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MAR 0 3 2017

U. S. Nuclear Regulatory Commission

Attn: Document Control Desk Washington, DC 20555-0001

10 CFR 50.73

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 50-387/2016-018-01 UNIT 1 LICENSE NO. NPF-14

PLA-7557

Docket No. 50-387

Attached is a supplement to Licensee Event Report (LER) 50-387/2016-018. The LER reported an event involving inoperability of Reactor Core Isolation Cooling (RCIC). This event was determined to be reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

There were no actual consequences to the health and safety of the public as a result of this event.

This letter contains no new regulatory commitments.

B. Berryman

Attachment: LER 50-387/2016-018-01

Copy: NRC Region I

Ms. L. H. Micewski, NRC Sr. Resident Inspector

Ms. T. E. Hood, NRC Project Manager

Mr. M. Shields, PA DEP/BRP

NRC FORM 366 (06-2016)

U.S. NUCLEAR REGULATORY COMMISSION | APPROVED BY OMB: NO. 3150-0104

EXPIRES: 10/31/2018



LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

(See NUREG-1022, R.3 for instruction and guidance for completing this form

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control

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FACILITY NAME Susquehanna Steam Electric Station Unit 1								2. DOCKET NUMBER 05000387				3. PAGE	3. PAGE 1 of 3				
4. TITLE	Inop	erability o	f RCIC	Due to	an Oil	Leak	Cause	ed by a	n Unide	ntifie	ed Gasket	Issue					
5. E	VENT	DATE	6. I	LER NUM	IBER		7. RE	PORT	ATE			8. OTHER FAC	CILITIES IN	VOLVE	D		
MONTH	DAY	YEAR	YEAR	SEQUEN	110		монтн	DAY	YEAR	FACILITY NAME				DOCKET NUMBER 05000			
04	22	2016	2016	018	C)1	03	03	2017	FAC	CILITY NAME		DOCKET NUMBER 05000				
9. OP	9. OPERATING MODE 11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)																
1 20.2201(b) 20.2203(a)(3)(i)							☐ 50.73(a)(2)(ii)(A)			☐ 50.73(a)(2)(viii)(A)							
1			☐ 20.2201(d)				20.2203(☐ 50.73(a)(2)(ii)(B)			□ 50.7	☐ 50.73(a)(2)(viii)(B)				
			20.2203(a)(1)				20.2203(☐ 50.73(a)(2)(iii)			□ 50.7	☐ 50.73(a)(2)(ix)(A)				
			20.2203(a)(2)(i)				50.36(c)(1)(i)(A)			☐ 50.73(a)(2)(iv)(A)			□ 50.7	☐ 50.73(a)(2)(x)			
10. P	OWER	LEVEL	20.2203(a)(2)(ii) [☐ 50.36(c)(1)(ii)(A)			☐ 50.73(a)(2)(v)(A)			73.7	☐ 73.71(a)(4)			
			☐ 20.2203(a)(2)(iii) [☐ 50.36(c)(2)			☐ 50.73(a)(2)(v)(B)			73.7	☐ 73.71(a)(5)			
010			20.2203(a)(2)(iv)				☐ 50.46(a)(3)(ii)			☐ 50.73(a)(2)(v)(C)			73.7	☐ 73.77(a)(1)			
			20.2203(a)(2)(v)				☐ 50.73(a)(2)(i)(A)			☐ 50.73(a)(2)(v)(D)			73.7	☐ 73.77(a)(2)(i)			
			☐ 20.2203(a)(2)(vi)			⊠:	50.73(a)(2)(i)(B)			☐ 50.73(a)(2)(vii)			73.7	☐ 73.77(a)(2)(ii)			
						☐ 50.73(a)(2)(i)(C) ☐ OTHER Specify in Abstract be				pecify in Abstract belo	low or in NRC Form 366A						
						12.	LICENS	EE CON	TACT FO	R TH	IS LER						
C. E. I		es, Jr., Se	nior En	gineer	- Nucle	ar R	egulato	ory Affa	airs				E NUMBER (Inc 542-3089	clude Area	(Code)		
			13.	COMPLE	TE ONE	LINEF	OR EACH	H COMP	ONENT FAI	LURE	DESCRIBED	IN THIS REPOR	Γ				
CAUS	E	SYSTEM	COME	PONENT	NT MANU- FACTURER		REPORTABLE TO EPIX		CAUSE		SYSTEM	COMPONENT	MANU- FACTURE		REPORTABLE TO EPIX		
X BN		Ga	asket	Dresser		Y											
14. SUPI	PLEME	NTAL REP	ORT EXF	ECTED							15. EX	PECTED	MONTH	DAY		YEAR	
YES (If yes, complete 15. EXPECTED SUBMISSION DATE)							⊠ NO		SUBMISSION DATE								
ABSTRACT	(Limit to	o 1400 spaces	, i.e., appro	ximately 1	5 single-s _l	paced t	ypewritten	lines)									
On An	ril 22	2016 at	appro	ximate	lv 14.	00 :	a two t	o thre	e drop	ner	second le	eak from the	Reacto	r Cor	e Is	olation	

Cooling (RCIC) Turbine Lube Oil Filter, 1F212B, was identified which was subsequently determined to make RCIC inoperable. Unit 1 had entered Mode 1 at approximately 11:25 on April 22, 2016, and RCIC was considered to have been inoperable prior to the transition to Mode 1. As a result, the condition was considered a violation of Technical Specification (TS) 3.0.4 and reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

The cause of the leak was determined to be an unidentified gasket issue. Corrective actions included repair of the oil leak and replacement of both filter elements and all gaskets.

