

Tennessee Valley Authority, Post Office Box 2000, Decatur, Alabama 35609-2000

April 28, 2017

10 CFR 50.73

ATTN: Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555-0001

> Browns Ferry Nuclear Plant, Unit 1 Renewed Facility Operating License No. DPR-33 NRC Docket No. 50-259

Subject: Licensee Event Report 50-259/2016-004-01

Reference: Letter from TVA to NRC, "Licensee Event Report 50-259/2016-004-00," dated December 7, 2016

On December 7, 2016, the Tennessee Valley Authority (TVA) submitted Revision 0 to Licensee Event Report (LER) 50-259/2016-004-00 (Reference). This LER was based on a preliminary investigation, which has since been finalized.

The enclosed Licensee Event Report provides details of the incorrect tap settings for the Unit 1 480V Shutdown Board Transformers and associated 480V shutdown boards that resulted in inoperability of several transformers. This resulted in a condition prohibited by Technical Specifications (TS) and a Safety System Functional Failure. TVA is submitting this report in accordance with 10 CFR 50.73(a)(2)(i)(B), as any operation or condition which was prohibited by the plant's TS; 10 CFR 50.73(a)(2)(ii)(B), as any event or condition that resulted in the nuclear power plant being in an unanalyzed condition that significantly degraded plant safety; and 10 CFR 50.73(a)(2)(v)(A)/(B)/(C)/(D), as any event or condition that could have prevented the fulfillment of the safety function of structures or systems that are needed to shut down the reactor and maintain it in a safe shutdown condition, remove residual heat, control the release of radioactive material, or mitigate the consequences of an accident.

U.S. Nuclear Regulatory Commission Page 2 April 28, 2017

There are no new regulatory commitments contained in this letter. Should you have any questions concerning this submittal, please contact M. W. Oliver, Acting Nuclear Site Licensing Manager, at (256) 729-2636.

Respectfully, S. M. BORO Site Vige President

Enclosure: Licensee Event Report 50-259/2016-004-01 – Incorrect Tap Settings for the Unit 1 480V Shutdown Board Transformers Results in Inoperability of Associated 480V Shutdown Boards

cc (w/ Enclosure):

NRC Regional Administrator - Region II NRC Senior Resident Inspector - Browns Ferry Nuclear Plant

ENCLOSURE

Browns Ferry Nuclear Plant Unit 1

Licensee Event Report 50-259/2016-004-01

Incorrect Tap Settings for the Unit 1 480V Shutdown Board Transformers Results in Inoperability of Associated 480V Shutdown Boards

