



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

January 18., 1979

BBS LTR #79-51

James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #78-068/03L-0, Docket # 050-237
is hereby submitted to your office in accordance with Dresden
Nuclear Power Station Technical Specification 6.6.B.2.(b),
conditions leading to operation in a degraded mode permitted
by a limiting condition for operation or plant shutdown required
by a limiting condition for operation.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:lg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

REGULATORY DOCKET FILE COPY

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ATTACHMENT TO LICENSEE EVENT REPORT 78-68/03L-0
COMMONWEALTH EDISON COMPANY (CWE)
DRESDEN UNIT-2 (ILDRS-2)
DOCKET #050-237

During normal operation with "A" recirc pump suction valve out-of-service in the open position, the breaker for the bypass valve around the recirc pump discharge valve was found thermally tripped after the valve opened. The recirc pump suction valves are presently normally kept OOS in the open position to eliminate the possibility of a spurious closure of the valve during a hypothetical loss-of-coolant accident. The recirc pump discharge bypass valves are normally open to prevent stagnation of the water in the bypass line, but the valve had been closed and reopened after an earlier recirc pump trip.

The low pressure coolant injection system would normally inject into the "B" recirc loop during a loss-of-coolant accident, and this loop was unaffected by this event. However, with this breaker tripped, had the low pressure coolant injection (LPCI) loop-select logic system been required to select the "A" recirc loop for LPCI injection, the bypass valve would not have shut upon signal and some of the LPCI flow could have been diverted backwards through the bypass valve, the recirc pump, and its suction valve, into the core annulus region.

Calculations indicate that the flow from the 16 inch diameter LPCI injection line which might have been diverted through the 4 inch diameter bypass valve line was minor. Approximately 2800 gpm could conservatively have been diverted. All four LPCI pumps were operable and each pump is rated at 6000 gpm. Technical Specifications require three LPCI pumps to be operable to deliver the design required 14,500 gpm LPCI flow.

Later testing could not identify any discrepancies with the operation of the bypass valve. Since the valve is infrequently used, the probable cause was dry packing which affected the opening torque and caused the thermal overload condition. Subsequent cycling probably lubricated the packing such that the problem did not reoccur. The valve stem will be lubricated during the next refueling outage.

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

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7	8	9	10		11		12		13		14	15	16	17	18	19	20									
17		EVENT YEAR						SEQUENTIAL REPORT NO.			O C C U R R E N C E CODE			REPORT TYPE			REVISION NO.									
LER/RO REPORT NUMBER		7	8		0		6	8		0		3		L			0									
21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER										
X	18	X	19	Z	20	Z	21	0	0	0	22	Y	23	N	24	N	25	C	6	6	5	26				

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

FACILITY STATUS (1) 5 (E) (28) % POWER (0) 9 (3) (29) OTHER STATUS (30) NA
 ACTIVITY CONTENT RELEASED OF RELEASE (1) 6 (Z) (33) (Z) (34) NA AMOUNT OF ACTIVITY (35) NA
 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) (0) 0 0 (37) (Z) (38) NA
 PERSONNEL INJURIES NUMBER DESCRIPTION (41) (0) 0 0 (40) NA
 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43) (1) 3 (Z) (42) NA
 METHOD OF DISCOVERY (A) (31) Operational Event DISCOVERY DESCRIPTION (32)
 LOCATION OF RELEASE (36) NA

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