There were no actual consequences to the health and safety of the public as a result of this event.

EXPIRES: 10/31/2018



CONTINUATION SHEET

(See NUREG-1022, R.3 for instruction and guidance for completing this form http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr1022/r3/)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME	2. DOCKET NUMBER	3. LER NUMBER			
Susquehanna Steam Electric Station	05000-387	YEAR	SEQUENTIAL NUMBER	REV NO.	
Unit 1		2016	- 018	- 01	

NARRATIVE

CONDITIONS PRIOR TO EVENT

Unit 1 – Mode 1, approximately 10 percent Rated Thermal Power

Unit 2 - Mode 1, 100 percent Rated Thermal Power

There were no other structures, systems, or components that were inoperable at the start of the event that contributed to the event.

EVENT DESCRIPTION

On April 22, 2016 at approximately 11:25, Unit 1 entered Mode 1.

On April 22, 2016 at approximately 12:09, Technical Specification (TS) 3.5.3 was entered and the RCIC [EIIS System Identifier: BN] Quarterly Flow Surveillance was performed with reactor pressure vessel (RPV) pressure at approximately 930 psig.

On April 22, 2016 at approximately 14:00, a two to three drops per second leak on the 1F212B, RCIC Turbine Lube Oil Filter [EIIS Component Identifier: FLT], was identified.

On April 22, 2016 at approximately 16:01, the main turbine [EIIS System Identifier: TA] was tripped due to a seal oil leak on the collector end of the generator [EIIS System Identifier: TB].

On April 22, 2016 at approximately 20:57, the reactor entered Mode 2.

On April 23, 2016 at approximately 00:46, the reactor entered Mode 3. RPV Pressure was below 150 psig at approximately 03:00.

On April 23, 2016 at approximately 03:55, an operability review concluded that RCIC was inoperable since there was no guarantee that RCIC would meet its mission time with the identified leak.

On April 23, 2016 at approximately 06:54, the reactor entered Mode 4.

On April 30, 2016, both filter elements and all gaskets were replaced. These actions corrected the leak and RCIC was subsequently declared operable.

The leakage identified on April 22, 2016 was considered sufficient to require declaring RCIC inoperable. RCIC was also considered to have been inoperable prior to the transition to Mode 1. As a result, the condition was considered a violation of Technical Specification (TS) 3.0.4 and reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

CAUSE OF EVENT

The two items replaced to correct the leak were the gaskets and the filters. The filters would not cause an oil leak in the system and no deficiencies within the filters were identified. The gaskets were disposed of prior to the evaluation, and were the only difference that could have potentially resulted in the oil leak. Based on this available information, the direct cause of the leak was determined to be an unidentified gasket issue. A manufacturing defect or poor seating caused by pressurization are two of the potential gasket issues that could have caused the leak; however, a definitive apparent cause could not be determined.

NRC FORM 366A (06-2016) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104

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LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

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Susquehanna Steam Electric Station	05000-387	YEAR	SEQUENTIAL NUMBER	REV NO.		
Unit 1		2016	- 018	- 01		

ANALYSIS/SAFETY SIGNIFICANCE

Over time, a three drops per second leak would have caused the RCIC system to stop injecting into the vessel. Engineering analysis determined that RCIC would have operated sufficiently long such that the transient analysis in FSAR Section 15.2.7, "Loss of Feedwater Flow" and the safe shutdown function described in FSAR Section 7.4 would have been unaffected.

CORRECTIVE ACTIONS

Key corrective actions include the following:

- 1. The oil leak was repaired.
- 2. The Dresser gaskets that were installed at the time of the leak were replaced with ARGO 330 gaskets. The ARGO 330 gaskets are preferred for this application, and Susquehanna plans to use of the ARGO 330 gaskets going forward.

COMPONENT FAILURE INFORMATION

Information on the failed gasket is as follows:

Manufacturer: Dresser

Model #: CUNO Buna-N 1BD

PREVIOUS SIMILAR EVENTS

There were two RCIC oil filter leaks at Susquehanna in 1985; however, the cause of those events was identified to be sharp edges on the filter canister. Corrective action to replace the filter housing and elements has prevented recurrence of this issue.

Internal searches identified no degraded trends associated with Dresser gaskets.