See Enclosed

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION				APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2018										
LICENSEE EVENT REPORT (LER)					Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects. Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.									
1. FACILITY NA	ME						2. DO	CKET NUMBER		3. PA	GE			
Browns Fei	rry Nucle	ear Plar	nt, Unit 1				0500	0259			1	OF 7		
4. TITLE														
Incorrect Tap Shutdown Boa		s for the	Unit 1 480	0V Shutd	own Bo	ard Tran	sforme	rs Results in I	noperabi	ility of	Associ	ated 48	80V	
5. EVENT D	ATE	6	LER NUMB	BER	7. F	REPORT D	ATE		OTHER F	ACILIT	TIES INVO	OLVED		
MONTH DAY	YEAR	YEAR	SEQUENTIA NUMBER	AL REV NO.	MONTH	DAY	YEAR	FACILITY NAME				DOCK	ET NUMBER	
10 08	2016	2016	- 004	- 01	04	28	2017	FACILITY NAME				DOCK	ET NUMBER	
9. OPERATING	MODE	11.	THIS REPOR	RT IS SUBI	MITTED F	PURSUAN	т то тн	E REQUIREMEN	TS OF 10	CFR §	: (Check	all that a	apply)	
		20	.2201(b)		20.2	2203(a)(3)(i)	50.73 (a	a)(2)(ii)(A)		50).73(a)(2)	(viii)(A)	
5		20	.2201(d)		20.2	2203(a)(3)(ii)	50.73 (a	a)(2)(ii)(B)		50	50.73(a)(2)(viii)(B)		
5		20	.2203(a)(1)		20.2	2203(a)(4)		50.73 (a	a)(2)(iii)		50).73(a)(2)	73(a)(2)(ix)(A)	
		20	.2203(a)(2)(i)		50.3	36(c)(1)(i)(A	\)	50.73(a)(2)(iv)(A) 50			0.73(a)(2)(x)			
10. POWER LE	VEL	20	.2203(a)(2)(ii)	50.3	36(c)(1)(ii)(ii)(A) 50.73(a)(2)(v)(A)				73.71(a)(4)			
		20	.2203(a)(2)(ii	i)	50.36(c)(2)			50.73(a)(2)(v)(B)			73.71(a)(5)			
		20	2203(a)(2)(iv	/)	50.46(a)(3)(ii)			50.73(a)(2)(v)(C)			73.77(a)(1)			
000		20.2203(a)(2)(v)			50.7	73(a)(2)(i)(A	A)	50.73(a	a)(2)(v)(D)		73	8.77(a)(2)	(i)	
		20	.2203(a)(2)(v	i)	S0.73(a)(2)(i)(B) 50.73(a)(2)(vii)			73	8.77(a)(2)	(ii)				
			1 20 1 21			73(a)(2)(i)(0	-		R Specify	in Abstra	ct below or in	n NRC Form	366A	
LICENSEE CONTACT				12. L	ICENSEE	E CONTAC	T FOR 1	HIS LER	TE	LEPHON	E NUMBER	(Include A	rea Code)	
Justin K. G	arner, L	-										29-795	5	
				MANU-	REPORTA	the second se		URE DESCRIBE	1		RT MANU-	RE	PORTABLE	
CAUSE	SYSTEM		ONENT FA	ACTURER	TO EPI		CAUSE	SYSTEM	COMPON	-+	FACTURE		TO EPIX	
N/A 14. SUPPLEME				N/A	N/A		N/A	N/A	N/A PECTED		N/A MONTH	DAY	N/A YEAR	
_			PECTED SUL	BMISSION	DATE)			SUE	MISSION	ŀ	N/A	N/A	N/A	
ABSTRACT (Limit	the second s								DATE		IN/A	IN/A	IN/A	
On October the 480V S would resul postulated I their associ an extender Specificatio are needed	r 8, 2016 hutdown lt in lowe loss of c iated 480 d outage ons, a co l to shut ces of an / degrad	6, during n Board er than r coolant a 0V shute e in 200 ndition t down th n accide	the performent transforment accident co down boar 7. This ev that could be reactor a ent, and a co	rmance of er TS1B, equired v bincident ds were ent is bei have pre and main	of preventives it was doubted oltages with deg determing reported vented tain it ir	ntive mail liscovere at the el graded v ned to be orted as a the fulfillin n a safe s	d that t ectrical oltage e inope a condi ment o shutdov	the of the transforme the transforme lly downstrear conditions. The rable since inst tion which was f the safety fun which was f the safety fun which condition, r clear plant bein	er was se n buses ransform stallation s prohibit nction of remove r	t on t and e ers T prior ted by struct esidu	he incol quipme S1A and to Unit to Unit the pla tures or al heat,	rrect tap nt durin d TS1B 1 resta ant's Te system or mitig	b. This ag a and rt from chnical hs that gate the	

NRC FORM 366A U.S. NUCLEAR REGULAT	ORY COMMISSION	APPROVED BY OMB: NO. 315	0-0104	EXPIRE	S: 10/31/2018	
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1. FACILITY NAME	2. DOC	KET NUMBER		3. LER NUMBER	2	
Browns Ferry Nuclear Plant, Unit 1	05000-259		YEAR	SEQUENTIAL NUMBER	REV NO.	
			2016	- 004	- 01	
NARRATIVE						

I. Plant Operating Conditions Before the Event

At the time of discovery, Browns Ferry Nuclear Plant (BFN), Unit 1, was in a planned refueling outage in Mode 5 at 0 percent rated thermal power.

II. Description of Event

A. Event Summary:

On October 8, 2016, during the performance of preventive maintenance of the transformer winding turns ratio test on the 480V Shutdown Board transformer 1-XFA-231-TS1B (TS1B)[XFMR], it was discovered that the transformer was set on the incorrect tap [TTC] (4160/480V instead of 3952/480V). After TS1B was found to be on the incorrect tap, additional inspections were performed on the 480V Shutdown Board transformer 1-XFA-231-TS1A (TS1A). TS1A was also discovered to be set on the incorrect tap (4160/480V instead of 3952/480V). The taps on 480V Shutdown Board transformers TS2A, TS2B, and THB were inspected and found to be correct. After discovery, TS1A was set on the correct tap setting on October 12, 2016, and TS1B was set on the correct tap setting on October 9, 2016.

