237/249/254/265

- References (a): "Load Line Limit Analysis License  
Amendment Submittal for Dresden  
Nuclear Power Station Units 2 & 3,  
and Quad-Cities Nuclear Power Station  
Units 1 & 2 (NEDO-24167, December 1978)  
enclosed
- (b): "Millstone Unit 1 Loadline Limit  
Analysis," (NEDO-21285-1, November  
1977, Revision 1) submitted on NRC  
Docket No. 50-245, D. C. Switzer  
letter to D. L. Ziemann dated  
February 15, 1978
- (c): Letter from A. J. Levine (GE) to  
Z. R. Rosztoczy (NRC), "Additional  
Effects of Core Flow on ECCS LOCA  
Analysis," August 24, 1976
- (d): Letter from A. J. Levine (GE) to  
Z. R. Rosztoczy (NRC), "Additional  
Effects of Core Flow on ECCS LOCA  
Analysis," November 4, 1976

Dear Sir:

Pursuant to 10 CFR 50.59, Commonwealth Edison Company  
proposes to amend Appendix A to Operating Licenses DPR-19, 25,  
29 and 30 by incorporating the attached proposed changes into  
the Technical Specifications of Dresden Units 2 & 3 and Quad-  
Cities Units 1 & 2, respectively. The changes (Attachments 1  
through 4) will allow reactor power ascension to proceed along  
a modified power/flow line, which will facilitate compliance  
with procedures aimed at reducing fuel pellet-clad interaction.  
The added operational flexibility during power ascension is ex-  
pected to result in significantly increased generation with  
associated reduction in replacement power costs.

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The FSAR and previous reload submittals for Dresden 2 & 3 and Quad-Cities 1 & 2 assumed normal operation within the power/flow region below the nominal 100% flow control line (also referred to as "load line" or "power/flow line"). Figure 2.1-3 of the Technical Specifications has, therefore, been modified to reflect an expanded region of the flow control map. The new region is supported by detailed transient and safety analyses (Reference 1, enclosed) which also considered scram and rodblock lines with a reduced slope of 0.58.

A similar analysis was recently reviewed and approved for Millstone Unit 1 (Reference 2) which effectively constitutes a lead plant analysis for BWR 3 class reactors.

The attached documents have received on-site and off-site review and approval. Based on General Electric's analyses of the proposed changes for potential impact on Scram Reactivity, Transients, ASME Pressure Vessel Code Compliance, Rod Withdrawal Error, Thermal Hydraulics, and Stability (Reference 1) and previous analyses in the areas of Xenon Transients (Reference 2) and ECCS Performance (References 3 and 4), our safety evaluation concludes that such operation does not involve an unreviewed safety question as defined in 10 CFR 50.59.

Pursuant to 10 CFR 170, Commonwealth Edison has determined that the proposed changes are two Class III and two Class I Amendments. As such, we have enclosed a fee remittance in the amount of \$8,800.00.

Three (3) signed originals and fifty-seven (57) copies of this transmittal are provided for your use.

If you or your Staff have questions related to this proposal, please address them to this office.

Very truly yours,

*D L Peoples*

D. L. Peoples  
Director of Nuclear Licensing

RFJ;mae  
attachments (4)  
enclosure (1)

SUBSCRIBED and SWORN to  
before me this 1st day  
of November, 1979.

*Mary D. Evans*  
Notary Public