The tap settings for the 480V Shutdown Board Transformers TS1A and TS1B were not set as specified on the applicable drawings and as established in the applicable calculation when installed. The consequences of the referenced transformer taps being improperly set would be lower than the minimum required voltages at the electrically downstream buses [BU] and equipment under degraded voltage conditions.

B. Status of structures, components, or systems that were inoperable at the start of the event and that contributed to the event:

There were no structures, systems, or components (SSCs) whose inoperability contributed to this event.

NRC FORM 366A (06-2016)) U.S. NUCL LICENSEE E CONTIN	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2018 Estimated burden per response to comply with this mandatory collection request: 80 hours. Reporte lessons learned are incorporated into the licensing process and fed back to industry. Sen comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T- F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail Infocollects. Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a mean used to impose an information collection does not display a currently valid OMB control number, th NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.					rs. Reported Istry. Send Branch (T-5 y e-mail to latory Affairs, If a means number, the		
1. FACILITY NAME		2. DOC	KET NUMBER		-		R	
Browns Ferry Nuclear Plant, U	Browns Ferry Nuclear Plant, Unit 1			year 2016	SE	OUENTIAL NUMBER	-	rev no. 01
NARRATIVE	an shaki dari ku da k		ang	an a		Ling of demokration of the second second		
C. Dates and approx	imate times	s of occurrence	es:					
November 18, 2004	TS1A wind	ing tap incorrec	tly set					
December 15, 2004	TS1B wind	TS1B winding tap incorrectly set						
May 21, 2007	Unit 1 ente	Unit 1 entered Mode 2						
October 8, 2016 0049 CST	Incorrect ta	Incorrect tap setting for transformer TS1B discovered						
October 9, 2016	TS1B set t	TS1B set to the correct tap						
October 10, 2016 2038 CST	Incorrect ta	Incorrect tap setting for transformer TS1A discovered						
October 12, 2016	TS1A set to	TS1A set to the correct tap						
December 7, 2016	LER 50-25	LER 50-259/2016-004-00 was submitted.						
December 21, 2016 1555 CST	8-hour Event Notification System (ENS) notification for an unanalyzed condition. (EN 52452)							
D. Manufacturer and during the event:		nber (or other i	dentification) of eac	h com	pone	ent that fa	ilec	-

No component failures were identified that occurred during the event.

E. Other systems or secondary functions affected:

There were no other systems or secondary functions affected.

F. Method of discovery of each component or system failure or procedural error:

This condition was discovered during preventive maintenance. It was determined that the TS1A and TS1B tap settings did not agree with the applicable drawing and calculation.

G. The failure mode, mechanism, and effect of each failed component, if known:

There were no components that failed during the event.

H. Operator actions:

There were no operator actions performed in response to this event.

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1. FACILITY NAME	2. DOC	KET NUMBER		3. LER NUMBER	8	
Browns Ferry Nuclear Plant, Unit 1	05000-259		YEAR	SEQUENTIAL NUMBER	REV NO.	
			2016	- 004	- 01	
NARRATIVE						

I. Automatically and manually initiated safety system responses:

There were no safety system responses initiated in response to this event.

III. Cause of the event

A. The cause of each component or system failure or personnel error, if known:

The installation Work Orders (WOs) were planned incorrectly in that the drawing which specified the transformer tap settings on TS1A and TS1B were listed in the implementation WOs as a Category 3 drawing. This represents both divisions and opposite trains. As a result during the installation of these transformers, there was not a step in the WOs to set the taps on the correct tap. Since there was no specific step in the WOs to change the settings for TS1A and TS1B to match the applicable drawing, the transformers were left on the same tap setting (4160/480V) as they were shipped from the factory.

B. The cause(s) and circumstances for each human performance related root cause:

There were no human performance related root causes.

IV. Analysis of the event:

The Tennessee Valley Authority (TVA) is submitting this report in accordance with 10 CFR 50.73(a)(2)(i)(B), as any operation or condition which was prohibited by the plant's Technical Specifications; 10 CFR 50.73(a)(2)(v)(A)/(B)/(C)/(D), as any event or condition that could have prevented the fulfillment of the safety function of structures or systems that are needed to shut down the reactor and maintain it in a safe shutdown condition, remove residual heat, control the release of radioactive material, or mitigate the consequences of an accident; and 10 CFR 50.73(a)(2)(ii)(B), as any event or condition that resulted in the nuclear plant being in an unanalyzed condition that significantly degrades plant safety.

Technical Specification (TS) 3.8.7 requires that the Unit 1 480V Shutdown Boards and the Unit 1 Reactor Motor Operated Valve (RMOV) Boards 1A and 1B be Operable. The required action for Condition B for the 480V shutdown board is to restore the board to Operable status within 8 hours and 12 days from discovery of the failure to meet the Limiting Condition for Operation (LCO). If the required action and associated completion time of Condition B is not met, the Unit must be in Mode 3 within 12 hours and Mode 4 within 36 hours. TS Bases Section B 3.8.7 states the Operability of the AC and DC electrical power distribution subsystems is consistent with the initial assumptions of the accident analyses and is based upon meeting the design basis of the unit. This includes maintaining distribution systems Operable during accident conditions in the event of:

- a) An assumed loss of all offsite power or all onsite AC electrical power sources; and
- b) A postulated worst case single failure.

The Specified Safety Function affected by the condition is the ability of the Unit 1 480V Shutdown Boards 1A and 1B to receive proper voltage from the 4kV Shutdown Board supply via transformers TS1A and TS1B, respectively. The transformer tap changer settings for TS1A and

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1. FACILITY NAME	2 000			3. LER NUMBER				
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Browns Ferry Nuclear Plant, Unit 1	05000-259		2016	SEQUENTIAL NUMBER - 004	NO. - 01			
NARRATIVE								
TS1B ensure proper voltage is available and DC buses ensures that the reading conditions for critical system loads of powered by 480V SD BD 1A, with 480V RMOV Boards 1A, 1C, and these boards coming from 480V S	nd accident sce quired power is connected to the backup power t 1E are powered	narios. Proper voltag readily available for n ese buses. 480V RM to these boards comin	e availa notive a OV Boa ng from	ability on the A0 s well as contro ards 1A and 1D 480V SD BD 1	ol are			
An electrical calculation software evaluation (ETAP) performed with the TS1A and TS1B taps set on the incorrect setting of 4160/480V concluded that, with a degraded voltage condition of greater than 3900 but less than 3983V at the 4kV Shutdown Board, the voltage at 480V Shutdown Board 1A would be 431V and the voltage of the 480V Shutdown Board 1B would be 433V. This is less than the required 440V specified in the associated calculation, which results in a lower voltage on the downstream RMOV boards. The 480V RMOV board bus voltages would be as follows: 480V RMOV Board 1A would be at 430V, 480V RMOV Board 1B would be at 431V, 480V RMOV Board 1C would be at 432V, and 480V Control Bay Vent Board A would be at 428V with 432V being the minimum required bus voltage at the boards specified in the applicable calculation.								
The required 90% of rated voltage RMOV Boards according to the ET transformer taps being improperly electrically downstream buses and the Reactor Building Closed Coolin 1-FCV-70-47, would not meet the these conditions. The failure of the	TAP calculation. set would be lo d equipment unc ng Water Prima requirements to	The consequences wer than the minimun ler degraded voltage ry Containment Disch assure that its valve	of the re n require conditio arge Or motor w	eferenced ed voltages at t ons. Specificall utlet valve, vould start unde	у,			
TVA determined an unanalyzed co Boards Transformers TS1A and T this issue, no alternative source of this system to perform its required Unit 1 entered the mode of applica TS1A and TS1B were set on the c BDs for longer than allowed by pla	S1B. Because power was ava safety function ability, and the c correct tap in Oc	both 480V SD BDs 1/ ilable to these boards . These BDs were inc ondition was not corre	A and 1 s, result operable ected ur	B were affected ing in the failure from 2007 wh ntil transformer	d by e of ien s			
With a qualified offsite power sour- considered an abnormal operating load tap changer attempts to take licensing and/or design basis requ Drop Calculation" is that connected protective device actuation as a re- reset value of 3983V is degraded, at the degraded voltage level did r (EDG)[DG] associated with the 4k power system is designed to be in related equipment during a LOCA,	condition that i action to increa irement as per d loads not bec sult of operating and operation i not recover with V Shutdown Bo dependent of th	s expected to be a sh se the voltage to above the "4.16kV and 480V ome damaged or beck g at a degraded voltage s expected to be a sh in 5.6 seconds, the Er ard would start and ca be diesel generators to	ort term ve the re ' Bus Lo ome un ge. Any ort dura mergen arry the o supply	a duration until eset setting. The bad and Voltage available due to voltage below ation. If the volt cy Diesel Gene loads. The off power to safe	he e o the tage erator site ty			

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Browns Ferry Nuclear Plant, Unit 1	05000-259		year 2016	SEQUENTIAL NUMBER - 004	REV NO. - 01		
NARRATIVE							
have been able to perform their s electrical power for the systems r condition after an abnormal oper Therefore, the offsite power sour 10 CFR Part 50, Appendix A, Ge unanalyzed condition that signific	equired to shut d ational transient o ce would not be o neral Design Crit	lown the reactor and r or a postulated Desigr considered Operable a eria (GDC) 17, and th	maintair n Basis as per t	n it in a safe Accident (OBA he requirement). ts of		
V. Assessment of Safety Consequence	uences						
A Probabilistic Risk Assessment inoperability and found that there during the time period that the bo and safety of the public or plant p	was a negligible ards were inope	increase in risk. TVA rable, there was no sig	has co	ncluded that,	alth		
A. Availability of systems or c as the components and sys			ed the	same functior	n		
No components or systems fa	ailed due to this e	event.					
B. For events that occurred w related systems or compon		was shut down, ava	ilability	of safety-			
This event occurred while the operability was required when operability was assessed.							
these boards coming from 48 powered by 480V SD BD 1B, BD 1A. Because both 480V	480V RMOV Boards 1A and 1D are powered by 480V SD BD 1A, with backup power to these boards coming from 480V SD BD 1B. 480V RMOV Boards 1A, 1C, and 1E are powered by 480V SD BD 1B, with backup power to these boards coming from 480V SD BD 1A. Because both 480V SD BDs 1A and 1B were affected by this issue, no alternative source of power was available to these boards.						
480V Control Bay Vent BD A board coming from 480V Cor was available to this board.							
C. For failure that rendered a t time from the discovery of					ed		
The time frame for the condit for TS1B for Modes 1, 2, and was recordable beginning in was corrected when transform transformer TS1B was set or	3 until the condit 2007 when Unit 7 mer TS1A was se	tion was corrected. H 1 entered the mode of at on the correct tap of	owever applica	, the inoperabil ability. The con	ity dition		
VI. Corrective Actions:							
NRC FORM 366A (06-2016)				Page 6			

NRC FORM 366A U.S. NUCLEAR REGULAT	ORY COMMISSION	APPROVED BY OMB: NO. 315	0-0104	EXPIRE	S: 10/31/2018			
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Browns Ferry Nuclear Plant, Unit 1	05000-259		YEAR 2016	SEQUENTIAL NUMBER	REV NO. - 01			
NARRATIVE			2010	004	01			
Corrective Actions (CA) are being Condition Report (CR) 1221273.	managed by T∖	/A's Corrective Action	Progra	ım under				
The CAs described below address	The CAs described below address this condition:							
 The transformer taps for transformers TS1A and TS1B have been set in accordance with the applicable drawing, which specify the correct tap settings. Verify that the 4160/480V transformer tap settings are correct in accordance with the applicable drawings. Revise the classification for the applicable drawing from Category 3 to Category 2. VII. Previous Similar Events: A review for CRs was performed for similar events. The following instance was discovered in regards to transformer tap settings. 								
 CR 03-001022-000 - Two Unit 3 transformer tap settings, TUSS 3A and THB, were found to disagree with the value used in the ETAP database, the applicable calculation, and drawing. The immediate corrective action was to perform calculations to verify that system voltages were acceptable. The corrective action for the CR was to revise the applicable calculation to support the correct tap settings for TUSS 3A and THB. 								
The actions for CR 03-001022-000) would not hav	e prevented this cond	ition.					
VIII. Additional Information:								
The offsite power system is designed to be independent of the EDGs to supply power to safety related equipment during a LOCA; therefore, the 480V Shutdown Boards were not considered to have been able to perform their specified safety function. In conclusion, this event is considered to be a Safety System Functional Failure in accordance with NUREG-1022.								
IX. COMMITMENTS								

There are no new commitments.